

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1)
Shoreham Nuclear Power Station Unit #1DOCKET NUMBER (2)
0 5 0 0 0 3 2 2 1 OF 0 2

TITLE (4)

Procedural Error in Primary Containment Air Sampling

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)							
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES		DOCKET NUMBER(S)					
0	1	1	6	8	6	8	6	0	0	1	0	0	0	0	0	
0	1	1	6	8	6	0	0	1	0	0	0	2	1	1	8	6

OPERATING MODE (9)	THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR § (Check one or more of the following): (11)									
POWER LEVEL (10)	20.402(b)	20.406(e)	80.73(a)(2)(i)-(v)	73.7(b)						
	20.406(a)(1)(i)	80.38(a)(1)	80.73(a)(2)(v)	73.71(e)						
	20.406(a)(1)(ii)	80.38(a)(2)	80.73(a)(2)(vi)	OTHER (Specify in Abstract below and in Text, NRC Form 366A)						
	20.406(a)(1)(iii)	80.73(a)(2)(i)	80.73(a)(2)(viii)(A)							
	20.406(a)(1)(iv)	80.73(a)(2)(ii)	80.73(a)(2)(viii)(B)							
	20.406(a)(1)(v)	80.73(a)(2)(iii)	80.73(a)(2)(ix)							

LICENSEE CONTACT FOR THIS LER (12)
NAME
Robert W. Grunseich, Operational Compliance EngineerTELEPHONE NUMBER
AREA CODE
5 1 6 9 2 9 - 8 3 0 0

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)									
CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRC	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRC

SUPPLEMENTAL REPORT EXPECTED (14)
YES (If yes, complete EXPECTED SUBMISSION DATE) ☐ NO ☒ X
EXPECTED SUBMISSION DATE (15)
MONTH DAY YEAR

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single space typewritten lines): (16)

On January 16, 1986 at 1500, the Radiochemistry Department discovered that the primary containment air sampling procedure did not adequately meet the requirements set forth in the Technical Specifications. The plant was in Operational Condition 5 with the mode switch in shutdown during an outage and all control rods inserted in the core. Sampling of primary containment air required by Technical Specification sections 3.11.2.1 and 3.11.2.8 must be analyzed for noble gases, particulates and iodines. However, the air samples were analyzed after the air had passed through particulate and charcoal filters. The sampling and analysis was performed by radiochemistry personnel in accordance with an approved station procedure, SP 74.020.04 (Containment Drywell Exhaust and Air Removal Pump Exhaust Analysis and Discharge Authorization). This procedure will be revised to meet the surveillance requirements of the Technical Specifications prior to the performance of the next test.

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LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

U.S. NUCLEAR REGULATORY COMMISSION

APPROVED OMB NO 3150-0104
EXPIRES 8/31/85

FACILITY NAME (1)	DOCKET NUMBER (2)	LER NUMBER (6)			PAGE (3)	
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER		
Shoreham Nuclear Power Station Unit #1	01500032286-001-				02	OF 02

TEXT (If more space is required, use additional NRC Form 288A's) (17)

On January 16, 1986 at 1500, the Radiochemistry Department discovered that the primary containment air sampling procedure did not adequately meet the requirements set forth in the Technical Specifications. The plant was in Operational Condition 5 with the mode switch in shutdown during an outage and all control rods inserted in the core. Primary containment was open at the time to perform plant modifications. Plant Management was notified of the event and the NRC was notified at 1437 the next day.

Technical Specification 3.11.2.1 requires that the containment air be sampled and analyzed on a monthly basis for particulates, iodines and noble gases. However, in accordance with the sampling procedure, SP74.020.04, the analysis was performed by taking an air sample downstream of the RMS skid (1D11*PNL-061) after passing through a particulate filter and a charcoal filter. The sample is then placed in a Gelli recorder for a complete isotopic analysis. Although the grab samples were taken and analyzed in accordance with the Tech Specs (as indicated in table 4.11.2.1.7) they did not fully meet the requirements because the iodines and particulates had been filtered out prior to the analysis. This procedure had been used for previous monthly sampling per Tech. Spec. 3.11.2.1 and for sampling done prior to and during purging and venting as required by Tech. Spec. 3.11.2.8. The noble gas analysis was done as required. Upon discovery of the event, grab air samples were taken and analyzed for particulates and iodines in accordance with an approved station procedure, SP 62.030.01 (Airborne Activity Survey Techniques and Determination). There is no safety significance to this event, because the drywell air is continuously monitored. Any time containment was purged or vented, the air was exhausted through the Reactor Building Ventilation Exhaust System (RBNVS) and was also sampled by the normal station vent monitors. These are Particulate, Iodine and Gaseous (PIG) monitors. No abnormal readings have occurred.

The sampling and analysis was performed by radiochemistry personnel in accordance with an approved station procedure. Sampling and analysis for noble gases will remain the same, by procedure, but the station procedure will be revised to change the method of sampling and analysis for particulates and iodines. The existing filters (charcoal and particulate) in the sample skid will be replaced with "grab sample" filters. After the sample air passes through the filters for a specified collection time, the filters are removed and analyzed for particulates and iodines. This procedure change will be incorporated into the procedure prior to the performance of the next surveillance test as required by the Technical Specification.