Arizona Nuclear Power Project
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U. S. Nuclear Regulatory Commission
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Subject: Palo Verde Nuclear Generating Station (PVNGS)
Unit 1
Docket No. STN 50-528, License No. NPF-41
Report of Notification of Unusual Event - Reactor Trip
File: 86-056-026; 86-020-404; G.1.01.10

Dear Sirs:

Attached please find a report describing a Notification of Unusual Event at the Palo Verde Nuclear Generating Station on February 3, 1986. This report addresses a reactor trip resulting from a low level in Steam Generator Number 2.

This report is prepared and submitted pursuant to Table 5.3.-1 of the PVNGS Emergency Plan Rev. 6. By copy of this letter, we are also forwarding a copy of the report to the Regional Administrator of the Region V Office and other offsite authorities.

If you have any questions or concerns, please contact Mr. W. F. Quinn of my staff.

Very truly yours,

E. E. Van Brunt

E. E. Van Brunt, Jr.
Executive Vice President
Project Director

EEVB/RAB/dlm
Attachment

cc: J. B. Martin (all w/a)
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Attachment
to ANPP-34968

PALO VERDE NUCLEAR GENERATING STATION UNIT 1

NOTIFICATION OF UNUSUAL EVENT OF FEBRUARY 3, 1986

Docket No. 50-528
License No. NPF-41
Special Report No. 1-SR-86-006

At 1207 MST on February 3, 1986, the NRC Operations Center was notified via the Emergency Notification System, of the declaration of a NOTIFICATION OF UNUSUAL EVENT for Unit 1 of the Palo Verde Nuclear Generating Station. The NOTIFICATION OF UNUSUAL EVENT was declared pursuant to the Palo Verde Emergency Plan, which requires the reporting of a reactor trip which is complicated by concurrent or subsequent events or conditions.

At approximately 1145, the plant was in Mode 1 at 60% reactor power, with pressure at 2225 psia and Reactor Coolant System cold leg temperature of 564 degrees F, when a reactor trip occurred due to low level in Steam Generator (SG) Number 2.

Main Feedwater Pump (MFP) "A" was operating, and MFP "B" was shutdown for maintenance. S.G. #1 had an abnormal blowdown to the condenser, and S.G. #2 had a high rate blowdown to the condenser when the primary side reactor operator noticed an increase in cold leg temperature as well as a Control Element Assembly (CEA) insertion demand. When informed of these changes, the secondary side reactor operator noted steady main generator load but decreasing S.G. level.

Feedwater to Main Steam differential pressure was noted to be approximately 30 psi low with both economizer feed regulating valves wide open. An attempt was made to increase the differential pressure by increasing MFP "A" speed through utilization of the speed bias on the MFP speed controller. Bias adjustment had no effect on pump speed, and the pump remained steady at approximately 4500 RPM. An unsuccessful attempt was made to match deviation in order to transfer MFP control to manual, and the reactor tripped on low S.G. #2 level.

Due to a double quick open of the Steam Bypass valves and the high flow rates of S.G. blowdown, a Safety Injection Actuation Signal (SIAS), Containment Isolation Actuation Signal (CIAS) and a Main Steam Isolation Signal (MSIS) occurred. Due to the actuation of safety systems and the classification per Emergency Classification EPIP-02, a Notification of Unusual Event was declared at 1207 and appropriate notifications were made.

All safety equipment functioned as designed and no safety limits were violated. Following the MSIS, plant parameters stabilized normally and the Notification of Unusual Event was terminated at 1311.

A Licensee Event Report will be submitted within 30 days to further describe this event.

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