



April 14, 1997

United States Nuclear Regulatory Commission
Attention: Document Control Desk
Washington, D.C. 20555-0001

Subject: Braidwood Nuclear Power Station, Units 1 and 2,
ASME Section XI Inservice Testing Program
Leakage Testing of Containment Isolation Valves
Facility Operating Licenses NPF-72 and NPF-77,
NRC Docket Nos. 50-456 and 50-457

- References:
1. R. J. Barrett (NRC) letter to T. J. Kovack (ComEd),
transmitting the SER for the IST program for pumps
and valves for Braidwood Station, dated October 15, 1991
 2. R. R. Assa (NRC) letter to D. L. Farrar (ComEd),
transmitting the SER for Appendix J Option B approval,
for Byron and Braidwood Stations, dated April 4, 1996

Commonwealth Edison Company's (ComEd's) Braidwood Nuclear Power Station, Units 1 and 2 (Braidwood), performs leakage testing of containment isolation valves in accordance with Section XI of the 1983 Edition of the American Society of Mechanical Engineers Boiler and Pressure Vessel Code (ASME Code), through the Summer 1983 Addenda, as required by Title 10, Code of Federal Regulations, Part 50, Section 55a, Paragraph f, Subparagraph 4 [10 CFR 50.55a(f)(4)], except where alternatives have been authorized or relief has been requested and granted by the United States Nuclear Regulatory Commission (USNRC). Reference 1 provided the approval to utilize the Appendix J leakage tests to satisfy ASME Section XI Inservice Testing requirements in accordance with relief request VR-1 of the Braidwood Inservice Testing Program. Braidwood has utilized the Appendix J leakage tests to satisfy the leak test requirements of containment isolation valves for both the Appendix J and the ASME Section XI Inservice Testing Program requirements.

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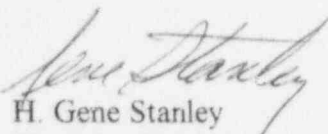
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Reference 2 provided the approval to utilize 10 CFR Part 50 Appendix J, Option B in accordance with Amendment 73 of the Braidwood Technical Specifications. Prior to Option B, the ASME Section XI frequency was at least once every 2 years. Appendix J Option B provides for scheduling based on past performance. In accordance with Nuclear Energy Institute (NEI) 94-01, Revision 0, July 25, 1995, valves which have had the last two Local Leak Rate Testing (LLRT) surveillances pass the Appendix J Type C leakage test acceptance criteria will have their frequency extended to once every 5 years. The leakage testing frequency for the ASME Section XI Inservice Testing Program containment isolation valves will be in accordance with 10 CFR 50 Appendix J, Option B commencing with Braidwood's current refueling outage which began March 21, 1997.

Please address any comments or questions regarding this matter to Ms. Patricia A. Boyle, at (815) 458-2801 extension 2519.

Sincerely,



H. Gene Stanley
Site Vice President
Braidwood Nuclear Generating Station

cc: A. B. Beach, Regional Administrator - RIII
G. F. Dick, Braidwood Project Manager - NRR
C. Phillips, Senior Resident Inspector - Braidwood
Office of Nuclear Facility Safety - IDNS