

TENNESSEE VALLEY AUTHORITY  
DIVISION OF NUCLEAR POWER  
SEQUOYAH NUCLEAR PLANT

MONTHLY OPERATING REPORT

TO THE

NUCLEAR REGULATORY COMMISSION

November 1, 1985 - November 30, 1985

UNIT 1

DOCKET NUMBER 50-327

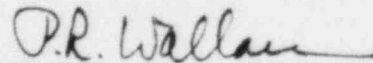
LICENSE NUMBER DPR-77

UNIT 2

DOCKET NUMBER 50-328

LICENSE NUMBER DPR-79

Submitted by:



P. R. Wallace, Plant Manager

8602190628 851130  
PDR ADOCK 05000327  
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## Operations Summary

November 1985

The following summary describes the significant operational activities for the month of November. In support of this summary, a chronological log of significant events is included in this report.

### Unit 1

The unit remained in the cycle three refueling/modification outage the entire month. The present schedule has the unit prepared to return to service on January 26, 1986. Actual start-up is dependent upon NRC review of the Sequoyah Startup Readiness Plan. The unit has been off-line for 100 days.

### Unit 2

The unit remained in the administrative shutdown the entire month due to documentation concerns relating to the environmental qualification of various electrical equipment (NUREG 0588). Outage related maintenance and modifications are being performed. The present schedule has the unit prepared to return to service on January 22, 1986. Actual start-up is dependent upon NRC review of the Sequoyah Startup Readiness Plan. The unit has been off-line 101 days.

### Significant Operational Events

#### Unit 1

<u>Date</u>	<u>Time</u>	<u>Event</u>
11/1/85	0001C	The reactor was in mode #6. Cycle 3 refueling/modification outage continued.
11/4/85	1030C	Core on-load began
11/12/85	1605C	Core on-load completed.
11/13/85	1713C	The reactor head was set in place.
11/18/85	0537C	The reactor entered mode #5.
11/20/85	1537C	The reactor re-entered mode #6. While reviewing the instruction package for tensioning the reactor head, it was determined that six bolts did not meet the acceptance criteria.
11/22/85	1540C	Entered mode #5. The bolts were tensioned and the acceptance criteria verified.
11/30/85	2400C	The reactor was in mode 5. Cycle 3 refueling/modification outage continues.

## Significant Operational Events (cont.)

### Unit 2

<u>Date</u>	<u>Time</u>	<u>Event</u>
11/1/85	0001C	The reactor was in mode #5. Administrative shutdown continued due to NUREG 0588 concerns.
11/30/85	2400C	The reactor was in mode #5. Administrative shutdown continues due to NUREG 0588 concerns.

### Fuel Performance

#### Unit 1

The core average fuel exposure accumulated during November was 0.00 MWD/MTU. The cycle 3 core reload of 193 fuel assemblies commenced on November 4, 1985, at 1033 CST. The core reload was completed on November 12, 1985, at 1605 CST. Fuel handling operations on reloading the core were delayed due to a number of mechanical breakdowns. These included the manipulator crane breakdown, the fuel transfer system's counter malfunction, and residual heater removal system (RHR) pump motor inoperability. These mechanical problems caused an estimated 97 hours of downtime.

Upon completion of fuel handling operations, verification of fuel orientation by serial number (TI-45: Physical Verification of Core Load prior to Vessel Closure), was performed on November 12, 1985, commencing at 1900 CST. This was conducted by two separate parties from plant Field Quality Engineering (FQE) with support assistance from plant Reactor Engineering Unit personnel who were not directly involved in the fuel movement. As the videotape recording was made, a core position map was prepared in accordance with TI-1. After the completion of core verification, plant FQE personnel verified this core position map against the Westinghouse Sequoyah Unit Core Loading Plan Cycle 4 (Drawing 1763E26). No discrepancies were observed.

A spent fuel pit inventory was conducted on November 15, 1985, commencing 0800 CST. This was conducted by Reactor Engineering Unit personnel. As the videotape recording was made, a spent fuel pit position map was prepared only on the affected discharged 72 bundles. No discrepancies were observed.

#### Unit 2

The core average fuel exposure accumulated during November was 0.00 MWD/MTU with the total accumulated core average exposure of 8097.51 MWD/MTU.

### Spent Fuel Pit Storage Capabilities

The total storage capability in the spent fuel pit (SFP) is 1,386. However, there are five cell locations which are not capable of storing spent fuel. Four locations (A10, A11, A24, A25) are unavailable due to a suction strainer conflict and one location (A16) is unavailable due to an instrumentation conflict. Presently, there is a total of 348 spent fuel bundles stored in the SFP. Thus, the remaining storage capacity is 1,033.



## PORVs and Safety Valves Summary

No PORVs or safety valves were challenged in November 1985.

### Licensee Events and Special Reports

The following licensee event report (LER) was reported to the Nuclear Regulatory Commission in November 1985.

<u>LER</u>	<u>DESCRIPTION OF EVENT</u>
1-85039	At 1300C, October 10, 1985 a containment ventilation isolation (CVI) occurred when an instrument mechanic went to the wrong radiation monitor to run a test.
1-85040	On October 9, 1985, at 1807C, during residual heat removal (RHR) pump swapover, indications showed that possible cavitation occurred resulting in a loss of both RHR trains. This problem was due to the low level of the reactor coolant system (RCS) in which the RHR pumps take their suction from loop #4.
1-85041	While aligning 1B-B diesel generator (D/G) for standby operation, an inadvertent start (ESF actuation) occurred at 1010C, October 9, 1985, when the fuse cover was lifted for a fuse inspection.
1-85042	A containment ventilation isolation (CVI) occurred from a false high-level radiation indication on "B" train purge air exhaust radiation monitor (1-RM-90-131) at 0715C, October 20, 1985. The impeller pump vanes were dislodged causing the monitor's recirculation pump motor to fail electrically.
1-85043	On October 23 and 24, the 1A-A and 1B-B centrifugal charging pumps were tested and "as found" failed to meet the acceptance criteria of technical specification 4.5.2.h.2.a. Surveillance Instruction (SI)-260.2; "Bit Cold Leg Injection Flow Balance, Pump Performance and Check Valve Test", requires the sum of the injection line flow rates, excluding the highest line flow must be greater than or equal to 346 gpm. The 1A-A and 1B-B pumps tested at 338 and 328 gpm respectively. The most probable cause is attributed to observed degradation of the pumps which has been documented as an industry-wide problem and is describe in Westinghouse Technical Bulletin No. NSD-TB-80-11. This test is performed every refueling outage.

## Licensee Events and Special Reports

### LER

### DESCRIPTION OF EVENT (cont.)

- 1-85044 The 1A-A and 1B-B safety injection pumps were tested on October 28 and November 8, 1985, and "as found" failed to meet the acceptance criteria of Technical Specification 4.5.2.h.1.b. Surveillance Instruction (SI)-260.3, "SIS Cold Leg Injection Flow Balance, Pump Performance and Check Valve Test" requires a total flow of less than or equal to 660 gpm. The 1A-A and 1B-B pumps were tested at 671 gpm and 664 gpm respectively. Both pumps were retested using calibrated Rosemounts. "As found" flow for 1A-A was 665 gpm and "as found" flow for 1B-B was 664 gpm. A review of the previous performance reveals that both pumps had been left with a total flow rates in excess of 660 gpm. The apparent reason for this deficiency was confusion of test personnel over which instrumentation could be used for determining technical specification compliance. This test is performed every refueling outage.
- 1-85045 At 0950C, October 31, 1985, an event occurred that resulted in a diesel generator (D/G) start and A & B train auxiliary building isolation (ABI). A defective racking mechanism on the alternate feeder breaker for the 1B start bus caused a loss of that bus when a normal transfer was attempted. This resulted in an undervoltage on the 1A-A shutdown board which started the D/Gs. The "B" train Auxiliary building isolation occurred on the loss of power because the "B" train spent fuel pool radiation monitor had been previously, inadvertently connected to a nondivisional power source. The 1B start bus alternate feeder breaker racking mechanism was found to have a bent racking lever which prevented the breaker from being fully racked in.
- 2-85011 A high radiation alarm was actuated at 0136C, October 10, 1985, which caused a containment ventilization isolation (CVI) to occur. A voltage spike occurred as a result of electromagnetic interference (EMI) which was spuriously generated by the low flow alarm switch in one incident. The source is unknown in the two other incidents.

### Special Reports

The following special report was transmitted to NRC in November 1985.

- 85-08 The steam generator tube inservice inspection was completed on unit 2 November 4, 1984. The results of this inspection were submitted to NRC by November 4, 1985.

### Offsite Dose Calculation Manual Changes

No changes were made to the Sequoyah Offsite Dose Calculation Manual during the month.

# OPERATING DATA REPORT

DOCKET NO. 50-327  
 DATE DECEMBER 9, 1985  
 COMPLETED BY D.C. DUPREE  
 TELEPHONE (615) 870-6544

## OPERATING STATUS

1. UNIT NAME: SEQUOYAH NUCLEAR PLANT, UNIT 1
2. REPORT PERIOD: NOVEMBER 1985
3. LICENSED THERMAL POWER(MWT): 3411.0
4. NAMEPLATE RATING (GROSS MWE): 1220.6
5. DESIGN ELECTRICAL RATING (NET MWE): 1148.0
6. MAXIMUM DEPENDABLE CAPACITY (GROSS MWE): 1183.0
7. MAXIMUM DEPENDABLE CAPACITY (NET MWE): 1148.0
8. IF CHANGES OCCUR IN CAPACITY RATINGS (ITEMS NUMBERS 3 THROUGH 7) SINCE LAST REPORT, GIVE REASONS: \_\_\_\_\_
9. POWER LEVEL TO WHICH RESTRICTED, IF ANY (NET MWE): \_\_\_\_\_
10. REASONS FOR RESTRICTIONS, IF ANY: \_\_\_\_\_

NOTES:

	THIS MONTH	YR.-TO-DATE	CUMULATIVE
11. HOURS IN REPORTING PERIOD	720.00	8016.00	38737.00
12. NUMBER OF HOURS REACTOR WAS CRITICAL	0.00	3797.25	24444.91
13. REACTOR RESERVE SHUTDOWN HOURS	0.00	0.00	0.00
14. HOURS GENERATOR ON-LINE	0.00	3762.18	23871.13
15. UNIT RESERVE SHUTDOWN HOURS	0.00	0.00	0.00
16. GROSS THERMAL ENERGY GENERATED (MWH)	0.00	12383285.96	77060971.91
17. GROSS ELECTRICAL ENERGY GEN. (MWH)	0.00	4239970.00	25976386.00
18. NET ELECTRICAL ENERGY GENERATED (MWH)	-2333.00	4061303.00	24942933.00
19. UNIT SERVICE FACTOR	0.00	46.93	61.62
20. UNIT AVAILABILITY FACTOR	0.00	46.93	61.62
21. UNIT CAPACITY FACTOR(USING MDC NET)	0.00	44.13	56.09
22. UNIT CAPACITY FACTOR(USING DER NET)	0.00	44.13	56.09
23. UNIT FORCED OUTAGE RATE	0.00	12.38	17.45
24. SHUTDOWNS SCHEDULED OVER NEXT 6 MONTHS (TYPE, DATE, AND DURATION OF EACH):			
-----			
25. IF SHUTDOWN AT END OF REPORT PERIOD, ESTIMATED DATE OF STARTUP:			
January 1986, pending NRC review of Sequoyah Nuclear Plant Startup Readiness Plan.			

NOTE THAT THE THE YR.-TO-DATE AND CUMULATIVE VALUES HAVE BEEN UPDATED.

# OPERATING DATA REPORT

DOCKET NO. 50-328  
DATE DECEMBER 9, 1985  
COMPLETED BY D.C. DUPREE  
TELEPHONE (615) 870-6544

## OPERATING STATUS

1. UNIT NAME: SEQUOYAH NUCLEAR PLANT, UNIT 2
2. REPORT PERIOD: NOVEMBER 1985
3. LICENSED THERMAL POWER(MWT): 3411.0
4. NAMEPLATE RATING (GROSS MWE): 1220.6
5. DESIGN ELECTRICAL RATING (NET MWE): 1148.0
6. MAXIMUM DEPENDABLE CAPACITY (GROSS MWE): 1183.0
7. MAXIMUM DEPENDABLE CAPACITY (NET MWE): 1148.0
8. IF CHANGES OCCUR IN CAPACITY RATINGS (ITEMS NUMBERS 3 THROUGH 7) SINCE LAST REPORT, GIVE REASONS: \_\_\_\_\_
9. POWER LEVEL TO WHICH RESTRICTED, IF ANY (NET MWE): \_\_\_\_\_
10. REASONS FOR RESTRICTIONS, IF ANY: \_\_\_\_\_

	THIS MONTH	YR.-TO-DATE	CUMULATIVE
11. HOURS IN REPORTING PERIOD	720.00	8016.00	30697.00
12. NUMBER OF HOURS REACTOR WAS CRITICAL	0.00	5289.42	21984.54
13. REACTOR RESERVE SHUTDOWN HOURS	0.00	0.00	0.00
14. HOURS GENERATOR ON-LINE	0.00	5224.24	21494.42
15. UNIT RESERVE SHUTDOWN HOURS	0.00	0.00	0.00
16. GROSS THERMAL ENERGY GENERATED (MWH)	0.00	17128966.35	69127977.22
17. GROSS ELECTRICAL ENERGY GEN. (MWH)	0.00	5845100.00	23536780.00
18. NET ELECTRICAL ENERGY GENERATED (MWH)	-3544.00	5615045.00	22636053.60
19. UNIT SERVICE FACTOR	0.00	65.17	70.02
20. UNIT AVAILABILITY FACTOR	0.00	65.17	70.02
21. UNIT CAPACITY FACTOR (USING MDC NET)	0.00	61.02	64.23
22. UNIT CAPACITY FACTOR (USING DER NET)	0.00	61.02	64.23
23. UNIT FORCED OUTAGE RATE	100.00	34.78	16.71
24. SHUTDOWNS SCHEDULED OVER NEXT 6 MONTHS (TYPE, DATE, AND DURATION OF EACH):			
25. IF SHUTDOWN AT END OF REPORT PERIOD, ESTIMATED DATE OF STARTUP:			
January 1986, pending NRC review of Sequoyah Nuclear Plant Startup Readiness Plan.			

NOTE THAT THE THE YR.-TO-DATE AND CUMULATIVE VALUES HAVE BEEN UPDATED.

## UNIT SHUTDOWNS AND POWER REDUCTIONS

REPORT MONTH NOVEMBER 1985

DOCKET NO. 50-327  
 UNIT NAME Sequoia One  
 DATE 12/9/85  
 COMPLETED BY D. C. Dupree  
 TELEPHONE (615) 870-6544

No.	Date	Type <sup>1</sup>	Duration (Hours)	Reason <sup>2</sup>	Method of Shutting Down Reactor <sup>3</sup>	Licensee Event Report #	System Code <sup>4</sup>	Component Code <sup>5</sup>	Cause & Corrective Action to Prevent Recurrence
6	850822	S	720	C	4				Refueling outage continues

<sup>1</sup>F: Forced  
 S: Scheduled

<sup>2</sup>Reason:  
 A-Equipment Failure (Explain)  
 B-Maintenance or Test  
 C-Refueling  
 D-Regulatory Restriction  
 E-Operator Training & License Examination  
 F-Administrative  
 G-Operational Error (Explain)  
 H-Other (Explain)

<sup>3</sup>Method:  
 1-Manual  
 2-Manual Scram.  
 3-Automatic Scram.  
 4-Cont. of Existing  
 Outage  
 5-Reduction  
 9-Other

<sup>4</sup>Exhibit G-Instructions  
 for Preparation of Data  
 Entry Sheets for Licensee  
 Event Report (LER) File  
 (NUREG-0161)

<sup>5</sup>Exhibit I-Same Source

## UNIT SHUTDOWNS AND POWER REDUCTIONS

REPORT MONTH NOVEMBER 1985DOCKET NO. 50-328UNIT NAME Sequoia TwoDATE 12/9/85COMPLETED BY D. C. DupreeTELEPHONE (615) 870-6544

No.	Date	Type <sup>1</sup>	Duration (Hours)	Reason <sup>2</sup>	Method of Shutting Down Reactor <sup>3</sup>	Licensee Event Report #	System Code <sup>4</sup>	Component Code <sup>5</sup>	Cause & Corrective Action to Prevent Recurrence
9	850821	F	720	D	4				NUREG 0588 Documentation concerns

<sup>1</sup>F: Forced  
S: Scheduled

<sup>2</sup>Reason:  
A-Equipment Failure (Explain)  
B-Maintenance or Test  
C-Refueling  
D-Regulatory Restriction  
E-Operator Training & License Examination  
F-Administrative  
G-Operational Error (Explain)  
H-Other (Explain)

<sup>3</sup>Method:  
1-Manual  
2-Manual Scram.  
3-Automatic Scram.  
4-Cont. of Existing  
Outage  
5-Reduction  
9-Other

<sup>4</sup>Exhibit G-Instructions  
for Preparation of Data  
Entry Sheets for Licensee  
Event Report (LER) File  
(NUREG-0161)

<sup>5</sup>Exhibit I-Same Source



SEQUOYAH NUCLEAR PLANT  
AVERAGE DAILY POWER LEVEL

DOCKET NO. : 50-327

UNIT : ONE

DATE : DECEMBER 9, 1985

COMPLETED BY : D. C. DUPREE

TELEPHONE : (615)870-6544

MONTH NOVEMBER 1985

DAY	AVERAGE DAILY POWER LEVEL (MWe Net)	DAY	AVERAGE DAILY POWER LEVEL (MWe Net)
01	0	17	0
02	0	18	0
03	0	19	0
04	0	20	0
05	0	21	0
06	0	22	0
07	0	23	0
08	0	24	0
09	0	25	0
10	0	26	0
11	0	27	0
12	0	28	0
13	0	29	0
14	0	30	0
15	0	31	
16	0		



SEQUOYAH NUCLEAR PLANT  
AVERAGE DAILY POWER LEVEL

DOCKET NO. : 50-328

UNIT : TWO

DATE : DECEMBER 9, 1985

COMPLETED BY : D. C. DUPREE

TELEPHONE : (615)870-6544

MONTH NOVEMBER 1985

DAY	AVERAGE DAILY POWER LEVEL (MWe Net)	DAY	AVERAGE DAILY POWER LEVEL (MWe Net)
01	0	17	0
02	0	18	0
03	0	19	0
04	0	20	0
05	0	21	0
06	0	22	0
07	0	23	0
08	0	24	0
09	0	25	0
10	0	26	0
11	0	27	0
12	0	28	0
13	0	29	0
14	0	30	0
15	0	31	
16	0		

## NUCLEAR PLANT OPERATING STATISTICS

SEQUOYAH NUCLEAR

Plant

Period Hours 720Month NOVEMBER 19 85

	Item No.	Unit No.	UNIT ONE	UNIT TWO	PLANT	
Generation	1	Average Hourly Gross Load, kW	0	0	0	
	2	Maximum Hour Net Generation, MWh	0	0	0	
	3	Core Thermal Energy Gen, GWD (t) <sup>2</sup>	0	0	0	
	4	Steam Gen. Thermal Energy Gen., GWD (t) <sup>2</sup>	0	0	0	
	5	Gross Electrical Gen., MWh	0	0	0	
	6	Station Use, MWh	2,333	3,544	5,877	
	7	Net Electrical Gen., MWh	-2,333	-3,544	-5,877	
	8	Station Use, Percent	N/A	N/A	N/A	
	9	Accum. Core Avg. Exposure, MWD/Ton <sup>1</sup>	0	8,098	8,098	
	10	CTEG This Month, 10 <sup>6</sup> BTU	0	0	0	
	11	SGTEG This Month, 10 <sup>6</sup> BTU	0	0	0	
	12					
Factors & Use	13	Hours Reactor Was Critical	0.0	0.0	0.0	
	14	Unit Use, Hours-Min.	0:00	0:00	0:00	
	15	Capacity Factor, Percent	0.0	0.0	0.0	
	16	Turbine Avail. Factor, Percent	0.0	0.0	0.0	
	17	Generator Avail. Factor, Percent	0.0	0.0	0.0	
	18	Turbogen. Avail. Factor, Percent	0.0	0.0	0.0	
	19	Reactor Avail. Factor, Percent	0.0	0.0	0.0	
	20	Unit Avail. Factor, Percent	0.0	0.0	0.0	
	21	Turbine Startups	0	0	0	
	22	Reactor Cold Startups	0	0	0	
	23					
Efficiency	24	Gross Heat Rate, Btu/kWh	N/A	N/A	N/A	
	25	Net Heat Rate, Btu/kWh	N/A	N/A	N/A	
	26					
	27					
Temp & Press	28	Throttle Pressure, psig	N/A	N/A	N/A	
	29	Throttle Temperature, °F	N/A	N/A	N/A	
	30	Exhaust Pressure, InHg Abs.	N/A	N/A	N/A	
	31	Intake Water Temp., °F	N/A	N/A	N/A	
	32		N/A	N/A	N/A	
Flows	33	Main Feedwater, M lb/hr	N/A	N/A	N/A	
	34					
	35					
	36					
Misc.	37	Full Power Capacity, EFPD	404.86 *	363.65	768.51	
	38	Accum. Cycle Full Power Days, EFPD	0.0	210.8416	210.8416	
	39	Oil Fired for Generation, Gallons			1782	
	40	Oil Heating Value, Btu/Gal.			138,000	
	41	Diesel Generation, MWh			27	
	42					
Station Data		Max. Hour Net Gen.	Max. Day Net Gen.	Load Factor, %		
		MWh	Time	Date	MWh	Date
	43	N/A	N/A	N/A	N/A	N/A
Remarks: <sup>1</sup> For BFNPP this value is MWD/STU and for SQNP and WBNP this value is MWD/MTU.						
<sup>2</sup> (t) indicates Thermal Energy.						
*Approximately						

Date Submitted DEC 10 1985Date Revised 11

*P.R. Waller*  
Plant Superintendent

## UNIT OUTAGE AND AVAILABILITY

SEQUOYAH Nuclear Plant

Licensed Reactor Power 3411 MW(th)Unit No. ONEGenerator Rating 1220.5 MW(e)Month/Year NOVEMBER 1985Design Gross Electrical Rating 1183 MWPeriod Hours 720

Day	Time Unit Available						Time Not Available								Unit				OUTAGE CAUSE	METHOD OF SHUTTING DOWN REACTOR	UNIT STATUS DURING OUTAGE	CORRECTIVE ACTION TAKEN TO PREVENT REPETITION		
	Total		Gen.		Not Used		Turbine		Gen.		Reactor		Unit		Time Out		Time In							
	Hrs	Min	Hrs	Min	Hrs	Min	Hrs	Min	Hrs	Min	Hrs	Min	Hrs	Min	Hrs	Min	Hrs	Min						
1	00	00	00	00			24	00	24	00	24	00	24	00					Refueling Outage continues	N/A	Mode 6			
2	00	00	00	00			24	00	24	00	24	00	24	00										
3	00	00	00	00			24	00	24	00	24	00	24	00										
4	00	00	00	00			24	00	24	00	24	00	24	00										
5	00	00	00	00			24	00	24	00	24	00	24	00										
6	00	00	00	00			24	00	24	00	24	00	24	00										
7	00	00	00	00			24	00	24	00	24	00	24	00										
8	00	00	00	00			24	00	24	00	24	00	24	00										
9	00	00	00	00			24	00	24	00	24	00	24	00										
10	00	00	00	00			24	00	24	00	24	00	24	00										
11	00	00	00	00			24	00	24	00	24	00	24	00										
12	00	00	00	00			24	00	24	00	24	00	24	00										
13	00	00	00	00			24	00	24	00	24	00	24	00										
14	00	00	00	00			24	00	24	00	24	00	24	00										
15	00	00	00	00			24	00	24	00	24	00	24	00										
16	00	00	00	00			24	00	24	00	24	00	24	00										
17	00	00	00	00			24	00	24	00	24	00	24	00										
18	00	00	00	00			24	00	24	00	24	00	24	00							Mode 5 @0537C			
19	00	00	00	00			24	00	24	00	24	00	24	00										
20	00	00	00	00			24	00	24	00	24	00	24	00							Mode 6 @1537C to retention bolts			
21	00	00	00	00			24	00	24	00	24	00	24	00							on the reactor head (11/20/85)			
22	00	00	00	00			24	00	24	00	24	00	24	00							Mode 5 @1940C (11/20/85)			
23	00	00	00	00			24	00	24	00	24	00	24	00										
24	00	00	00	00			24	00	24	00	24	00	24	00										
25	00	00	00	00			24	00	24	00	24	00	24	00										
26	00	00	00	00			24	00	24	00	24	00	24	00										
27	00	00	00	00			24	00	24	00	24	00	24	00										
28	00	00	00	00			24	00	24	00	24	00	24	00										
29	00	00	00	00			24	00	24	00	24	00	24	00										
30	00	00	00	00			24	00	24	00	24	00	24	00										
31																								
Total	00	00	00	00			720	00	720	00	720	00	720	00										

UNIT OUTAGE AND AVAILABILITY

SEQUOYAH Nuclear Plant  
Unit No. TWO

Month/Year NOVEMBER 1985

Period Hours 720

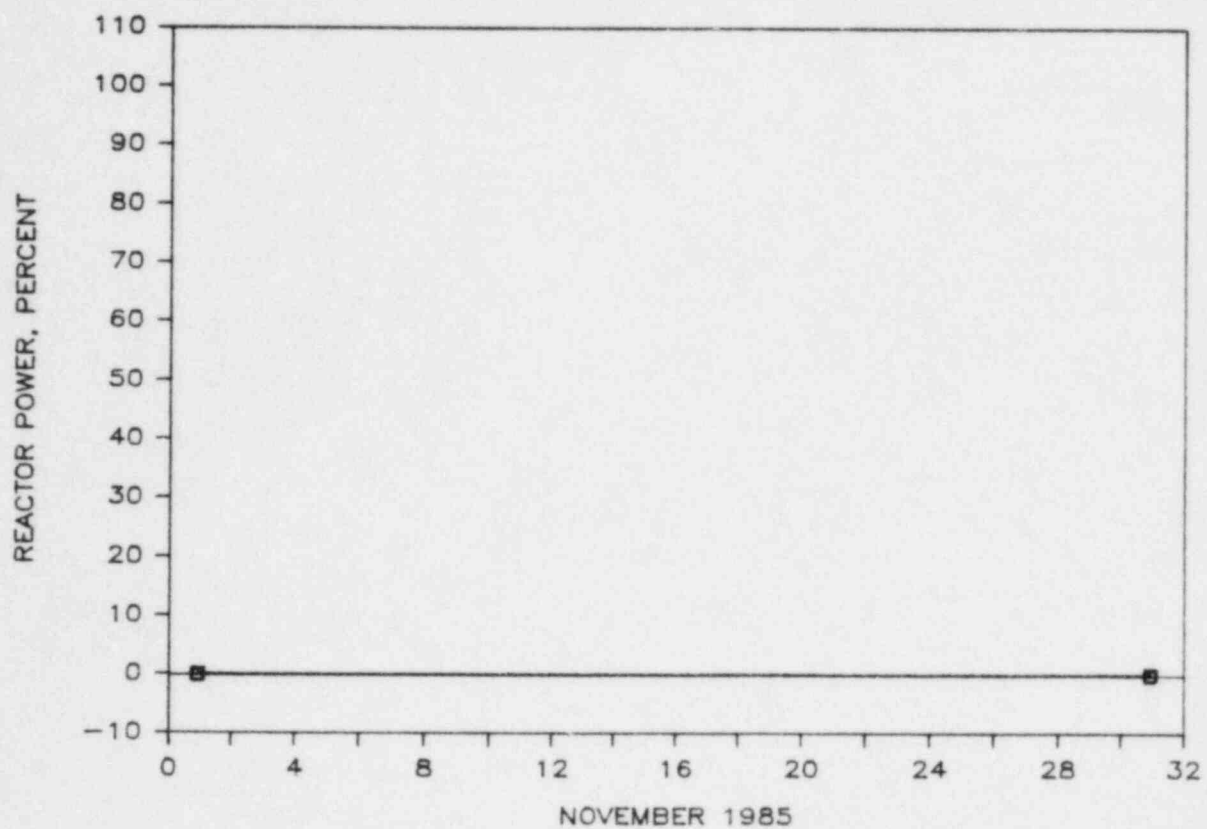
Licensed Reactor Power 3411 MW(th)

Generator Rating 1220.5 MW(e)

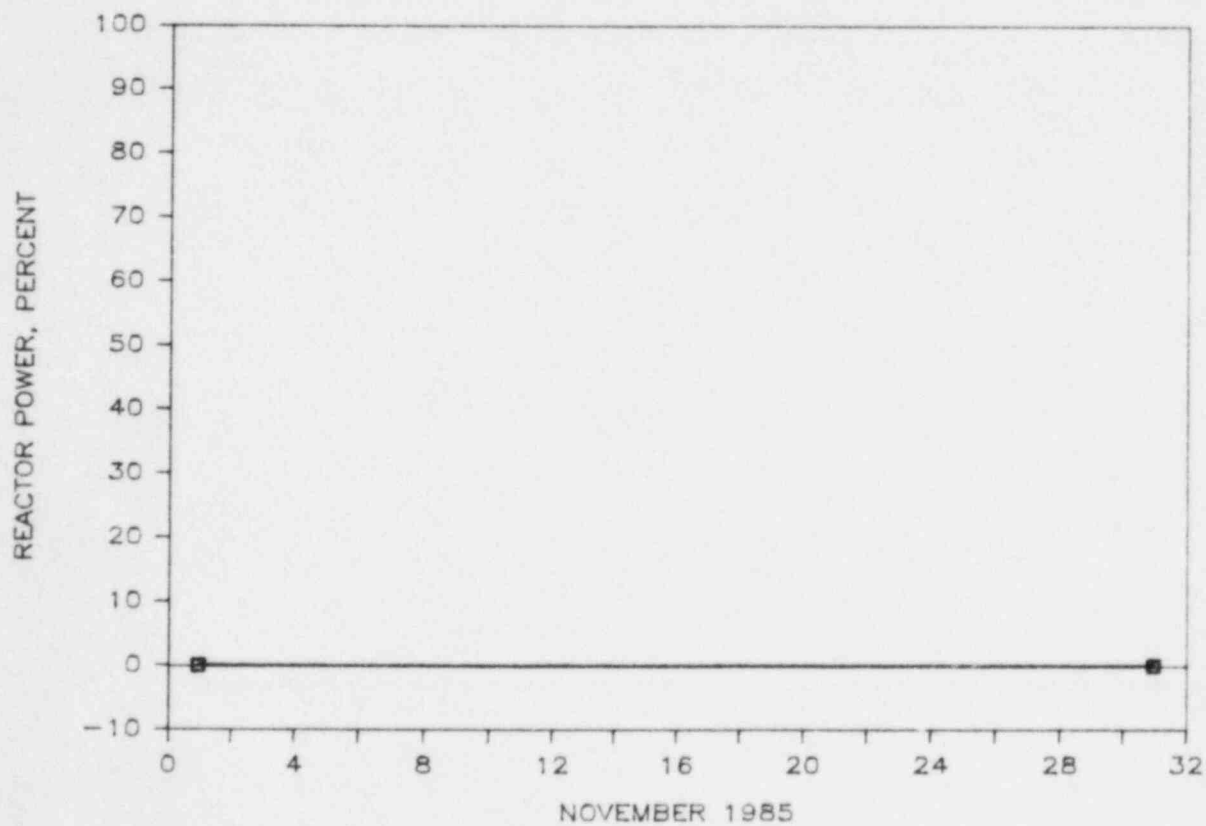
Design Gross Electrical Rating 1183 MW

Day	Time Unit Available						Time Not Available						Unit				OUTAGE CAUSE	METHOD OF SHUTTING DOWN REACTOR	UNIT STATUS DURING OUTAGE	CORRECTIVE ACTION TAKEN TO PREVENT REPLETION			
	Total			Gen			Not Used			Turbine			Reactor			Time Out					Time In		
	Hrs	Min	Sec	Hrs	Min	Sec	Hrs	Min	Sec	Hrs	Min	Sec	Hrs	Min	Sec	Hrs					Min	Hrs	Min
1	00	00	00	00	00	00				24	00	00	24	00	00	24	00	24	00	NUREG 0588 continues	N/A	Mode 5	
2	00	00	00	00	00	00				24	00	00	24	00	00	24	00	24	00				
3	00	00	00	00	00	00				24	00	00	24	00	00	24	00	24	00				
4	00	00	00	00	00	00				24	00	00	24	00	00	24	00	24	00				
5	00	00	00	00	00	00				24	00	00	24	00	00	24	00	24	00				
6	00	00	00	00	00	00				24	00	00	24	00	00	24	00	24	00				
7	00	00	00	00	00	00				24	00	00	24	00	00	24	00	24	00				
8	00	00	00	00	00	00				24	00	00	24	00	00	24	00	24	00				
9	00	00	00	00	00	00				24	00	00	24	00	00	24	00	24	00				
10	00	00	00	00	00	00				24	00	00	24	00	00	24	00	24	00				
11	00	00	00	00	00	00				24	00	00	24	00	00	24	00	24	00				
12	00	00	00	00	00	00				24	00	00	24	00	00	24	00	24	00				
13	00	00	00	00	00	00				24	00	00	24	00	00	24	00	24	00				
14	00	00	00	00	00	00				24	00	00	24	00	00	24	00	24	00				
15	00	00	00	00	00	00				24	00	00	24	00	00	24	00	24	00				
16	00	00	00	00	00	00				24	00	00	24	00	00	24	00	24	00				
17	00	00	00	00	00	00				24	00	00	24	00	00	24	00	24	00				
18	00	00	00	00	00	00				24	00	00	24	00	00	24	00	24	00				
19	00	00	00	00	00	00				24	00	00	24	00	00	24	00	24	00				
20	00	00	00	00	00	00				24	00	00	24	00	00	24	00	24	00				
21	00	00	00	00	00	00				24	00	00	24	00	00	24	00	24	00				
22	00	00	00	00	00	00				24	00	00	24	00	00	24	00	24	00				
23	00	00	00	00	00	00				24	00	00	24	00	00	24	00	24	00				
24	00	00	00	00	00	00				24	00	00	24	00	00	24	00	24	00				
25	00	00	00	00	00	00				24	00	00	24	00	00	24	00	24	00				
26	00	00	00	00	00	00				24	00	00	24	00	00	24	00	24	00				
27	00	00	00	00	00	00				24	00	00	24	00	00	24	00	24	00				
28	00	00	00	00	00	00				24	00	00	24	00	00	24	00	24	00				
29	00	00	00	00	00	00				24	00	00	24	00	00	24	00	24	00				
30	00	00	00	00	00	00				24	00	00	24	00	00	24	00	24	00				
31	00	00	00	00	00	00				24	00	00	24	00	00	24	00	24	00				
Total	00	00	00	00	00	00				720	00	00	720	00	00	720	00	720	00				

## SEQUOYAH ONE REACTOR HISTOGRAM



## SEQUOYAH TWO REACTOR HISTOGRAM



12:28:43 DATE....	12-04-85 COMPONENT.....	ELECTRICAL MAINTENANCE MONTHLY REPORT FOR NOVEMBER				PAGE 1
		FAILURE DESCRIPTION.....	CAUSE OF FAILURE.....	CORRECTIVE ACTION.....	MR.NO..	
01-25-85	1-MTRB-30-455A -A	MOTOR THERMALS OUT AFTER RESET AND START	MOTOR WAS PULLING EXCESSIVE CURRENT	REPLACED MOTOR AND CHECKED FOR PROPER OPERATION	A525886	
09-10-85	2-INVB-250-QU- G	#1 FAN MOTOR WOULD NOT RUN	FAN MOTOR BURNED UP DUE TO NORMAL WEAR	REPLACED #1 VITAL INVERTER FAN MOTOR	A542188	
09-19-85	1-BKRB-201-DM /5B-8	BREAKER CLOSING SPRING DID NOT RECHARGE WHEN BREAKER WAS CLOSED	1-LS-201-DM/5B-8 SPRING RECHARGE LIMIT SWITCH WAS DIRTY AND OUT OF MECHANICAL ADJUSTMENT PREVENTING LIMITS FROM MAKING UP	CLEANED AND ADJUSTED LIMIT SWITCH THEN CHECKED FOR PROPER OPERATION OF BREAKER	A533783	
09-20-85	2-INVB-250-QU- G	FAN FAILURE ALARM FLASHING	FAN POSSIBLY HAD DEFECTIVE FLOW SWITCH	REPLACED 2-IV VITAL INVERTER FAN AND CHECKED FOR PROPER OPERATION	A536332	
09-24-85	1-RLY-202-PHBB -ST	REPLACE TIMER WHICH CAN NOT BE ADJUSTED	TIMER DEFECTIVE	REPLACED TIMER AND RESET TIME PER SI-220.1	A544577	
09-24-85	1-RLY-201-XD20 87	REPLACE CRYDOM RELAY FOR 1-PMP-62-C-1	POSSIBLY RELAY COIL HAD OPENED DUE TO NORMAL WEAR	REPLACED CRYDOM RELAY	A085180	
10-11-85	2-FCV-001-0150	VALVE FAILED SI-166.6. THE LOCAL AND REMOTE STRIKE TIMES WERE GREATER THAN 1 SEC. APART.	UNKNOWN	ADJUSTED LIMIT STRIKERS ON VALVE SHAFT TO ALLOW THE LIGHT ACTUATION TO OCCUR WITH VALVE CLOSING	A536872	
10-13-85	2-INVB-250-OP- E	VITAL INVERTER 2-II FAN FAILURE ALARM WOULD NOT RESET AT INVERTER	VITAL INVERTER RELAY CONTACTS WERE DIRTY PREVENTING CONTACTS FROM MAKING UP	CLEANED AND REPAIRED RELAY CONTACTS AND CHECKED ALL FAN FLOW SWITCHES FOR PROPER OPERATION	A536101	
10-13-85	2-AHU-061-0024	FAN WOULD NOT RUN WITH	HANDSWITCH WIRES WERE	CHANGED WIRES ON	A534240	



12:28:43 DATE....	12-04-85 COMPONENT.....	ELECTRICAL MAINTENANCE MONTHLY REPORT FOR NOVEMBER			PAGE 2
		FAILURE DESCRIPTION.....	CAUSE OF FAILURE.....	CORRECTIVE ACTION.....	MR.NO..
	B	2-HS-061-0434B IN NORMAL POSITION BUT RUNS WHEN IN ON POSTION	CONNECTED TO NORMALLY CLOSED CONTACT	HANDSWITCH FROM NORMALLY CLOSED CONTACT TO NORMALLY OPEN CONTACT	
10-13-85	2-AHU-061-0026 B	FAN WOULD NOT RUN WITH 2-HS-61-454B IN NORMAL POSITION BUT RUNS IN ON POSITION	HANDSWITCH WIRES WERE CONNECTED TO NORMALLY CLOSED CONTACT	CHANGED WIRES ON HANDSWITCH FROM NORMALLY CLOSED CONTACT TO NORMALLY OPEN CONTACT	A534241
10-18-85	1-ZS-030-0020	LIMIT SWITCH INDICATES CONSTANT OPEN POSITION	LAMP HOLDER BURNED OUT OR DEFECTIVE	REPLACED INDICATING LAMP HOLDER IN MCR	A536411
10-18-85	2-FSV-062-0085 -B	VALVE INDICATOR LIGHTS SHOW BOTH OPEN AND CLOSED POSTION WHILE VALVE IN OPEN POSITION	SOLENOID VALVE WAS DEFECTIVE CAUSING EXCESSIVE AIR TO ESCAPE THROUGH BY-PASS PORT	REPLACED 2-FSV-62-85 SOLENOID VALVE AND BROKEN NIPPLE AND CHECKED FOR PROPER VALVE INDICATION	A527148
10-18-85	2-RLY-072-2A29 4	REPLACE DEFECTIVE CRYDOM RELAY 2A2-94 FOR TSC POINT 2-PMP-72-A-A	POSSIBLY RELAY COIL HAD OPENED DUE TO NORMAL WEAR	REPLACED RELAY	A116737
10-18-85	2-RLY-072-2B29 4	REPLACE DEFECTIVE CRYDOM RELAY 2B2-94 FOR TSC POINT 2-PMP-72-B-B	POSSIBLY RELAY COIL HAD OPENED DUE TO NORMAL WEAR	REPLACED RELAY	A116738
10-18-85	2-RLY-074-2B29 6	REPLACE DEFECTIVE RELAY 2B2-96 FOR TSC POINT 2-PMP-74-B-B	POSSIBLY RELAY COIL HAD OPENED DUE TO NORMAL WEAR	REPLACED RELAY	A116739
10-18-85	2-RLY-63-2A298	REPLACE DEFECTIVE CRYDOM RELAY 2A2-98 FOR TSC POINT 2-PMP-63-A-A	POSSIBLY RELAY COIL HAD OPENED DUE TO NORMAL WEAR	REPLACED RELAY	A116740
10-20-85	1-BKRC-082-TT/ 2-A	REPLACE 125V DC BREAKERS IN DISTRIBUTION PANEL FOR D/G ENGINE CONTROL PANEL,	UNKNOWN	REPLACED 4 20 AMP BREAKERS IN THE 125V DC DISTRIBUTION PANEL AND	A546189



12 28:43 DATE....	12-04-85 COMPONENT.....	ELECTRICAL MAINTENANCE MONTHLY REPORT FOR NOVEMBER		PAGE 3	
		FAILURE DESCRIPTION.....	CAUSE OF FAILURE.....	CORRECTIVE ACTION.....	MR.NO..
		GEN PROTECTIVE RELAYS, DC LUBE OIL PUMPS, EXCITATION PANEL, AND BATTERY CHARGER		CLEANED OIL OUT OF PANEL	
10-22-85	1-CRN-079-JN /9E1	REPLACE THE LS-6 ACTUATING ARM AND CHECK THE SWITCH AND REPLACE IF DAMAGED	LIMIT SWITCH HAD STRUCK HAND RAIL BOLTS	REPLACED LIMIT SWITCH ACTUATOR HEAD	A551209
10-23-85	1-CRN-079-JN /9E1	LS-16 IS NOT WORKING PROPERLY	DEFECTIVE LEVEL SWITCH	REPLACED LEVEL SWITCH 16 WITH LEVEL SWITCH FROM UNIT 2 MANIPULATOR CRANE	A545515
10-24-85	1-BATB-082-UB- A	CORRECT EXCESSIVE CONNECTION RESISTANCES ON 1A-A DIESEL GENERATOR BATTERY	BOLT, NUT AND WASHERS HAD DETERIORATED DUE TO EXCESSIVE ACID	REPLACED BOLT, NUT AND WASHER. CLEANED CONNECTIONS AND COATED WITH NO-OX	A518211
10-25-85	2-GENB-082-000 2A-A	REPLACE THE ELECTRICAL WIRING HARNESS ASSEMBLY BETWEEN THE ENGINE CONTROL CABINETS AND THE HYDRAULIC GOVERNORS	DEFECTIVE ELECTRICAL WIRING HARNESS ASSEMBLY	REPLACED ELECTRICAL WIRING HARNESS	A539669
10-28-85	2-LOCL-013-Z37 4B	ZONE 374B TROUBLE ALARM IN AND WILL NOT CLEAR	BAD FIRE DETECTOR HEAD	REPLACED FIRE DETECTOR HEAD AND RAN SI-234.7 AND CHECKED FOR PROPER OPERATION	A545142
10-28-85	1-BKRC-202-KE /201-A	BREAKER WOULD NOT CLOSE	1-BKRC-202-KE/201-A RELAY 30RX CONTACTS WERE DIRTY PREVENTING LIMITS FROM MAKING UP	CLEANED RELAY CONTACTS AND CHECKED FOR PROPER OPERATION	A545227
10-30-85	2-FCV-001-0032	BOTH RED AND GREEN	LIMIT SWITCH TRIP BAR WAS	ADJUSTED LIMIT SWITCH	A549039

12-28-83	12-04-85	ELECTRICAL MAINTENANCE MONTHLY REPORT FOR NOVEMBER			PAGE 4
DATE....	COMPONENT.....	FAILURE DESCRIPTION.....	CAUSE OF FAILURE.....	CORRECTIVE ACTION.....	MR.NO..
		ACTUATOR LIGHTS WERE ON WHILE VALVE IN CLOSED POSITION	OUT OF ADJUSTMENT	TRIP BAR	
10-30-85	1-FCV-061-0193	VALVE WOULD NOT OPEN WHILE HANDSWITCH IS HELD IN OPEN POSITION	ELECTRICAL COIL WAS DEFECTIVE DUE TO NORMAL WEAR	REPLACED COIL	A543868
10-30-85	1-RDB-201-DK-A	REPLACE 480V / 120V POTENTIAL TRANSFORMER ON 480V SHUTDOWN BOARD 1A2A	BAD POTENTIAL TRANSFORMER	REPLACED TRANSFORMER AND VERIFIED VOLTAGE METER READING CORRECTLY	A119571
10-31-85	0-MTRB-311-003 4	BATTERY ROOM EXHAUST FAN MOTOR BEARINGS WERE BAD	FAN MOTOR B HAD BAD MOTOR BEARINGS DUE TO NORMAL WEAR	REPLACED MOTOR BEARINGS AND REITERMINATED MOTOR	A533952
11-02-85	2-MTRB-030-007 7-A	FLEX FEEDING FAN MOTOR IS BROKEN AT BOTH ENDS EXPOSING CONDUCTORS	PERSONNEL CLIMBING ON FLEX CAUSING IT TO BREAK AT BOTH ENDS	REPAIRED FLEX AND JOINTS	A126916
11-03-85	1-PMP-067-0436	ERCW PUMP K-A WOULD NOT START FROM CONTROL ROOM HANDSWITCH	DIRTY RELAY CONTACTS CAUSING LATCHING MECHANISM NOT TO WORK PROPERLY	CLEANED CONTACTS ON 1X, 30X, 30RX AND REWORKED LATCH ON 30RX	A536032
11-03-85	2-FCV-062-0072	VALVE HAS BLOWN FUSES	RELAY KS1A HAD A BAD RELAY COIL CAUSING FUSES TO BLOW	REPLACED REALY KS1A WITH A SPARE RELAY FROM SAME CABINET 2-R-54	A560063
11-03-85	2-FSV-062-0074 A	VALVE WAS BLOWING CONTROL POWER FUSES WHEN OPENING FROM CLOSED POSITION	120V DC COIL WAS BAD IN KS1C RELAY	REPLACED REALY KS1C IN RELAY CABINET 2-R-54	A534072
11-03-85	2-FCV-62-72	VALVE HAS BLOWN FUSES TWICE IN ONE SHIFT	RELAY KS1A HAD A BAD COIL CAUSING FUSES TO BLOW	REPLACED RELAY KS1A WITH A SPARE RELAY FROM	A560063

12-28-43	12-04-85	ELECTRICAL MAINTENANCE MONTHLY REPORT FOR NOVEMBER			PAGE 5
DATE....	COMPONENT.....	FAILURE DESCRIPTION.....	CAUSE OF FAILURE.....	CORRECTIVE ACTION.....	MR.NO..
				CABINET 2-R-54	
11-04-85	1-MVOP-070-001 0-A	VALVE WOULD NOT ELECTRICALLY CLOSE TO FULL CLOSE POSITION	VALVE TORQUES OUT PRIOR TO REACHING FULL CLOSED POSITION	ADJUSTED TORQUE SWITCH	A534194
11-05-85	1-FCV-074-0012	SUSPECT BAD GEARS ON FLOW CONTROL VALVE. VALVE WOULD NOT OPEN	MOTOR WAS BAD	REPLACED CAM, TRIPPER MECHANISM AND MOTOR	A545028
11-06-85	0-MTRB-079-HC /681	MOTOR BACK SPINS AFTER TORQUE SWITCH OPENS	PIVOT PIN OUT OF KEYWAY	PUSHED PIVOT PIN BACK INTO POSITION AND CHECKED FOR PROPER OPERATION OF MOTOR	A536036
11-07-85	1-BCTA-202-CL/ 1416	MECHANICAL TRIP FREE ARM WOULD NOT CONTINUALLY LOCK INTO PLACE WHEN BREAKER WAS RACKED ONTO THE BUSS, MAKING THE BREAKER INOPERABLE	MECHANICAL TRIP FREE ARM WAS BENT	REPAIRED TRIP FREE ARM AND FUNCTIONALLY CHECKED BREAKER	A541276
11-07-85	2-AHU-061-0023 A	AIR HANDLING UNIT WOULD NOT GO INTO DEFROST CYCLE AUTOMATICALLY OR MANUALLY	RELAY WAS DEFECTIVE	REPLACED DR REALY AND CHECKED FOR PROPER OPERATION	A543803
11-08-85	2-PNLS-082-TV- A	BATTERY DISTRIBUTION PANEL AMMETER WOULD NOT WORK	METER WAS DEFECTIVE	REPLACED AMMETER	A547975
11-08-85	1-MVOP-003-006 7	REBUILD LIMITORQUE OPERATOR	OPERATOR HAD MIXED GREASE AND INTERNAL RUSTING	REBUILT OPERATOR AND RAN MOVATS TEST	A299992
11-10-85	2-XI-068-03400	VALVE WAS SHOWING OPEN POSITION WHILE THE CONTROLLER WAS IN THE	LIMIT SWITCH WAS OUT OF ADJUSTMENT	ADJUSTED LIMIT SWITCH STOP	A534212

12:28:43 DATE....	12-04-85 COMPONENT.....	ELECTRICAL MAINTENANCE MONTHLY REPORT FOR NOVEMBER			PAGE 6
		FAILURE DESCRIPTION.....	CAUSE OF FAILURE.....	CORRECTIVE ACTION.....	MR.NO..
		CLOSED POSITION			
11-11-85	1-FCV-026-0240 -A	VALVE WOULD NOT OPEN ELECTRICALLY FROM FULL CLOSED POSITION, ALSO VALVE BODY HAD EXCESSIVE MOVEMENT WHEN OPENING VALVE WITH HANDWHEEL	VALVE YOKE WAS ROTATING DUE TO SETSCREWS VIBRATING LOOSE	TIGHTENED SETSCREWS AND CLEANED INTERLOCK CONTACTS ON CONTACTOR	A545012
11-12-85	2-GENB-082-000 2A-A	ANNUNCIATOR WOULD NOT CLEAR ON 0-M-26 WINDOW 15 AS REQUIRED BY SI 102	FIELD CONTACT WAS CLOSED CAUSING ANNUNCIATOR TO STAY ON	REPAIRED POINT CARD CLEARING ANNUNCIATOR	A561105
11-13-85	1-GENB-082-000 1B-B	REPLACE THE ELECTRICAL WIRING HARNESS ASSEMBLY BETWEEN THE ENGINE CONTROL CABINETS AND EGB-BPS	UNKNOWN	INSTALLED GOVERNOR CABLE ASSEMBLY ON 1B-1 AND 1B-2 DIESEL GENERATOR GOVERNORS	A539671
11-14-85	1-PNLA-082-TT- A	1A-A BATTERY DISTRIBUTION PANEL AMMETER WOULD NOT ADJUST PER MI-10.2, PM 1357-082	AMMETER WAS DEFECTIVE	REPLACED AMMETER	A522732
11-15-85	2-GENB-082-000 2A-A	GOVERNOR ACTUATOR DIFFERENTIAL WOULD NOT WORK AS REQUIRED BY SI-102	TDC RELAY WAS DEFECTIVE	REPLACED TDC RELAY	A561101
11-17-85	1-INVB-250-GL- D	FLOW SWITCH FAILED TO OPERATE	CONTACTS WERE STICKING ON RELAY MRLB	REPLACED MRLB RELAY AFTER TROUBLE SHOOTING THE ENTIRE COOLING CIRCUITS AND ALARM CIRCUITS	A545025
11-19-85	1-GENB-082-000 1A-A	REPLACE ELECTRICAL WIRING HARNESS ASSEMBLY BETWEEN	UNKNOWN	REPLACED WIRING HARNESS	A539670

12-28-83 12-04-85 ELECTRICAL MAINTENANCE MONTHLY REPORT FOR NOVEMBER PAGE 7  
 DATE.... COMPONENT..... FAILURE DESCRIPTION..... CAUSE OF FAILURE..... CORRECTIVE ACTION..... MR.NO..

THE ENGINE CONTROL  
 CABINETS AND THE EGB-BP

DATE	COMPONENT	FAILURE DESCRIPTION	CAUSE OF FAILURE	CORRECTIVE ACTION	MR.NO.
11-23-85	0-BKRD-202-KC/ 202	6.9 KV BREAKER FOR SERVICE BUILDING MAIN BOARD BUS B HAD A BAD CHARGING MOTOR	BAD MOTOR AND CONTROL DEVICE	REPLACED MOTOR AND CONTROL DEVICE	A594570

48 records listed.

## INSTRUMENT MAINTENANCE

### Unit 1

Completed installation of new automatic level controllers for the main feedwater bypass valves. This modification was authorized by ECN-6173 and performed on workplan 11786.

Rescaling and recalibration of the Reactor Vessel Level Indication System (RVLIS) electronics was performed by workplan 11841 and Westinghouse procedure NSID-EIS-85-07. A preliminary post-modification test was performed by the Systems Engineering Section to compare the train B RVLIS upper plenum indication with the actual level (as determined from the tygon tubing used during filling and venting operations). All readings taken on the RVLIS indicator were satisfactory (within  $\pm 2\%$  of actual level). The official post-modification test will be performed during startup.

During performance of SI-90.2, a main feedwater flow bistable, 1-FS-3-90B, was found inoperable (PRO-1-85-348). The bistable had failed in the energized state. A maintenance request was initiated and the problem traced to a shorted transistor. The failed component was replaced and the bistable recalibrated.

### Unit 2

Completed Westinghouse modification RVLIS circuit cards per FCV-TENO-40558A. This was necessary to eliminate seismic drift problems identified with the associated electronics.

Completed installation of new automatic flow controllers for the main feedwater pump mini-flow recirculation line valves. This modification was authorized by ECN-6017 and performed on workplan 11489.

### Common

Completed workplan 11703 to change the setpoints on the diesel generator soak-back oil pressure switches (ECN-5451).

Provided engineering support for the environmental qualification program. Performed field verifications, reviewed and prepared comments on preliminary QMDS requirements, and began scoping of additional work activities/modifications required to ensure equipment qualification.

COMP

MR.	COMP	U	FUNC	SYS	ADDRESS	DATE	DESCRIPTION	CORRECTIVE ACTION
A299300	2	PM	030	310	11/05/85	2-PM-030-310--TROUBLE SHOOT LOOP TO DETERMINE PROBLEM WITH DRIFTING TSC POINT	SIGNAL CABLE SHIELD NOT GROUNDED A ONE END. RETERMINATE WHEN TEMPORARY CABLE IS REMOVED.	
A300823	1	PS	082	164	11/22/85	1-PS-082-164--*I# Y100203 DN 1 SI 102 DG 1AA STEP 5.3.1.1.4 LOW AIR PRESS ALARM AT 188 PSI SHOULD BE 200 +OR-10	SWITCH OUT OF CALIBRATION. RECALIBRATE SWITCH.	
A524904	2	FI	003	163A	11/25/85	2-FI-003-163A--*NPRD# FLOW IND IS READING 70 GPM W/LCV CLOSED AND AUX FLOWTR PMP OFF	LOW FLUID REFERENCE LEG. BACK FILL REFERENCE	
A535812	0	RM	090	102	11/26/85	0-RM-090-102--RELAY VKAR 6 DID NOT ENERGIZE ON HI RAD SIGNAL FROM 0 RM 90 102 DURING SI 83 1 R 73	HIGH RAD SIGNAL GIVEN TO MONITOR SO RELAY WOULDN'T MOVE. DELETE RELAY SI 83. <i>Relay has been deleted from ckt</i>	
A536026	0	RM	090	103	11/04/85	0-RM-090-103--*I# RAD MON 90-103 PLUGGED INTO NON DIVISIONAL PLUGMOLD IN CAB CORRECT	MONITOR PLUGGED INTO NON-DIVISIONAL PLUG IN CABINET. PLUG MONITOR INTO TRAIN B/PLUG MOLD.	
A536346	0	RM	090	122	11/07/85	0-RM-090-122--CLEAN OUT DETECTOR HOUSING. BACKGROUND TOO HIGH	BACKGROUND RADIATION CAUSING FALSE READINGS. CLEAN OUT DETECTOR HOUSING TO LOWER BACKGROUND READING.	
A538645	1	FT	003	488	11/04/85	1-FT-003-488--HIGH SIDE DRN VLV STOPPED UP	INSTALL NEW HIGH SIDE DRAIN VALVE	
A539518	2		099		11/13/85	2--099--*NPRD# INSPECT BARTON LOT 2 XMTRS FOR HARDWIRED PIN CONNECTORS	NONE. INSPECT BARTON LOT 2 TRANSMITTERS FOR HARDWIRED PIN CONNECTIONS.	
A539526	2	LT	003	42	11/29/85	2-LT-003-42--HARD WIRE PIN CONNECTOR FOR BARTON LOT 2 XMTR TO MAINTAIN QUALIFICATIONS PER SMI 2 317 23	PIN CONNECTOR NEED HARDWIRING PER SMI 2-317-23. HARDWIRE PIN CONNECTS AND RECALIBRATE TRANSMITTER.	
A541094	0	FS	090	122	11/20/85	0-FS-090-122--*I# FLOW SW NOT PICKING UP AND GIVING LIGHT OR AUDIBLE ALARM	DEBRIS IN FLOW SWITCH. CLEAN OUT SWITCH AND VERIFY OPERABILITY.	
A541281	0	PS	077	119	11/25/85	0-PS-077-119--*I# PRESS SW WON'T ALLOW 0 FCV 77 119 TO BE OPEN	NO PROBLEM FOUND, NONE	
A546144	0	FS	065	31A/B	11/18/85	0-FS-065-31A/B--NEED TO CLEAN & CHECK RELIABILITY OF CONTACTS	READ JUST REFERENCE BALANCE RESISTANCE TO THE PR OPER VALVE	
A548863	1	FI	062	1	11/26/85	1-FI-062-1-A-*NPRD# #1 RCP SEAL FLOW IND INDICATES APPROX 7 GPM FLOW WHEN ALL SEAL FLOW IS ISOL	AIR IN TRANSMITTER SENSING LINES. VENT AIR FROM LINES	
A548864	1	FI	062	27	11/26/85	1-FI-062-27-A-*NPRD# #3 RCP SEAL FLOW IND INDICATES APPROX 48 1/2 GPM FLOW WHEN ALL SEAL FLOW IS ISOL	VENT AIR FROM SENSING LINES. AIR IN TRANSMITTER SENSING LINES	
A548904	1	PDT	030		11/07/85	1-PDT-030--INSP THE AMPLIFIER CARD IN THE TRANSMITTER PER THE ATTACHED FOXBORD NOTICE	NONE, INSPECT AMP CARD FOR EQ PROJECT VERIFICATION.	
A548906	1	PDT	030	43	11/07/85	1-PDT-030-43--INSP AMPLIFIER CARD IN THE	NONE, INSPECT AMP CARD PER EQ PROJECT	



COMP

MR. COMP U FUNC SYS ADDRESS. DATE.... DESCRIPTION..... CORRECTIVE ACTION.....

TRANSMITTER PW NEEDED FOR EQ PROJECT  
VERIF

MR. COMP	U	FUNC	SYS	ADDRESS	DATE	DESCRIPTION	CORRECTIVE ACTION
A549040	2	LT	003	97	11/01/85	2-LT-003-97-, +=#NPRD# RE CALAIBRATE OTHERS READ 24% THIS ONE READS 33%	SENSE LINES NEED BACKFILLING. BACKFILL SENSE LINES.
A550568	1	TE	068	RCPI	11/26/85	1-TE-068-RCPI-, REPAIR AND REPLACE WIRING AND LUGS ON RTD'S FOR RC PMP 1	BAD LUGS ON RTD'S. REPLACE LUGS ON RTD/S
A559802	1	LS	087	21	11/22/85	1-LS-087-21-, MEASURE REFER LEG	NONE. MEASURE REFERENCE LEG.
A559803	1	LS	087	22	11/22/85	1-LS-087-22-, MEASURE REFER LEG	NONE. MEASURE REFERENCE LEG.
A559804	1	LS	087	23	11/22/85	1-LS-087-23-, MEASURE REFER LEG	NONE. MEASURE REFERENCE LEG.
A559806	1	LS	087	24	11/22/85	1-LS-087-24-, MEASURE REFER LEG	NONE. MEASURE REFERENCE LEG.
A561107	2	PS	082	240	11/13/85	2-PS-082-240-, COMPR 1 DID NOT START AT 200 +OR-PSI D/G 2AA BLDG	PRESSURE SWITCH OUT OF CALIBRATION. RECALIBRATE PRESSURE SWITCH.
A594001	2	PM	030	311	11/05/85	2-PM-030-311-, TROUBLE SHOOT LOOP TO DETERMINE PROBLEM WITH DRIFTING TSC POINT	SIGNAL CABLE NOT GROUNDED AT ONE END. RETERMINATE WHEN TEMPORARY CABLE REMOVED.
A594002	1	PM	030	310	11/05/85	1-PM-030-310-, TROUBLE SHOOT LOOP TO DETERMINE IF PROBLEM EXISTS ON TSC POINT	SIGNAL CABLE NOT GROUNDED AT ONE END. RETERMINATE WHEN TEMPORARY CABLE REMOVED.
A594003	1	PM	030	311	11/05/85	1-PM-030-311-, TROUBLE SHOOT LOOP TO DETERMINE IF PROBLEM EXISTS WITH TSC POINT	SIGNAL CABLE NOT GROUNDED AT ONE END. RETERMINATE WHEN TEMPORARY CABLE REMOVED.
A594005	1	FM	068	3810	11/13/85	1-FM-068-3810-, CARD DRIFTS REPLACE CARD W/NEW CARD S/N A12972	BAD MODIFIER CARD. INSTALL NEW CARD.

27 records listed.

## Mechanical Maintenance Section

November 1985

### Unit 0

- (1) Completed 5 year inspection on 2AA lube oil cooler.
- (2) Completed monthly inspection on 2BB diesel generator.

### Unit 1

- (1) Completed all 'J' Tube work.
- (2) Preparing reactor coolant pump #2 for starting.
- (3) Machined threads off RHR pump 1B to determine weld procedure.
- (4) Loaded 21 fuel bundles into core.
- (5) Replaced solenoid on 1-LSV-3-174.
- (6) Completed handholes on steam generator #1.
- (7) Removed low pressure seals in seal table for ferrule inspection and installed new fittings.
- (8) Completed reroll on steam generator #1.
- (9) Placed 1-VLV-63-635 back in service.
- (10) Retrieved dropped bolt from steam generator #4 and closed up.
- (11) Installed oil deflector on main turbine.

### Unit Two

- (1) Adjusted main steam dump drain header.
- (2) Completed SI-107.
- (3) Welding completed on 2A RFP suction.
- (4) Repaired MTOT Head Leak.
- (5) Inspected all but one of LP turbine explosion diaphragms, all cracked so far.

COMP

MR.	COMP	U	FUNC	SYS	ADDRESS.	DATE....	DESCRIPTION.....	CORRECTIVE ACTION.....
A525607	1	HGR	062	1CVCH152	11/08/85	1-HGR-062-1CVCH152-, REPLACE BOLT D AS INDICATED ON SMR 0 317 21	BOLT D IS TOO SHORT FOR REQ'D THREAD ENGAGEMENT PER SMI 0 317 21. INSTALLED NEW BOLT	
A525608	1	HGR	063	163SIH28	11/07/85	1-HGR-063-163SIH28-, REPLACE BELT 'A' AS INDICATED ON SMI 0 317 21	REPLACE BOLT 'A' PER SMI 0 317 21. INSTALLED NEW BOLT	
A525612	1	HGR	062	1CVCH863	11/07/85	1-HGR-062-1CVCH863-, BOLT D IS TOO SHORT FOR REQ'D THREAD ENGAGEMENT AND HAS DAMAGED THREADS	BOLT D TOO SHORT TO FOR REQ'D THREAD ENGAGEMENT PER SMI 0 317 21. INSTALLED NEW BOLT	
A528101	2	FLT	062	97	11/12/85	2-FLT-062-97-, SEAL WTR INJ FILTER HAS HIGH DELTA P. REPLACE FILTER	FLTR HAS HIGH DELTA P READING. REPLACED FLTR.	
A532705	2	FCV	065	9	11/22/85	2-FCV-065-9-, #NPRD# REBUILD ACTUATOR AND CHANGE GREASE	NA	
A532706	1	FCV	065	10	11/22/85	1-FCV-065-10-, #NPRD# REBUILD ACTUATOR AND CHANGE GREASE	NA	
A532707	2	FCV	065	29	11/22/85	2-FCV-065-29-, #NPRD# REBUILD ACTUATOR AND CHANGE GREASE	NA	
A532722	2	FCV	030	47	11/22/85	2-FCV-030-47-, #NPRD# REBUILD ACTUATOR AND CHANGE GREASE	NA	
A532723	1	FCV	030	48	11/22/85	1-FCV-030-48-, #NPRD# REBUILD ACTUATOR	NA	
A536965	1	FCV	003	67	11/12/85	1-FCV-003-67-, INVESTIGATE WHY VLV STICKING OPEN	NA	
A541263	1	OMP	082	180	11/04/85	1-OMP-082-180-, COMP UNLOADER IS LEAKING. IT IS ALSO LIFTING BOTH RELIEF VLVs AND WILL BUILD NOT ENOUGH PRESS ONLY GOES TO 190 PSIG BEFORE LIFTING	REPLACED INLET & DISCHARGE VALVES HYD UNLOADER GOVERNOR REPLACED O RING AN PUMP SHAFT	
A543881	1	PCV	001	12	11/26/85	1-PCV-001-12-, WHERE THE TWO HALVES OF THE DIAP CONTROLLER BOLT TOGETHER THERE IS AN AIR LEAK	INSTALLED NEW DIAPHRAGM	
A545233	0	OMP	032	86	11/20/85	0-OMP-032-86-, INSPECT COMP AND REPLACE INTAKE VLVs AND DISCHARGE VLVs	NA	
A545912	2	ENG	082	2A2	11/18/85	2-ENG-082-2A2-, REPLACE THE 3/8 AIR HOSE BETWEEN THE AIR MOTORS AND THE SUPPLY HEADER	REPLACED AIR LINES ON ALL 4 AIR START MOTORS	
A546355	2	FLT	062	96	11/12/85	2-FLT-062-96-, DP > 30. CHANGE OUT FILTER	CHANGE OUT FLTR.	
A549803	1	HGR	015	11/19/85	1-HGR-015-, DETERMINE WHETHER NUTS ARE WELDED BEHIND THE BASEPLATE RATHER THAN THE BOLT BEING ANCHORED INTO THE CONCRETE	NA		
A549805	1	HGR	030	1AHU301A	11/01/85	1-HGR-030-1AHU301A-, DETERMINE WHETHER NUTS ARE WELDED BEHIND THE BASEPLATE RATHER THAN THE BOLT BEING ANCHORED	DETERMINE WHETHER NUTS ARE WELDED BEHIND THE BASEPLATE RATHER THAN BEING ANCHORED IN THE CONCRETE PER EMPLOYEE	

COMP

MR.COMP	U	FUNC	SYS	ADDRESS.	DATE....	DESCRIPTION.....	CORRECTIVE ACTION.....
						INTO THE CONCRETE. INVESTIGATION OF EMPLOYEE CONCERN XX 85 031001 CHARGE TO E661	CONCERN XX 85 031 001. NO NUT FND BEHIND BASEPLATE BOLTS A B C ARE 2.75" LONG D IS 2.5 " LONG
A549808	1	HGR	074	1RHRH465	11/04/85	1-HGR-074-1RHRH465-, DETERMINE WHETHER NUTS ARE WELDED BEHIND THE BASEPLATE RATHER THAN THE BOLT BEING ANCHORED INTO THE CONCRETE. INVESTIGATION OF EMPLOYEE CONCERN XX 85 031 001 CHARGE TO E661	DETERMINE WHETHER NUTS ARE WFLDED BEHIND THE BASEPLATE RATHER THAN BEING ANCHORED IN THE CONCRETE PER EMPLOYEE CONCERN XX 85 031 001. BOLT A IS 1.9" LONG B IS 2.2" LONG C IS 2.1" LONG D IS 2.0 " LONG
A549809	1	HGR	072	1CSH449	11/04/85	1-HGR-072-1CSH449-, DETERMINE WHETHER NUTS ARE WELDED BEHIND THE BASEPLATE RATHER THAN THE BOLT BEING ANCHORED INTO THE CONCRETE. INVESTIGATION OF EMPLOYEE CONCERN XX85031 CHARGE TO E661	NA
A549810	1	HGR	063	658A4-14	11/01/85	1-HGR-063-658A4-14-, DETERMINE WHETHER NUTS ARE WELDED BEHIND THE BASEPLATE RATHER THAN THE BOLT BEING ANCHORED INTO THE CONCRETE. INVEST. OF EMPLOYEE CONCERN XX-085-031-001 CHG TO E	DETERMINE WHETHER NUTS ARE WFLDED BEHIND THE BASEPLATE RATHER THAN BEING ANCHORED IN THE CONCRETE PER EMPLOYEE CONCERN XX 85 031 001. BOLTS C AND D ARE 1.8" LONG B IS 1.7" LONG A IS 1.6" LONG
A549811	1	HGR	063	1SIH444	11/01/85	1-HGR-063-1SIH444-, DETERMINE WHETHER NUTS ARE WELDED BEHIND THE BASEPLATE RATHER THAN THE BOLT BEING ANCHORED INTO THE CONCRETE. INVEST. OF EMPLOYEE CONCERN XX8531001 CHARGE TO E 661	DETERMINE WHETHER NUTS ARE WFLDED BEHIND THE BASEPLATE RATHER THAN BEING ANCHORED IN THE CONCRETE PER EMPLOYEE CONCERN XX 85 031 001. NO NUT FND BEHIND BASEPLATE BOLTS A C D F G H ARE 2.5 " LONG B IS 2.125" LONG E IS 2.4" LONG
A549812	1	HGR	074	H63539	11/01/85	1-HGR-074-H63539-, DETERMINE WHETHER NUTS ARE WELDED BEHIND THE BASEPLATE RATHER THAN THE BOLT BEING ANCHORED INTO THE CONCRETE. INVEST. OF EMPLOYEE CONCERN XX85031001 CHARGE TO E661	DETERMINE WHETHER NUTS ARE WFLDED BEHIND THE BASE PLATE RATHER THAN ANCHORED IN THE CONCRETE PER EMPLOYEE CONCERN XX 85 031 001. BASE PLATE THICKNESS AT BOLTS B AND C IS 3/4" THE REST ARE 7/8". 1/8" GAP AT HOLE H BETWEEN BASEPLATE AND CONCRETE
A549814	1	HGR	062	1H34154	11/04/85	1-HGR-062-1H34154-, DETERMINE WHETHER NUTS ARE WELDED BEHIND THE BASEPLATE RATHER THAN THE BOLT BEING ANCHORED INTO THE CONCRETE. INVEST. OF EMPLOYEE CONCERN XX 85031001 CHARGE TO E661	DETERMINE WHETHER NUTS ARE WFLDED BEHIND THE BASE PLATE RATHER THAN BEING ANCHORED IN THE CONCRETE PER EMPLOYEE CONCERN XX 85 031 001. BOLT A WAS FND 1/8" TO SHORT REPLACE BOLTA PER MR A525606
A549815	1	HGR	062	1H34-152	11/04/85	1-HGR-062-1H34-152-, DETERMINE WHETHER	NA

COMP

MR.COMP U FUNC SYS ADDRESS. DATE.... DESCRIPTION..... CORRECTIVE ACTION.....

NUTS ARE WELDED BEHIND THE BASEPLATE  
RATHER THAN THE BOLT BEING ANCHORED INTO  
THE CONCRETE

A549816	1 HGR	063 1-SIH-28	11/04/85	1-HGR-063-1-SIH-28-, DETERMINE WHETHER NUTS ARE WELDED BEHIND THE BASEPLATE RATHER THAN THE BOLT BEING ANCHORED INTO THE CONCRETE	DETERMINE WHETHER NUTS ARE WELDED BEHIND THE BASE PLATE RATHER THAN BEING ANCHORED IN THE CONCRETE PER EMPLOYEE CONCERN XX 85 031 001. BOLT A IS 3/16" TO SHORT REPLACE PER MR A525608 NO NUT FND BEHIND BASEPLATE
A549817	1 HGR	068 1-L-360	11/04/85	1-HGR-068-1-L-360-, DETERMINE WHETHER NUTS ARE WELDED BEHIND THE BASEPLATE RATHER THAN THE BOLT BEING ANCHORED INTO THE CONCRETE INVS OF EMPLOYEE CONCERN XX-85-031-001 C	DETERMINE WHETHER NUTS ARE WELDED BEHIND THE BASE PLATE RATHER THAN BEING ANCHORED IN THE CONCRETE PER EMPLOYEE CONCERN XX 85 031 001
A549819	1 HGR	067 47A450 2	11/04/85	1-HGR-067-47A450 2-, DETERMINE WHETHER NUTS ARE WELDED BEHIND THE BASEPLATE RATHER THAN THE BOLT BEING ANCHORED INTO THE CONCRETE. INVS OF EMPLOYEE CONCERN XX-85-031-001 CHARG	DETERMINE WHETHER NUTS ARE WELDED BEHIND THE BASE PLATE RATHER THAN BEING ANCHORED IN THE CONCRETE PER EMPLOYEE CONCERN XX 85 031 001. NO NUT FND BEHIND BASEPLATE BOLTS A AND C AND D AND B TYPE 55D BOLT A IS 4.0" B IS 4.2" C AND D ARE 4.1" LONG
A549820	1 HGR	063 1SIH-806	11/04/85	1-HGR-063-1SIH-806-, DETERMINE WHETHER NUTS ARE WELDED BEHIND THE BASEPLATE RATHER THAN THE BOLT BEING ANCHORED INTO THE CONCRETE INVS OF EMPLOYEE CONCERN XX-85-031-001 C	DETERMINE WHETHER NUTS ARE WELDED BEHIND THE BASEPLATE RATHER THAN BEING ANCHORED IN THE CONCRETE PER EMPLOYEE CONCERN XX 85 031 001. NO NUT FND BEHIND BASEPLATE BOLT C 3/8" TO SHORT REPLACE PER MR A525609
A549821	1 HGR	063 HCR11355	11/04/85	1-HGR-063-HCR11355-, DETERMINE WHETHER NUTS ARE WELDED BEHIND THE BASEPLATE RATHER THAN THE BOLT BEING ANCHORED INTO THE CONCRETE. INVEST. OF EMPLOYEE CONCERN XX 85031001 CHARGE TO E661	DETERMINE WHETHER NUTS ARE WELDED BEHIND THE BASE PLATE RATHER THAN BEING ANCHORED IN THE CONCRETE PER EMPLOYEE CONCERN XX 85 031 001. NO NUT FND BEHIND BASEPLATE BOLTS A B C ARE 1.8" LONG BOLT D IS 1.5" LONG
A549822	1 HGR	063 1SIH 110	11/04/85	1-HGR-063-1SIH 110-, DETERMINE WHETHER NUTS ARE WELDED BEHIND THE BASEPLATE RATHER THAN THE BOLT BEING ANCHORED INTO THE CONCRETE INVS OF EMPLOYEE CONCERN XX-85-031-001 C	DETERMINE WHETHER NUTS ARE WELDED BEHIND THE BASEPLATE RATHER THAN BEING ANCHORED IN THE CONCRETE PER EMPLOYEE CONCERN XX 85 031 001. NO NUT FND BEHIND BASEPLATE BOLT C IS 5/8" DIA THE REST ARE 1/2" BOLTS A AND B ARE 2.2" LONG C IS 1.9" LONG

MR. COMP U FUNC SYS ADDRESS. DATE..... DESCRIPTION..... CORRECTIVE ACTION.....

A549824	1	HER	067	47A450-2	11/04/85	1-HGR-067-47A450-2-, DETERMINE WHETHER NUTS ARE WELDED BEHIND THE BASEPLATE RATHER THAN THE BOLT BEING ANCHORED INTO THE CONCRETE INVEST. OF EMPLOYEE CONCERN XXX-85-031-001	DETERMINE WHETHER NUTS ARE WELDED BEHIND THE BASEPLATE RATHER THAN BEING ANCHORED IN THE CONCRETE PER EMPLOYEE CONCERN XX 85 031 001. NO NUT FND BEHIND BASEPLATE BOLTS A C D ARE 2.5" LONG B IS 2.8" LONG
A549825	1	HER	087	1H45-95	11/04/85	1-HGR-087-1H45-95-, DETERMINE WHETHER NUTS ARE WELDED BEHIND THE BASEPLATE RATHER THAN THE BOLT BEING ANCHORED INTO THE CONCRETE INVEST. OF EMPLOYEE CONCERN XX-85-031-001 C	DETERMINE WHETHER NUTS ARE WELDED BEHIND THE BASEPLATE RATHER THAN ANCHORED IN THE CONCRETE PER EMPLOYEE CONCERN XX 85 031 001. NO NUT FND BEHIND BASEPLATE BOLT D IS 5/8" DIA THE REST ARE 3/4" BOLTS A B C ARE 3.5" LONG D IS 3/4" LONG
A549826	1	HER	063	1-S14-73	11/04/85	1-HGR-063-1-S14-73-, DETERMINE WHETHER NUTS ARE WELDED BEHIND THE BASEPLATE RATHER THAN THE BOLT BEING ANCHORED INTO THE CONCRETE INVEST. OF EMPLOYEE CONCERN XX-85-031-001 C	DETERMINE WHETHER NUTS ARE WELDED BEHIND THE BASEPLATE RATHER THAN BEING ANCHORED IN THE CONCRETE PER EMPLOYEE CONCERN XX 85 031 001. NO NUT FND BEHIND BASEPLATE AT HOLE C IS 1" THE REST ARE 3/4" BOLTS A B D ARE 3.9" LONG C IS 3.4" LONG
A549827	1	HER	030	47W60032	11/04/85	1-HGR-030-47W60032-, DETERMINE WHETHER NUTS ARE WELDED BEHIND THE BASEPLATE RATHER THAN THE BOLT BEING ANCHORED INTO THE CONCRETE. INVEST. OF EMPLOYEE CONCERN XX8531001 CHARGE TO E661	DETERMINE WHETHER NUTS ARE WELDED BEHIND THE BASE PLATE RATHER THAN BEING ANCHORED IN THE CONCRETE PER EMPLOYEE CONCERN XX 85 031 001. NO NUTS BEHIND BASEPLATE
A549828	1	HER	015	1H47107	11/04/85	1-HGR-015-1H47107-, DETERMINE WHETHER NUTS ARE WELDED BEHIND THE BASEPLATE THAN THE BOLT BEING ANCHORED INTO THE CONCRETE. INVEST. OF EMPLOYEE CONCERN XX85031001 CHARGE TO E661	NA
A549829	1	HER	015	H47107	11/04/85	1-HGR-015-H47107-, DETERMINE WHETHER NUTS ARE WELDED BEHIND THE BASEPLATE RATHER THAN THE BOLT BEING ANCHORED INTO THE CONCRETE. INVEST. OF EMPLOYEE CONCERN XX 85031001 CHARGE TO E661	NA
A549830	1	HER	062	CVC857	11/04/85	1-HGR-062-CVC857-, DETERMINE WHETHER NUTS ARE WELDED BEHIND THE BASEPLATE RATHER THAN THE BOLT BEING ANCHORED INTO THE CONCRETE. INVEST. OF EMPLOYEE CONCERN XX85031001 CHARGE TO E661	DETERMINE WHETHER NUTS ARE WELDED BEHIND THE BASEPLATE RATHER THAN ANCHORED IN THE CONCRETE PER EMPLOYEE CONCERN XX 85 031 001. NO NUT FND BEHIND BASEPLATE BOLTS A AND D ARE 1.5" LONG B AND C ARE 1.8" LONG



COMP

MR.	COMP	U	FUNC	SYS	ADDRESS	DATE	DESCRIPTION	CORRECTIVE ACTION
A549831	1	HGR	062	1CVCH-86	11/19/85	1-HGR-062-1CVCH-86-, DETERMINE WHETHER NUTS ARE WELDED BEHIND THE BASEPLATE RATHER THAN THE BOLT BEING ANCHORED INTO THE CONCRETE	NA	
A549834	1	HGR	067		11/04/85	1-HGR-067--, DETERMINE WHETHER NUTS ARE WELDED BEHIND THE BASEPLATE RATHER THAN THE BOLT BEING ANCHORED INTO THE CONCRETE. INVEST. OF EMPLOYEE CONCERN XX85031001 CHARGE TO E661	DETERMINE WHETHER NUTS ARE WFLDED BEHIND THE BASE PLATE RATHER THAN BEING ANCHORED IN THE CONCRETE PER EMPLOYEE CONCERN XX 85 31 001. BOLTS A AND B ARE 4.3" LONG. BOLTS C AND D ARE 4.4" LONG	
A550569	1	FCV	062		177 11/07/85	1-FCV-062-177-, VLV LEAKS AND WTR SPRAYS OUT OF THE SEAL	VLV LEAKS DUE TO WORN DIAP. INSTALLED NEW DIAP	
A561756	1	VLV	067	508A	11/17/85	1-VLV-067-508A-, INSPECT VLV IN ACCORDANCE W/SI 166.37	REMOVED VLV IN ACCORDANCE W/SI 166.37 FOR ENRG TEST TO CHECK	
A561757	1	VLV	067	513A	11/17/85	1-VLV-067-513A-, INSPECT VLV IN ACCORDANCE W/SI 166.37	REMOVED VLV IN ACCORDANCE W/SI 166.37 FOR ENRG TEST TO CHECK	
A561761	2	VLV	067	512A	11/17/85	2-VLV-067-512A-, INSPECT VLV IN ACCORDANCE W/SI 166.38	INSPECT VLV PER SI 166.38	
A561763	2	VLV	067	517A	11/17/85	2-VLV-067-517A-, INSPECT VLV IN ACCORDANCE W/SI 166.38	INSPECT VLV PER SI 166.38	

44 records listed.



COMP

MR.HIST U FUNC SYS ADDRESS. DATE.... DESCRIPTION..... CORRECTIVE ACTION.....

A549033	0	VLV	026	1036	11/12/85	0-VLV-026-1036-, DRIP CK VLV IS LEAKING THRU AND HANDLE IS MISSING	VLV LEAKING THRU DUE TO WORN INTERNALS. INSTALLED NEW VLV
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One record listed.

COMP

MR2.... U FUNC SYS ADDRESS. DATE.... DESCRIPTION..... CORRECTIVE ACTION.....

A521089	1	VLV	067	566C	11/02/85	1-VLV-067-566C--,RV LEAKING THRU. REPAIR	LEAKING THRU DUE TO DIRTY INTERNALS. CLEANED INTERNALS RETURNED TO SERV.
A521092	1	VLV	062	505	11/01/85	1-VLV-062-505--,SET VLV AT 220 PSIG PER SI 164 INST CHANGE 85 1213.VLV WAS SET AT 150 PSIG PER MR A521090 & HAS NOT BEEN RETURNED TO SERV.ERROR IDENTIFIED ON DWG 47W811-1	SET VLV AT 220 PSIG PER SI 164
A533170	0	VLV	078		11/17/85	0-VLV-078--,REMOVE VLV,TEST AND REPAIR THIS MUST BE WORKED WHILE THE SPENT FUEL PIT GATE IS OUT.CONTACT TIM MASSEY @ 6869 WHEN READY TO WORK	REMOVE VLV TEST AND REPAIR. LAPPED SEATS,CLEANED INTERNALS;SET VLV AT 30 PSIG

3 records listed.

## SUMMARY OF WORK COMPLETED

### MODIFICATIONS

NOVEMBER 1985

#### NUREG 0588

##### ECN 6231 - Remove Interferences

ERCW hangers and piping located in the annulus were removed for access to the motor operators. Efforts were initiated with OE to redesign the supports and piping configuration.

See attached listing.

#### APPENDIX R

##### ECNs 5265, 5435, and 6343 - Fire Doors

The postmodification test was performed with excess leakage found around the door latch and corners. Resolution is continuing.

##### ECN 6235 - Reroute Various Cables

Work is on hold for the nine workplans previously in work. Workplan writing is on hold.

##### ECN 6305 - Elevation 714 Fire Barrier

Framing and penetration work is near completion. Installation of fire board has not begun.

##### ECN 6311 - Operator Extension on PORV

Installation work was started on both units.

##### ECN 6315 - Replace Fuses

No work was performed this month.

##### ECN 6319 - Fire Protection Piping

Installation of piping and hangers was resumed.

#### OTHER ITEMS

##### ECN 5009 - ERCW Piping Changeout From Carbon Steel to Stainless Steel

Unit 1 upper compartment cooler work was completed. Prefabrication activities for unit 2 work in the same area were started. The auxiliary air compressor piping changeout was completed. An additional hanger is being installed.

## OTHER ITEMS (Continued)

### ECNs 5034, 5713, and 5743 - Various Platforms in Lower Containment

Platform work between Nos. 1 and 4 steam generators was completed, and the application of protective coating was begun. Prefabrication and installation activities between Nos. 2 and 3 steam generators continue. Work will be discontinued during CILRT.

### ECN 5198 - Technical Support Center

SPDA consoles were installed in accordance with ECN 6152.

### ECN 5200 - Postaccident Sampling Facility

No work was performed this period.

### ECN 5202 - Fifth Diesel Generator

The tape coat was satisfactorily tested. The workplan was approved for backfilling, which will begin in early December. Workplans are being prepared for electrical interface work.

### ECN 5237 - Laundry Facility

No work was performed this period.

### ECN 5252 - Label Node Voltages in Manholes

No progress on this job this period; four manholes remain.

### ECN 5347 - Replace Doors C-49 and C-50 (Electrical Portion)

This work is on hold until door C-50 passes the leak test.

### ECN 5373 - Condensate Demineralizer Air Compressor

The decision was made to modify the existing system tie-in points for the air. OE is working on the revised design. Installation will follow.

### ECN 5620 - Add Instrumentation for Auxiliary Feedwater Pump

Work is on hold.

### ECN 5645 - Replacement of Flow-Control Valve 2-329

The vendor representative is expected the first week of December to modify the valve.

### ECN 5657 - Installation of MSR Drain Valves

The unit 2 workplan to install approximately 150 drain valves is in the approval cycle. Insulation reinstallation on unit 1 still remains incomplete.

## OTHER ITEMS (Continued)

### ECN 5795 - Field Services Building

Fire detection system work is on hold for materials.

### ECN 5914 - Improve Reliability of Steam Dump

Conduit is complete; valve remains to be wired in.

### ECNs 5938 and 6506 - Feedwater Heater Replacement

Installation activities are continuing on unit 1. All large piping has been completed on unit 2. Installation of miscellaneous small piping continues, and insulation work on unit 2 continues.

### ECN 6057 - Cable Tray Covers

Approximately 240 out of 290 cable tray covers have been remanufactured or replaced.

### ECN 6147 - Airlock Packing Nut

The installation of the antirotation devices on the unit 1 airlock shaft packing nuts was completed.

### ECN 6152 - SPDS

ASE consoles have been installed. One console in each horseshoe has been installed.

### ECN 6196 - Pressurizer Hangers and Valves

All hangers except one have been completed. Insulation work has been completed as far as possible until ECN 6495 has been completed. The remaining work will be completed after CILRT.

### ECN 6204 - Electrical Penetration Overcurrent Protection

Fuse replacement and fuse block installation are complete. A technical specification change to place the circuits in operation has been received.

### ECNs 6251 and 6532 - Waste Disposal Hangers

OE has determined that a review of an additional support was required on each unit. The unit 1 work was completed. OE is preparing a unit 2 drawing for issue. Workplan preparation will continue upon receipt.

### ECN 6259 - Moisture Separator Reheater Tube Bundle Replacement

SECO completed their work and left the site. The reinstallation of the doghouses was started. Insulation work was begun.

OTHER ITEMS (Continued)

ECN 6263 - Feedwater Hangers Lower Containment

One support was completed, and work continues on the others.

ECN 6352 - Fire Protection Isolation Valves

No work was performed this period.

ECNs 6402 and 6439 - Pressurizer Instrumentation Relocation

The unit 1 piping is complete through the second valve, allowing sweep and vent activities to continue. The remaining piping will be completed after CILRT.

ECN 6417 - Install Alternate Seal Water for Pumps, CDWE

Electrical drawings remain to be issued.

ECN 6491 - ERCW Supports

The reanalysis of the ERCW support scheme was completed with approximately 47 supports requiring modification. Workplans were written, approved, and placed in work status for 20 supports. Workplan preparation continues for the remaining supports.

ECN 6494 - ERCW Pipe Replacement at CCW Heat Exchangers

All piping work has been completed. The changeout of the deteriorated valve is scheduled to be performed when the CCW heat exchanger retube work is performed.

ECN 6495 - Modification of Pressurizer Pots

This workplan was approved and placed in work status. The root valve has been replaced, the core drills made, and the piping work started. This work will be discontinued during CILRT.



Date: 12/3/85

SCR No.	Description	Memo	ECN	Issued	Drawings	Engineer	Workplan No.	Estimated Date of Completion		Comments
								U-1	U-2	
EQP 8501	Disconnect 1- & 2-HS-62-61	9/18	6524	Yes	11/29-30			12/31	12/24	Drawings issued.
EQP 8502	Replace penetrations 23, 48	9/13	6490	Yes	Yes	Peters	11801, 11802, 11810, 11811	12/8	12/8	Termination of cables is impacting schedule.
EQP 8503	Relocate RE-90-273, -274	9/25	6500	Yes	Yes	Peters	11810, 11811	12/8	N/A	REs are relocated. Functional test is constrained by penetration work.
EQP 8504	Splice methods not correct	10/2	N/A	N/A	N/A	Stockton		C	C	Verifying completion.
EQP 8505	Drawing	10/11	N/A	N/A	N/A	N/A	N/A	N/A	N/A	
EQP 8506	Seal cntmt isolation valve	10/17	6514	Yes	Yes	Kimsey		12/20	12/15	Writing workplan.
EQP 8507	Rewire MOV	10/29	N/A	N/A	N/A	Rutledge		12/31	12/23	Rewired-55. Functioned-1. MOVATS-___.
EQP 8508	JB weepholes (press)	10/17	6523		12/12			12/24	12/17	
EQP 8509R1	Conduit seals	11/6	6529	11/29	12/17			12/19	12/12	
EQP 8510	Disconnect local hand-switches	11/12	6527		12/8			1/4	12/29	
EQP 8511	Submerged JB inside cntmt	11/13								No longer an issue.
EQP 8512R2	Rewire JB	N/A	N/A	N/A	N/A	Amburn		12/24	12/16	U-2 58/83, U-1 7/91.
EQP 8513	Weep holes (moisture)	11/14	6547	11/29	12/12			12/23	12/16	
EQP 8514	Motor insulation 74-1, -2	11/14	6540		12/3			N/A	12/24	Motors expected 11/28.
EQP 8515	Replace 2-PDT-30-43	11/15	6546		12/8			N/A	12/30	This ECN will resolve 8514 and 8529.
EQP 8516	Replace 2-LT-3-174	11/15	N/A			Instrument Maintenance		N/A	12/5	To be done by MR.

Date: 12/3/85

SCR No.	Description	Memo	ECN	Issued	Drawings	Engineer	Workplan No.	Estimated Date of Completion		Comments
								U-1	U-2	
								N/A	12/30	12/5 Failure evaluation.
EQP 8517	ABCTS humidity control							12/31	12/31	38/unit. Writing workplans.
EQP 8518	Submerged cables	11/15	6533	11/27	11/27			12/23	12/23	22/23.
EQP 8519	Tee drains	11/22	N/A	N/A	N/A	Electrical Maintenance		1/13	1/13	5 cables/unit.
EQP 8520	Expired cables	11/22	6553	12/5	12/12			12/26	12/26	
EQP 8521	Delete TB and rework splices	11/22	6550		12/11			12/17	12/17	Writing workplan to cover 8522, 8523, and 8527.
EQP 8522	Rewire local panels	11/26				Stockton		12/20	12/20	Writing workplan to cover 8522, 8523, and 8527.
EQP 8523	Missing bolts and washers and misplaced brackets	N/A	N/A (MR)	N/A	N/A	Stockton		12/24	12/24	
EQP 8524	Change setpoints	11/26	6551	12/6	12/13	Instrument Maintenance		12/20	12/20	
EQP 8525	Retermine hydrogen recombiner	N/A	N/A (MR)	N/A	N/A	Electrical Maintenance			12/30	
EQP 8526	Replace FSVs, U-1 1, U-2 11	11/26	6552		12/11			12/9	12/9	Writing workplan to cover 8522, 8523, and 8527.
EQP 8527	Coat TB, U-1 3, U-2 8	N/A	N/A (MR)	N/A	N/A	Stockton		12/10	12/10	Work in progress.
EQP 8528	Solder strain gauge Barton transmitters.	11/20	IM1			Instrument Maintenance		12/30	12/30	
EQP 8529	PDT-30-42, -43 capacitor	11/26	6546		12/8			12/15	12/15	38 MRs written.
EQP 8530	Gasket, Namco L/S	N/A	N/A (MR)	N/A	N/A	Electrical Maintenance				

Date: 12/3/85

SCR No.	Description	Memo	ECN	Issued	Drawings	Engineer	Workplan No.	Estimated Date of Completion		Comments
								U-1	U-2	
EQP 8531	Delete MOV heaters		6544	11/27	12/9	Rutledge		12/31	12/23	Incorporated in MOV rewire workplan. Drawings are FCR'd.
EQP 8532	Delete L/S 1-, 2-43-201, -202, -207, and -208									
EQP 8533	Delete dual voltage splice	:	N/A	N/A	N/A	Rutledge		12/31	12/23	Incorporated in MOV rewire workplan.
EQP 8534	Respace valve positioner 3-174 and -175									
EQP 8535	Replace limit switches, U-1 10, U-2 12.									

Date: 12/3/85

SCR No.	Description	Memo	ECN	Issued	Drawings	Engineer	Workplan No.	Estimated Date of Completion		Comments
								U-1	U-2	
N/A	Move surge suppression network for PORV	N/A	5773	Yes	Yes	Kimsey		12/6	12/6	Workplan in approval cycle 12/3/85.
EEB 8523	Penetration overcurrent protection		6204 6219 6452	Yes	Yes	Legg		12/31	12/31	6204 and 6219 are complete. Writing workplan on 6452. Material expected 12/16/85.
N/A	Work FCR to delete 1, 2-PS-3-160A, -160B, -165A, and -165B	N/A	5883	Yes	Yes	Hall		11/27	11/27	Instrument Maintenance calibrating.
N/A	Replace 1-FT-1-3A, -3B, -10A, -10B, -21A, -21B, -28A, -28B	N/A	6347	Yes	Yes	Instrument Maintenance		?	N/A	
NEB 8510	Relocate LT-68-320, PT-68-323; -320	N/A	6439	U-1 U-2	U-1	Carrasquillo Peters		12/20	12/31	U-1 conduit in progress. U-2 in submerged cable workplan.
	Remount 63-71, 68-308	N/A	6496	Yes	Yes	Legg	11865	11/27	12/5	U-1 63/71 complete. Making brackets for others.
	Replace LS-65-4, -5	N/A	6504	Yes	Yes	Legg	11865	11/27	12/5	
MEB 8410R3	Replace LS-77-127	11/22	6525	Yes	Yes	Legg	11865	11/27	12/5	
	Delete brakes FCV-62-61		6521	Yes	12/3			12/31	12/23	Motor onsite. Gears on way. Expected delivery date was 11/27/85.

SEQUOYAH NUCLEAR PLANT  
FUEL CYCLE DESIGN BASIS INFORMATION SHEET

Unit 1

	<u>Cycle 3</u>	<u>Cycle 4</u>	<u>Cycle 5</u>	<u>Cycle 6</u>	<u>Cycle 7</u>
Operating Cycle Length (Days)	492	501	503	502	499
*Operating Capacity Factors	76	85	85	85	85
Scheduled Mid-cycle Outage Days	37	17	17	17	17
Estimated Coastdown Days	1	6	3	2	0
Core Full Power Days	345	405	410	410	410

Unit 2

	<u>Cycle 3</u>	<u>Cycle 4</u>	<u>Cycle 5</u>	<u>Cycle 6</u>	<u>Cycle 7</u>
Operating Cycle Length (Days)	576	510	502	501	506
*Operating Capacity Factors	64	85	85	85	85
Scheduled Mid-cycle Outage Days	0	17	17	17	17
Estimated Coastdown Days	5	9	2	1	6
Core Full Power Days	364	410	410	410	410

ATP:CCM 12/03/85

SEQUOYAH NUCLEAR PLANT  
DATE SUMMARY OF UPCOMING OUTAGES

<u>Unit 1</u>	<u>Start Date</u>	<u>Duration (Days)</u>	<u>Finish Date</u>
Initial Criticality	07/05/80		
U1, C3	08/22/85	158*	01/27/86*
Surveillance/Ice	03/13/87	17	03/30/87
U1, C4	06/12/87*	50	08/01/87*
Surveillance/Ice	09/23/88	17	10/10/88
U1, C5	12/16/88*	51	02/05/89*
Surveillance/Ice	03/09/90	17	03/26/90
U1, C6	06/22/90*	75	09/05/90*
Surveillance/Ice	09/27/91	17	10/14/91
U1, C7	01/17/92*	45	03/02/92*

<u>Unit 2</u>	<u>Start Date</u>	<u>Duration (Days)</u>	<u>Finish Date</u>
Initial Criticality	11/05/81		
U2, C3	07/25/86*	50	09/13/86*
Surveillance/Ice	09/18/87	17	10/05/87
U2, C4	02/05/88*	51	03/27/88*
Surveillance/Ice	03/10/89	17	03/27/89
U2, C5	08/11/89*	45	09/25/89*
Surveillance/Ice	09/28/90	17	10/15/90
U2, C6	02/08/91*	75	04/24/91*
Surveillance/Ice	03/27/92	17	04/13/92
U2, C7	09/11/92*	45	10/26/92*

NOTE: The unit 2 schedule assumes a 01/23/86 return-to-service from the forced outage that began on 08/21/85.

\*Denotes changes since last update.

ATP:CCM 12/03/85



TENNESSEE VALLEY AUTHORITY

Sequoyah Nuclear Plant  
P. O. Box 2000  
Soddy-Daisy, Tennessee 37379

December 13, 1985

Nuclear Regulatory Commission  
Office of Management Information  
and Program Control  
Washington, DC 20555

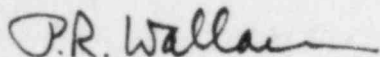
Gentlemen:

SEQUOYAH NUCLEAR PLANT - MONTHLY OPERATING REPORT - NOVEMBER 1985

Enclosed is the November 1985 Monthly Operating Report to the NRC for Sequoyah Nuclear Plant.

Very truly yours,

TENNESSEE VALLEY AUTHORITY



P. R. Wallace  
Plant Manager

Enclosure

cc (Enclosure):

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Nuclear Regulatory Commission  
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Atlanta, GA 30323 (1 copy)

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Sequoyah Nuclear Plant

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