

APPENDIX B

US NUCLEAR REGULATORY COMMISSION

NRC Inspection Report: 50-482/86-01

LP: NPF-42

Docket: 50-482

Licensee: Kansas Gas and Electric Company (KG&E)
Post Office Box 203
Wichita, Kansas 67201

Facility Name: Wolf Creek Generating Station (WCGS)

Inspection At: Wolf Creek Site, Coffey County, Burlington, Kansas

Inspection Conducted: January 5 to 31, 1986

Inspectors: for D M Hunnicutt
J. E. Cummins, Senior Resident Inspector,
Operations

2/12/86
Date

for D M Hunnicutt
B. L. Bartlett, Resident Reactor Inspector,
Operations

2/12/86
Date

Approved: D M Hunnicutt
D. M. Hunnicutt, Acting Chief, Project
Section B, Reactor Projects Branch

2/12/86
Date

Inspection Summary

Inspection Conducted January 5 to 31, 1986 (Report 50-482/86-01)

Areas Inspected: Routine, unannounced inspection including plant status; followup on previously identified NRC items; operational safety verification; engineered safety features system walkdown; onsite followup of events; licensee event report (LER) review; monthly maintenance observation; monthly surveillance observation; and plant tours. The inspection involved 148 inspector-hours onsite by two NRC inspectors including 12 inspector-hours onsite during offshifts.

Results: Within the nine areas inspected, one violation was identified (failure to comply with licensee's temporary modification procedure, paragraph 10).

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DETAILS

1. Persons Contacted

Principal Licensee Personnel

G. L. Koester, Vice President-Nuclear
*J. A. Bailey, Interim Site-Director
*F. T. Rhodes, Plant Manager
R. M. Grant, Director-Quality
*G. D. Boyer, Superintendent of Technical Support
*J. A. Zell, Superintendent of Operations
*M. D. Rich, Superintendent of Maintenance
*M. G. Williams, Supt. of Regulatory, Quality, and Administrative Services
H. K. Chernoff, Licensing
K. Peterson, Licensing
G. Pendergrass, Licensing
*W. M. Lindsay, Supervisor Quality Systems
*C. J. Hoch, QA Technologist
*A. A. Freitag, Manager of Nuclear Plant Engineering, WCGS
*C. M. Herbst, Project Engineer, Bechtel

The NRC inspectors also contacted other members of the licensee's staff during the inspection period to discuss identified issues.

*Denotes those personnel in attendance at the exit meeting held on January 31, 1986.

2. Plant Status

The plant operated in Mode 1 during this inspection period.

3. Followup on Previously Identified NRC Items

(Closed) Open Items (50-482/8508-07 and 8508-09): Issuance of Surveillance Procedures.

These open items tracked the issuance of surveillance procedures that had not been issued at the time of the inspection. The NRC inspector verified the following surveillance procedures had been issued:

- STS IC-214, Rev. 1, Trip Actuating Device Operational Test Phase A Containment Isolation Manual Initiation
- STS IC-225, Surveillances in this STS were incorporated into STS GT-003, Rev. 0, CPIS Manual Test, therefore STS IC-225 was cancelled.

- STS IC-229A, Rev. 0- Actuation Logic Test Load Shedder and Emergency Load Sequencer Group I (Red).
- STS IC-229B, Rev. 0- Actuation Logic Test Load Shedder and Emergency Load Sequencer Group IV (Yellow).
- STS MT-054, Rev. 1 - Feedwater Venturi Inspection.

These items are closed.

(Closed) Open Item (50-482/8459-09): System Restoration Instructions in Preoperational Test Procedures.

The following startup test procedures were reviewed to verify that they contained comprehensive instructions for system restoration upon completion of testing:

- SU7-0009.1, Rev. 1, Thirty Percent Power Load Swing Test Section 8.0 System Restoration.
- SU7-SF06.3, Rev. 1, Operational Alignment of Process Temperature Instrumentation Section 8.0 System Restoration.
- SU7-SF09.1, Rev. 1, RCCA or Banks Worth Measurement at Power (30% Power Pseudo Rod Ejection).
- SU7-S011, Rev. 1, Initial Criticality and Low Power Test Sequence.
- SU7-SR03, Rev. 1, Incore Movable Detector Test
- SU7-S010, Rev. 2, Post Core Loading Precritical Test Sequence.

This item is closed.

(Closed) Open Item (50-482/8504-01): Incorporation of Certain Evacuation Activities into Procedures.

This open item tracked the incorporation in appropriate procedures of instructions for the evacuation of nonessential personnel from the plant evacuation initial assembly point for those personnel who do not have transportation immediately available to them. Instructions to cover this situation have been incorporated in Wolf Creek Radiological Emergency Response Plan Implementing Procedure EPP 01-2.2, "Activation of Emergency Plan/Organization. This procedure, in Attachment 2.0, "Initial Announcement," and Attachment 3.0, "Reclassification Announcement," states: "If you do not have a vehicle try to ride with someone else who has a vehicle or notify security."

This item is closed.

(Closed) Open Item (50-482/8459-08): Discrepancy in Startup Procedure.

This open item was initiated to track a discrepancy in Power Ascension Test Procedure SU7-SF09.2, Revision 0, "RCCA or Bank Worth Measure at Power (50% Pseudo Rod Drop)."

4. Operational Safety Verification

The NRC inspectors verified that the facility is being operated safely and in conformance with regulatory requirements by direct observation of licensee facilities, tours of the facility, interviews and discussions with licensee personnel, independent verification of safety system status and limiting conditions for operations, and reviewing facility records. The NRC inspectors, by observation, interview of personnel, and review of documents, verified the physical security plan was being implemented in accordance with the security plan and that radiation protection activities were controlled.

By observing valve position, electrical breaker position, and control room indication, the NRC inspectors confirmed the operability of the safety injection system, residual heat removal system, and the containment spray system. The NRC inspectors also visually inspected safety components for leakage, physical damage, and any other impairment that could prevent them from performing their designed function.

No violations or deviations were identified.

5. Engineered Safety Features (ESF) System Walkdown

The NRC inspectors verified the operability of ESF systems by walking down selected accessible portions of the systems. The NRC inspectors verified valves and electrical circuit breakers were in the required position, power was available, and valves were locked where required. The NRC inspectors also inspected system components for damage or other conditions that could degrade system performance.

The ESF systems listed below were walked down during this inspection report period:

- Class 1E 4160 Volt AC Electrical Systems
- Auxiliary Feedwater System
- Class 1E 120 Volt DC Electrical Systems

No violations or deviations were identified.

6. Onsite Followup of Events

The NRC inspectors performed onsite followup of nonemergency events that occurred during this report period. The NRC inspectors (when available) observed control room personnel response, observed instrumentation indicators of reactor plant parameters, reviewed logs and computer printouts, and discussed the event with cognizant personnel. The NRC inspectors verified the licensee had responded to the event in accordance with procedures and had notified the NRC and other agencies as required in a timely fashion.

ESF actuations that occurred during the report period are listed in the table below. The NRC inspector will review the license event report (LER) for each of these events and will report any findings in subsequent NRC inspection reports.

<u>Date</u>	<u>Event</u>	<u>Plant Status</u>	<u>Cause</u>
1/04/86	CRVIS*	Mode 1	Broken paper sensing tape in chlorine monitor
1/04/86	CRVIS*	Mode 1	Blown fuse on chlorine monitor

*CRVIS - control room ventilation isolation signal

No violations or deviations were identified.

7. Review of Licensee Event Reports (LERs)

During this inspection period, the NRC inspectors performed followup on selected Wolf Creek LERs. The LERs were reviewed to ensure:

- The corrective action stated in the report has been properly completed.
- Responses to the events were adequate.
- Responses to the events met license conditions, commitments, or other applicable regulatory requirements.
- The information contained in the report satisfied applicable reporting requirements.
- That any generic issues were identified.
- The report conformed to the guidelines contained in NUREG-1022 and Supplements 1 and 2.

The following LERs were reviewed and closed.

- 50-482/85-024-00, "ESF Actuation-Feedwater Isolation, Auxiliary Feedwater Actuation"
- 50-482/85-025-00, "Condition Prohibited by Technical Specifications"
- 50-482/85-026-00, "ESF Actuation-Containment Purge, Fuel Building, and Control Room Ventilation Isolation"
- 50-482/85-027-00, "ESF Actuation-Safety Injection and Main Steamline Isolation"
- 50-482/85-028-00, "ESF Actuation-Auxiliary Feedwater Actuation"
- 50-482/85-029-00, "ESF Actuation-Containment Purge Isolation, Fuel Building and Control Room Ventilation Isolation"
- 50-482/85-030-00, "ESF Actuation-Feedwater Isolation"
- 50-482/85-031-00, "ESF Actuation-Feedwater Isolation, Auxiliary Feedwater Actuation, Steam Generator Blowdown and Sample Isolation"

- 50-482/85-032-00, "ESF Actuation-Containment Purge and Control Room Ventilation Actuation"
- 50-482/85-033-00, "ESF Actuation-Control Room Ventilation Isolation"
- 50-482/85-034-00, "Technical Specification Violation"
- 50-482/85-036-00, "ESF Actuation-Feedwater Isolation, Auxiliary Feedwater Actuation, Steam Generator Blowdown and Sample Isolation"
- 50-482/85-037-00, "ESF Actuation-Control Room Ventilation Isolation"
- 50-482/85-038-00, "Auxiliary Feedwater Actuation"
- 50-482/85-039-00, "ESF Actuation and Reactor Trip on Low Steam Generator Level"
- 50-482/85-040-00, "ESF Actuation-Control Room Ventilation Isolation"

No violations or deviations were identified.

8. Monthly Maintenance Observation

The NRC inspector observed maintenance activities performed on safety-related systems and components to verify that these activities were conducted in accordance with approved procedures, Technical Specifications, and applicable industry codes and standards. The following elements were considered by the NRC inspector during the observation and/or review of the maintenance activities:

- Limiting conditions for operation (LCO) were met and, where applicable, redundant components were operable.
- Activities complied with adequate administrative controls.
- Where required, adequate, approved, and up-to-date procedures were used.
- Craftsmen were qualified to accomplish the designated task and technical expertise (i.e., engineering, health physics, operations) were made available when appropriate.
- Replacement parts and materials being used were properly certified.
- Required radiological controls were implemented.
- Fire prevention controls were implemented where appropriate.
- Required alignments and surveillances to verify post maintenance operability were performed.
- Quality control hold points and/or checklists were used when appropriate and quality control personnel observed designated work activities.

Selected portions of the maintenance activities accomplished on the work requests (WR) listed below were observed and related documentation reviewed by the NRC inspector:

- WR 00310-86, Containment H₂ Analyzer, GS 065B-H₂ Analyzer span is extremely low.
- WR 50146-86, Three month inspection and lubrication of the personnel air lock during operation.

No violations or deviations were identified.

9. Monthly Surveillance Observation

The NRC inspectors observed selected portions of the performance of surveillance testing and/or reviewed completed surveillance test procedures to verify that surveillance activities were performed in accordance with Technical Specifications (TS) requirements and administrative procedures. The NRC inspectors considered the following items while inspecting surveillance activities:

- Testing was being accomplished by qualified personnel in accordance with an approved procedure.
- The surveillance procedure conforms to TS requirements.
- Required test instrumentation was calibrated.
- TS limiting conditions for operation were satisfied.
- Test data was accurate and complete. The NRC inspectors performed independent calculations of selected test data to verify their accuracy.
- The performance of the surveillance procedure conformed to applicable administrative procedures.
- The surveillance was performed within the required frequency and the test results met the required limits.

Surveillances witnessed and/or reviewed by the NRC inspectors are listed below:

- STS NB-005, Rev. 3, "Breaker Alignment Verification"
- STS KJ-005A, Rev. 5, "Manual/Auto Start, Synchronization, and Loading of Emergency Diesel Generator NE01"
- STS IC-915, Rev. 2, "Analog Channel Operational Test Component Cooling Water Non-nuclear Safety-Related"
- STS KJ-005B, Rev. 4, "Manual/Auto Start, Synchronization, and Loading Of Emergency Diesel Generator NE02"
- STS IC-913, Rev. 3, "Containment Hydrogen Analyzer GS 065B Calibration Test"
- STS PE-013, Rev. 4, "Personnel Airlock Seal Test"

No violations or deviations were identified.

10. Plant Tours

At various times during the course of the inspection period the NRC inspectors conducted general tours of the reactor building, auxiliary building, radwaste building, fuel handling building, control building, turbine building, essential service water building, and the secured area surrounding the buildings. During the tours, the NRC inspectors observed housekeeping practices, fire protection barriers and equipment, and maintenance on equipment. The NRC inspectors also discussed various subjects with licensee personnel.

Selected NRC inspector observations are discussed below:

On January 23, 1986, on the 2026' level (northeast corner) of the auxiliary building, the NRC inspector observed that a temporary modification had been made in that safety-related flexible Electrical Conduit 1J1E1D was tied off with wire to Essential Service Water Pipe Support 1-EF02-R009/134Q. The licensee has established and implemented Administrative Procedure (ADM) 02-101, Revision 13, "Temporary Modification," to control temporary modifications; however, the actions required by Procedure ADM 02-101 had not been performed to install the temporary modification described above.

This failure to control this temporary modification in accordance with an established procedure is an apparent violation. (50-482/8601-01) When this condition was identified to licensee personnel, they took immediate action and removed the wire attaching the flexible conduit to the pipe support, returning the installation to design configurations.

10. Exit Meeting

The NRC inspectors met with licensee personnel to discuss the scope and findings of this inspection on January 31, 1986. The NRC inspectors also attended entrance/exit meetings of other NRC region based inspectors identified below:

<u>Inspection Period</u>	<u>Lead Inspector</u>	<u>Area Inspected</u>	<u>Inspection Report No.</u>
1-27/31-86	J. Kelly	Security	86-03