

April 10, 1997

Mr. Joseph J. Hagan
Vice President, Operations GGNS
Entergy Operations, Inc.
P. O. Box 756
Port Gibson, MS 39150

SUBJECT: REQUEST FOR ADDITIONAL INFORMATION RELATED TO PRESSURE-TEMPERATURE
LIMITS IN THE TECHNICAL SPECIFICATIONS, GRAND GULF NUCLEAR STATION,
UNIT 1 (TAC NO. M97520)

Dear Mr. Hagan:

The staff is continuing its review of your application dated October 22, 1996, for an amendment to the pressure-temperature limit curves in the Technical Specifications for Grand Gulf Nuclear Station, Unit 1. To permit us to continue our review on our current schedule, we require that the information requested in the enclosure be provided within 45 days of your receipt of this letter.

Sincerely,

Jack N. Donohew 4/10/97
Jack N. Donohew, Senior Project Manager
Project Directorate IV-1
Division of Reactor Projects III/IV
Office of Nuclear Reactor Regulation

Docket No. 50-416

Enclosure: Request for Additional Information

cc w/encl: See next page

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

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Vice President, Operations GGNS
Entergy Operations, Inc.
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Sincerely,

A handwritten signature in cursive script, reading "Jack N. Donohew", is positioned above the typed name and title.

Jack N. Donohew, Senior Project Manager
Project Directorate IV-1
Division of Reactor Projects III/IV
Office of Nuclear Reactor Regulation

Docket No. 50-416

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Mr. Joseph J. Hagan
Entergy Operations, Inc.

Grand Gulf Nuclear Station

cc:

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REQUEST FOR ADDITIONAL INFORMATION
RELATED TO PRESSURE-TEMPERATURE LIMITS IN TECHNICAL SPECIFICATIONS
ENTERGY OPERATIONS, INC.
GRAND GULF NUCLEAR STATION, UNIT 1
DOCKET NO. 50-416

The following request for additional informations is concerned with the licensee's letters dated May 2, 1996, (GNRO-96/00035) and October 22, 1996, (GNRO-96/00120), for an amendment to the pressure-temperature limit curves in the Technical Specifications (TSs) for Grand Gulf Nuclear Station, Unit 1.

1. **General question:** Provide the methodology used to generate pressure-temperature (P-T) limits for the bottom head and the feedwater nozzle.
2. **Page 2 of 8:** The detailed information needed to calculate the reported adjusted reference temperature (ART) of 57.57 DEG F can be found in your letter of May 2, 1996, from which the internal-diameter (ID) fluence of $2.5E18$ n/cm² at 32 EFPY was reported. The May 2, 1996, letter indicated that this fluence value was from General Electric (GE) Report EAS-35-0387 (April 1987). The ID fluence value at 32 EFPY in the NRC reactor vessel integrity database (RVID), is 3.11 n/cm², which was reported to be the latest data in your letter dated May 5, 1994, in response to the NRC close-out letter regarding GL 92-01. The latter value is in line with all your recent submittals. Resolve this discrepancy.
3. **Page 3.4-31 to 3.4-35 of the Technical Specifications:** Change of fluence should not affect the P-T limit curves which are controlled by the feedwater nozzle, as evidenced in the five figures of Figures 3.4.11-1. Explain the discrepancy between the current curves that are controlled by the feedwater nozzle for 10 EFPYs and the proposed curves that are controlled by the feedwater nozzle for various EFPY intervals.
4. **Page 3.4-31 to 3.4-32 of the Technical Specifications:** Change of fluence should not affect the P-T limit curves for the bottom head; however, the curves vary among the current Figure 3.4.11-1 and the first two figures of the proposed Figure 3.4.11-1. Provide an explanation for this variation.

ENCLOSURE