September 30, 1985

Docket No. 50-293

Mr. William D. Harrington
Senior Vice President, Nuclear
Boston Edison Company
800 Boylston Street
Boston, Massachusetts 02199

Dear Mr. Harrington:

SUBJECT: NUREG-0737, ITEM II.B.3, POST-ACCIDENT SAMPLING

Re: Pilgrim Nuclear Power Station, Unit 1

We have reviewed the revised procedure for estimating core damage, which was submitted with your letter of September 9, 1985, and we have determined that it meets the remaining criterion number 2 of NUREG-0737, Item II.B.3. It is, therefore, acceptable. This completes our review of the Pilgrim Nuclear Power Station against the requirements of Item II.B.3 of NUREG-0737. We conclude that Item II.B.3 is resolved for your facility.

Enclosed is a copy of our Safety Evaluation.

Sincerely,

Original signed by

Domenic B. Vassallo, Chief
Operating Reactors Branch #2
Division of Licensing

Enclosure:
As stated

cc w/enclosure:
See next page

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PDR ADOCK 05000293
P PDR

Mr. William D. Harrington
Boston Edison Company

Pilgrim Nuclear Power Station

cc:
Mr. Charles J. Mathis, Station Mgr.
Boston Edison Company
RFD #1, Rocky Hill Road
Plymouth, Massachusetts 02360

Resident Inspector's Office
U. S. Nuclear Regulatory Commission
Post Office Box 867
Plymouth, Massachusetts 02360

Mr. David F. Tarantino
Chairman, Board of Selectman
11 Lincoln Street
Plymouth, Massachusetts 02360

Office of the Commissioner
Massachusetts Department of
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One Winter Street
Boston, Massachusetts 02108

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Regional Administrator, Region I
U. S. Nuclear Regulatory Commission
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Mr. A. Victor Morisi
Boston Edison Company
25 Braintree Hill Park
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Braintree, Massachusetts 02184

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

RELATED TO THE POST-ACCIDENT SAMPLING SYSTEM (NUREG-0737, II.B.3)

BOSTON EDISON COMPANY

PILGRIM NUCLEAR POWER STATION

DOCKET NO. 50-293

1.0 INTRODUCTION

In our Safety Evaluation dated July 1, 1985, we determined that Boston Edison Company (the licensee) met ten of the eleven NUREG-0737, Item II.B.3 criteria. To meet the final criterion relative to estimating core damage during accident conditions, the licensee submitted for our review Revision 1 to Pilgrim Nuclear Power Station Procedure No. 5.7.5 on September 9, 1985.

2.0 EVALUATION

Core damage estimates are based on utilizing post-accident sampling system (PASS) measurements of fission product concentrations in primary coolant and in containment. The revised procedure provides for estimating the extent of metal-water reaction based on measured hydrogen concentration in containment and for estimating the extent of core damage based on containment high range radiation monitors. These provisions meet Criterion (2) of Item II.B.3 and are, therefore, acceptable.

3.0 CONCLUSION

Based on the above evaluation and our previous evaluation dated July 1, 1985, we conclude that the licensee's post-accident sampling system meets all the criteria of Item II.B.3 of NUREG-0737 and is, therefore, acceptable.

Principal Contributor: F. Witt

Dated: September 30, 1985