



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

NORTHEAST NUCLEAR ENERGY COMPANY

DOCKET NO. 50-245

MILLSTONE NUCLEAR POWER STATION, UNIT 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No.100
License No. DPR-21

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Northeast Nuclear Energy Company (the licensee) dated September 5, 1996, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended as indicated in the attachment to this license amendment, and paragraphs 2.C.(2) and 2.C.(5) of Facility Operating License No. DPR-21 are hereby amended to read as follows:

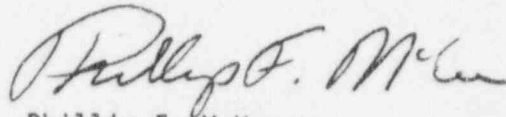
(2) Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 100, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

(5) Deleted

3. This license amendment is effective as of the date of issuance, to be implemented within 60 days of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Phillip F. McKee
Deputy Director for Licensing
Special Projects Office
Office of Nuclear Reactor Regulation

Attachment: Page 4 of Facility Operating License
(included for convenience)

Date of Issuance: April 15, 1997

ATTACHMENT TO LICENSE AMENDMENT NO. 100

FACILITY OPERATING LICENSE NO. DPR-21

DOCKET NO. 50-245

Replace the following page of Operating License No. DPR-21 with the attached page. The revised page is identified by amendment number and contains vertical lines indicating the areas of change.

Remove

4

Insert

4

November 9, 1995

- 4 -

(5) Deleted

(6) Spent Fuel Pool Operations

NNECO shall conduct refueling operations that include full-core offload as a normal end-of-cycle event in accordance with the controls proposed in Attachment 4 of the application for amendment dated July 28, 1995, as supplemented September 12, 1995.

NNECO shall update the UFSAR to reflect the hardware modifications, heat load analysis, and operational controls consistent with the application for amendment dated July 28, 1995, as supplemented September 12, 1995, in accordance with 10 CFR 50.71(e).

D. The facility has been granted certain exemptions from the requirements of 10 CFR Part 50 as set forth below:

- (1) Section III.G of Appendix R to 10 CFR Part 50, "Fire Protection Program for Nuclear Power Facilities Operating Prior to January 1, 1979." This section relates to fire protection features for ensuring the systems and associated circuits used to achieve and maintain safe shutdown are free of fire damage. The staff safety evaluation, dated November 6, 1985, concluded that the licensee's existing fire-protection configuration with proposed modifications achieves an equivalent level of safety. Exemption granted November 6, 1985.
- (2) Appendix J to 10 CFR Part 50, "Primary Reactor Containment Leakage Testing for Water-Cooled Power Reactors." Appendix J relates to containment leakage test requirements, specifically periodic verification by tests of the leak-tight integrity of the primary reactor containment and systems and components which penetrate containment. Three exemptions were granted on May 10, 1985, on the basis of the staff safety evaluation. The two which remain in effect relate to testing of expansion bellows at containment penetrations and main steam isolation valves.