



Omaha Public Power District
444 South 16th Street Mall
Omaha NE 68102-2247

April 15, 1997
LIC-97-0054

U. S. Nuclear Regulatory Commission
Attn: Document Control Desk
Mail Station P1-137
Washington, D.C. 20555

Reference: Docket No. 50-285

SUBJECT: March 1997 Monthly Operating Report (MOR)

Enclosed please find the March MOR for Fort Calhoun Station (FCS) Unit No. 1
as required by FCS Technical Specification 5.9.1.

If you should have any questions, please contact me.

Sincerely,

S. K. Gambhir
Division Manager -
Engineering & Operations Support

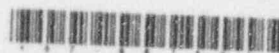
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Enclosures

c: Winston & Strawn
E. W. Merschoff, NRC Regional Administrator, Region IV
L. R. Wharton, NRC Project Manager
W. C. Walker, NRC Senior Resident Inspector
R. J. Simon, Westinghouse
INPO Records Center

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OMAHA PUBLIC POWER DISTRICT
Fort Calhoun Station Unit No. 1

March 1997
Monthly Operating Report

1. OPERATIONS SUMMARY

The Fort Calhoun Station (FCS) operated at a nominal 100% power level between March 1, 1997 and March 22, 1997. On March 22, 1997 at 2002 hours, FCS began a power reduction in order to complete scheduled quarterly main turbine control valve testing. The 95% power level was reached at 2236 hours on March 22, 1997. Testing was completed satisfactorily and a power ascension began at 0200 hours on March 23, 1997. The station reached 100% power at 0910 hours on March 23, 1997 and remained there through the end of the month.

On March 12, 1997, FCS made a 1-hour report, concerning a condition which met the criteria of 10 CFR 50.72(b)(2)(I). The condition was a follow-up to LER 96-005, which involved the design of the FCS Auxiliary Building for tornado venting. The FCS Updated Safety Analysis Report (USAR) does not indicate whether the Control and Cable Spreading Rooms were evaluated for the stresses imposed by the repressurization phase of a tornado. However, a recently completed safety analysis for operability (SAO-96-01, Rev. 2) determined that the Control Room and Cable Spreading Room envelopes remain operable and within the design basis following a design basis tornado.

During the period of March 17, 1997 through March 26, 1997 a self-assessment of the Operations Department was completed. Personnel from FCS and several other utilities were utilized to conduct the assessment, which was supported by the Utilities Service Alliance (USA).

2. SAFETY VALVES OR PORV CHALLENGES OR FAILURES WHICH OCCURRED

During the month of March 1997, no power operated relief valves (PORV) or primary system safety valve challenges or failures occurred.

3. RESULTS OF LEAK RATE TESTS

The reactor coolant system (RCS) leak rate was steady throughout the month continuing the trend of minimal RCS leakage following the 1996 refueling outage. March daily leak rates were constant at approximately 0.1 to 0.2 gpm.

4. CHANGES, TESTS AND EXPERIMENTS REQUIRING NUCLEAR REGULATORY COMMISSION AUTHORIZATION PURSUANT TO 10CFR50.59

Amendment No.	Description
181	This amendment revises Section 5.2 of the Technical Specifications to relocate controls for working hours to the Updated Safety Analysis Report.

5. SIGNIFICANT SAFETY RELATED MAINTENANCE FOR THE MONTH OF March 1997

- Maintenance on Component Cooling Water Heat Exchanger AC-1D was performed which consisted of plugging a tube, repairing several small leaks by rolling tubes, and tube cleaning.
- Boric Acid Storage Tank (BAST) CH-11B, Bypass Valve CH-462 was disassembled and parts were discovered missing. The valve was rebuilt and the manway was removed from BASTs CH-11A & CH-11B to search for the missing parts. The outlet valve (CH-114) on BAST CH-11B, was also searched for the missing parts. Four (4) pieces of the broken valve gate were found in the bottom of BAST CH-11B.
- Seal welded coupling threads on FO-5A-2 (Transfer Pump FO-4A-2; Suction Strainer).
- Seal welded coupling threads on FO-5B-2 (Transfer Pump FO-4B-2; Suction Strainer).
- Replaced thermocouple on Containment Stack Radiation Monitor RM-052.
- Replaced relays 74-2A/D2, 94-AS/D2, 94-BS/D2 on Diesel Generator No. 2 per ECN-FC-95-374.
- Replaced tubing and fittings on HCV-400E (VA-1A Cooling Coil, Raw Water Inlet Valve) to stop air leakage.

LIC-97-0054

Enclosure

Page 3

6. OPERATING DATA REPORT

Attachment I

7. AVERAGE DAILY UNIT POWER LEVEL

Attachment II

8. UNIT SHUTDOWNS AND POWER REDUCTIONS

Attachment III

9. REFUELING INFORMATION, FORT CALHOUN STATION UNIT NO. 1

Attachment IV

ATTACHMENT I
OPERATING DATA REPORT

DOCKET NO.	50-285
UNIT	FORT CALHOUN STATION
DATE	APRIL 09, 1997
COMPLETED BY	M. L. EDWARDS
TELEPHONE	402-533-6929

OPERATING STATUS

1. Unit Name: FORT CALHOUN STATION
2. Reporting Period: MARCH 1997

NOTES

3. Licensed Thermal Power (MWt): 1500
4. Nameplate Rating (Gross MWe): 502
5. Design Elec. Rating (Net MWe): 478
6. Max. Dep. Capacity (Gross MWe): 502
7. Max. Dep. Capacity (Net MWe): 478

8. If changes occur in Capacity Ratings (3 through 7) since last report, give reasons:
N/A

9. Power Level to which restricted, if any (Net MWe): N/A

10. Reasons for restrictions, if any:
N/A

	THIS MONTH	YR-TO-DATE	CUMULATIVE
11. Hours in Reporting Period.....	744.0	2160.0	206138.0
12. Number of Hours Reactor was Critical	744.0	2160.0	162851.6
13. Reactor Reserve Shutdown Hours.....	.0	.0	1309.5
14. Hours Generator On-line.....	744.0	2160.0	161027.8
15. Unit Reserve Shutdown Hours.....	.0	.0	.0
16. Gross Thermal Energy Generated (MWH)	1114132.6	3202857.7	215706737.7
17. Gross Elec. Energy Generated (MWH)..	379728.0	1091632.0	71309865.1
18. Net Elec. Energy Generated (MWH)....	362923.4	1042584.6	68028705.9
19. Unit Service Factor.....	100.0	100.0	78.1
20. Unit Availability Factor.....	100.0	100.0	78.1
21. Unit Capacity Factor (using MDC Net)	102.1	101.0	71.2
22. Unit Capacity Factor (using DER Net)	102.1	101.0	69.7
23. Unit Forced Outage Rate.....	.0	.0	4.0

24. Shutdowns scheduled over next 6 months (type, date, and duration of each):
N/A

25. If shut down at end of report period, estimated date of startup: _____

26. Units in test status (prior to comm. oper.): Forecast Achieved

INITIAL CRITICALITY
INITIAL ELECTRICITY
COMMERCIAL OPERATION

N/A

ATTACHMENT II
AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO. 50-285
UNIT FORT CALHOUN STATION
DATE APRIL 09 1997
COMPLETED BY M. L. EDWARDS
TELEPHONE 402-533-6929

MONTH MARCH 1997

DAY AVERAGE DAILY POWER LEVEL
(MWe-Net)

1	488
2	489
3	489
4	488
5	488
6	488
7	489
8	489
9	489
10	489
11	488
12	488
13	488
14	488
15	488
16	488

DAY AVERAGE DAILY POWER LEVEL
(MWe-Net)

17	488
18	488
19	488
20	488
21	487
22	484
23	481
24	488
25	488
26	488
27	488
28	488
29	488
30	488
31	488

INSTRUCTIONS

On this form, list the average daily unit power level in MWe-Net for each day in the reporting month. Compute to the nearest whole megawatt.

ATTACHMENT III
UNIT SHUTDOWNS AND POWER REDUCTIONS

DOCKET NO. 50-285
UNIT NAME Fort Calhoun St.
DATE April 3, 1997
COMPLETED BY M. L. Edwards
TELEPHONE (402) 533-6929

REPORT MONTH March 1997

No.	Date	Type ¹	Duration (Hours)	Reason ²	Method of Shutting Down Reactor ³	Licensee Event Report No.	System Code ⁴	Component Code ⁵	Cause & Corrective Action to Prevent Recurrence
None									

1
F: Forced
S: Scheduled

2
Reason:
A-Equipment Failure (Explain)
B-Maintenance or Test
C-Refueling
D-Regulatory Restriction
E-Operator Training & License Examination
F-Administrative
H-Other (Explain)

3
Method:
1-Manual
2-Manual Scram
3-Automatic Scram
4-Other (Explain)

4
Exhibit F - Instructions
for Preparation of Data
Entry Sheets for Licensee
Event Report (LER) File (NUREG-0161)

5
Exhibit H - Same Source

Attachment IV
Refueling Information
Fort Calhoun Station Unit No. 1

Report for the month ending: March 31, 1997	
1. Scheduled date for next refueling shutdown.	March 21, 1998
2. Scheduled date for restart following refueling.	May 2, 1998
3. Will refueling or resumption of operations thereafter require a technical specification change or other license amendment?	No
a. If answer is yes, what, in general, will these be?	N/A
b. If answer is no, has the reload fuel design and core configuration been reviewed by your Plant Safety Review Committee to determine whether any unreviewed safety questions are associated with the core reload?	No
c. If no such review has taken place, when is it scheduled?	Prior to May 2, 1998
4. Scheduled date(s) for submitting proposed licensing action and support information.	No submittal required
5. Important licensing considerations associated with refueling, e.g., new or different fuel design or supplier, unreviewed design or performance analysis methods, significant changes in fuel design, new operating procedures.	None
6. The number of fuel assemblies: a) in the core b) in the spent fuel pool c) spent fuel pool storage capacity	133 Assemblies 662 Assemblies 1083 Assemblies
7. The projected date of the last refueling that can be discharged to the spent fuel pool assuming the present licensed capacity.	2007 Outage
Prepared by: <u>Mary J. Davis</u> Date: <u>4/3/97</u>	