

NRC Form 386
(9-83)

U.S. NUCLEAR REGULATORY COMMISSION

APPROVED OMB NO. 3150-0104

EXPIRES: 9/31/86

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1) Grand Gulf Nuclear Station - Unit 1	DOCKET NUMBER (2) 0 5 0 0 0 4 1 6	PAGE (3) 1 OF 0 3
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TITLE (4) Two Isolation Valves Exceed Leak Test Frequency Due to Procedure Error

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)		
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES		DOCKET NUMBER(S)
0 1	1 0	8 6	8 6	0 0 2	0 0	0 2	1 0	8 6	N/A		0 5 0 0 0
											0 5 0 0 0

OPERATING MODE (9) 1		THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: (Check one or more of the following) (11)									
POWER LEVEL (10) 0 9 0		20.402(b)		20.406(c)		50.73(a)(2)(iv)		73.71(b)			
		20.406(a)(1)(i)		50.36(c)(1)		50.73(a)(2)(v)		73.71(e)			
		20.406(a)(1)(ii)		50.36(c)(2)		50.73(a)(2)(vii)		OTHER (Specify in Abstract below and in Text, NRC Form 386A)			
		20.406(a)(1)(iii)	X	50.73(a)(2)(i)		50.73(a)(2)(viii)(A)					
		20.406(a)(1)(iv)		50.73(a)(2)(ii)		50.73(a)(2)(viii)(B)					
		20.406(a)(1)(v)		50.73(a)(2)(iii)		50.73(a)(2)(ix)					

LICENSEE CONTACT FOR THIS LER (12)		TELEPHONE NUMBER	
NAME Ronald Byrd/Licensing Engineer		AREA CODE 6 1 0 1 1	4 3 7 1 - 1 2 1 1 4 1 9

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)										
CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRC	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRC	

SUPPLEMENTAL REPORT EXPECTED (14)		EXPECTED SUBMISSION DATE (15)	MONTH	DAY	YEAR
YES (If yes, complete EXPECTED SUBMISSION DATE)	X NO				

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

On January 10, 1986, the leak rate testing of two containment purge system isolation valves was determined to have exceeded the 92 day surveillance interval of Technical Specifications 4.6.1.2.j and 4.6.1.9.2. The valves are required to be demonstrated operable at least once per 92 days by the performance of a leakage rate test due to their resilient material seals. The Local Leak Rate Test (LLRT) procedure required the test every 24 months. The valves were last leak rate tested on October 11, 1984. Testing on January 10, 1986, showed the leakage rate to be well within acceptable limits. A review of the leakage rate testing requirements of Technical Specification 4.6.1.9.2 against the LLRT procedure was performed on January 10, 1986, and revealed no other associated errors. A comprehensive review of the entire LLRT procedure to ensure that no other areas of Technical Specification noncompliance exists is scheduled to be complete by March 7, 1986.

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NRC Form 366A
(9-83)

LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

U.S. NUCLEAR REGULATORY COMMISSION

APPROVED OMB NO. 3150-0104

EXPIRES 8/31/85

FACILITY NAME (1)	DOCKET NUMBER (2)	LER NUMBER (5)			PAGE (3)		
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER			
Grand Gulf Nuclear Station - Unit 1	05000416	86	002	00	02	OF	03

TEXT (If more space is required, use additional NRC Form 366A's) (17)

A. Reportable Occurrence

On January 10, 1986 at 1615 hours, the leak rate testing of two containment purge system isolation valves was determined to have exceeded the 92 day surveillance interval of Technical Specifications 4.6.1.2.j and 4.6.1.9.2. Noncompliance with this surveillance requirement is reportable pursuant to 10CFR50.73 (a)(2)(i)(B).

B. Initial Conditions

At the time of the discovery the plant was operating in mode 1 at 90 percent reactor power.

C. Description of Occurrence

Technical Specification 4.6.1.9.2 requires each containment purge supply and exhaust isolation valve with resilient material seals to be demonstrated operable at least once per 92 days by the performance of a leakage rate test. Containment purge system isolation valves Q1E61F009 and Q1E61F010 have resilient material seals but were identified by the Local Leak Rate Test (LLRT) procedure as requiring the test every 24 months.

Other containment purge system isolation valves with resilient material seals were included in the procedure as requiring the 92 day test frequency. Valves Q1E61F009 and Q1E61F010 were last leak rate tested on October 11, 1984.

A Limiting Condition for Operation (LCO) was entered pursuant to Technical Specification 3.6.1.9 until the valves were verified operable by performance of the leak test. The valves were returned to operable status at 2030 on January 10, 1985.

D. Apparent Cause

The cause of the noncompliance with the Technical Specification surveillance requirement was a procedural error in the LLRT procedure.

E. Supplemental Corrective Action

A review of the leakage rate testing requirements of Technical Specification 4.6.1.9.2 against the LLRT procedure was performed on January 10, 1986, and revealed no other associated errors. A comprehensive review of the entire LLRT procedure, 06-ME-1M61-V-0001, to ensure that no other areas of Technical Specification noncompliance exists is scheduled to be complete by March 7, 1986. The procedure is scheduled to be revised by March 28, 1986 to correct the specific discrepancy identified and any that may be found during the review.

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LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

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TEXT (If more space is required, use additional NRC Form 388A's) (17)

F. Safety Assessment

Valves Q1E61F009 and Q1E61F010 are inboard and outboard containment isolation valves at the same penetration on the discharge line of the containment purge compressor. The combined leakage through the two valves was found to be 199 Standard Cubic Centimeters per Minute (SCCM). The allowable leakage is 1435 SCCM for each valve. The leakage rates of the valves were well within the acceptable limits.



MISSISSIPPI POWER & LIGHT COMPANY

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February 10, 1986

NUCLEAR LICENSING & SAFETY DEPARTMENT

Document Control Desk
U. S. Nuclear Regulatory Commission
Washington, D. C. 20555

Gentlemen:

SUBJECT: Grand Gulf Nuclear Station
Unit 1
Docket No. 50-416
License No. NPF-29
File: 0260/L-835.0
Two Isolation Valves Exceed
Leak Test Frequency Due to
Procedure Error
LER 86-002-0
AECM-86/0038

Attached is Licensee Event Report (LER) 86-002-0 which is a final report.

Yours truly,

L. F. Dale
Director

JRM/SHH:bms
Attachment

cc: Mr. O. D. Kingsley, Jr. (w/a)
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Mr. N. S. Reynolds (w/a)
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