



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

April 10, 1997

Mr. David Modeen
Manager, Engineering
Nuclear Energy Institute
1776 I Street, NW
Suite 400
Washington, DC 20006-3708

Dear Mr. Modeen: *Dave*

This letter is in reference to a proposed protocol for the Electric Power Research Institute (EPRI) Steam Generator Degradation Specific Management (SGDSM) Database. The database is intended to provide a framework that will allow utilities access to up-to-date correlations to support steam generator tube alternate repair criteria. The NRC has received and completed a review of a revised SGDSM Database protocol provided by letter dated January 15, 1997. The updated protocol included in the letter incorporated comments provided by the staff via a letter to you dated September 24, 1996, and discussions held during a meeting held on November 6, 1996. The staff has concluded that the overall protocol appears acceptable with the exception of one item.

Item 5 of the protocol establishes guidelines for making judgements as to when a shift in a correlation is deemed to be significant and non-conservative. These criteria for the burst and leakage correlations are associated with potential changes in the voltage at the structural limit of 1.4 times main steam line break differential pressure and the leak rate at the largest repair limit, respectively. The NRC staff requests that the industry submit an assessment supporting the technical bases for this item of the protocol. The assessment should demonstrate the effects of shifts in the correlations equal to the limits established in Item 5 on, for example, the conditional burst probability and leak rate calculations. The staff will make a determination on the acceptability of Item 5 pending its review of the assessment.

Based on the staff's review of the proposed SGDSM Database protocol, it has concluded that the protocol is acceptable with the exception of Item 5. The staff notes that acceptance of the protocol at this time is based on the low likelihood of a significant non-conservative shift in the correlations in the near term. Therefore, we request that you provide the bases for supporting Item 5 for staff review within 90 days. Per the SGDSM Protocol, the staff would anticipate the annual update of the burst and leakage correlations as soon as possible. If, however, delays are expected in the implementation of the protocol, please inform the staff as

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to when the first update to the database will occur. If you have any questions, please call me at (301) 415-2722.

Sincerely,

ORIGINAL SIGNED BY

Brian W. Sheron, Director
Division of Engineering
Office of Nuclear Reactor Regulation

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