

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1) Haddam Neck										DOCKET NUMBER (2) 0 5 0 0 0 2 1 1 3										PAGE (3) 1 OF 0 3	
TITLE (4) Steam Generator Eddy Current Testing																					
EVENT DATE (5)			LER NUMBER (6)				REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)											
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES			DOCKET NUMBER(S)									
0 1	2 7	8 6	8 6	0 0 3	0 0	0 2	0 5	8 6	N/A			0 5 0 0 0									
OPERATING MODE (9) 5			THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR § (Check one or more of the following) (11):									FACILITY NAMES			DOCKET NUMBER(S)						
POWER LEVEL (10) 0 0 0			20.402(b)			20.405(c)			50.73(a)(2)(iv)			73.71(b)									
			20.405(a)(1)(i)			50.36(c)(1)			50.73(a)(2)(v)			73.71(c)									
			20.405(a)(1)(ii)			50.36(c)(2)			50.73(a)(2)(vi)			OTHER (Specify in Abstract below and in Text, NRC Form 366A)									
			20.405(a)(1)(iii)			50.73(a)(2)(i)			50.73(a)(2)(vii)(A)												
			20.405(a)(1)(iv)			50.73(a)(2)(ii)			50.73(a)(2)(viii)(B)												
			20.405(a)(1)(v)			50.73(a)(2)(iii)			50.73(a)(2)(ix)												
LICENSEE CONTACT FOR THIS LER (12)												TELEPHONE NUMBER									
NAME M. J. Ranieri, Associate Engineer												AREA CODE 2 1 0 3									
												2 1 0 3 2 1 6 1 7 1 - 1 2 1 5 1 6									
COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)																					
CAUSE	SYSTEM	COMPONENT	MANUFAC TURE	REPORTABLE TO NPDOS		CAUSE	SYSTEM	COMPONENT	MANUFAC TURE	REPORTABLE TO NPDOS											
					NA						NA										
					NA						NA										
SUPPLEMENTAL REPORT EXPECTED (14)										EXPECTED SUBMISSION DATE (15)		MONTH	DAY	YEAR							
<input type="checkbox"/> YES (If yes, complete EXPECTED SUBMISSION DATE)										<input checked="" type="checkbox"/> NO											

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single space typewritten lines) (16)

During eddy current testing of steam generator #2, analyzed results placed the steam generator in the C-3 category according to Technical Specification 4.10.1, "Inservice Inspection of Steam Generator Tubes". According to Technical Specification 6.9, "Reporting Requirements" a steam generator in the C-3 category requires a prompt 24 hour notification and a two week written follow-up (LER).

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LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

APPROVED CMB NO. 3150-0104

EXPIRES: 8/31/88

FACILITY NAME (1) Haddam Neck	DOCKET NUMBER (2) 0 5 0 0 0 2 1 3	LER NUMBER (6)			PAGE (3)		
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER			
		8 6	— 0 0 3	— 0 0	0 2	OF	0 3

TEXT (If more space is required, use additional NRC Form 365A's) (17)

Background

On January 19, 1986, eddy current testing (ECT) began in (EIIS Code-SG) steam generator (S.G.) #2. A total of 3610 tubes (EIIS Code-TBG) were to be inspected. According to Technical Specification 4.10.1 a steam generator falls into category C-3 when 10% of the total number of tubes to be inspected in a steam generator are degraded or when 1% of the total number of tubes to be inspected are defective. For S.G.#2, 361 tubes degraded or 36 tubes defective would place it in category C-3. On January 27, 1986, 367 tubes were analyzed as degraded (19 defective) placing S.G.#2 into category C-3. Category C-3 requires all tubes in the affected steam generator, and a larger sample in the other steam generators, to be inspected. The Licensees inspection plan for this refueling outage requires all tubes in all steam generators to be inspected. According to Technical Specification 6.9, a prompt report was made to the NRC. This LER is the two week follow-up report also required by Technical Specification 6.9.

Reportability

This event is reportable under 10CFR50.73(a)(2)(i) since it involved a requirement of a plant Technical Specification.

Root Cause

The steam generator tubes are experiencing corrosive degradation forming pits mainly at or above the tubesheet on the cold leg side of the steam generator.

Evaluation

ECT is performed each refueling outage to monitor steam generator tubes for degradation and percent through wall penetration. Type of degradation is monitored with the hope of optimizing plant parameters to slow or stop the tubes from degrading. Percent through wall penetrations is monitored to allow repairs to be made before a tube degrades to the point where rupture is possible.

LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

APPROVED OMB NO. 3150-0104

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TEXT (If more space is required, use additional NRC Form 385A's) (17)

Corrective Action1). Short Term Action

All tubes with degradation greater than or equal to 50% through wall are required to be repaired (sleeved or plugged).

2). Long Term Action

Each refueling outage steam generator tubes will have ECT performed to monitor and update their histories. Other special tests may be performed, such as profilometry and/or intergranular attack testing to help in identifying other types of degradation. ECT data may then possibly be used to change S.G. chemistry and/or other plant parameters to help reduce or stop tube degradation.

It should be noted that the licensee will continue to perform sludge lance cleaning of the secondary sides of the steam generators. The licensee will continue to eliminate copper bearing alloys from major secondary side components such as condenser tubes, feedwater heater tubes and moisture separator reheater tubes. Finally, the licensee shall continue to rigidly follow the Steam Generator Owners Group chemical impurity guidelines.



CONNECTICUT YANKEE ATOMIC POWER COMPANY

HADDAM NECK PLANT

RR#1 • BOX 127E • EAST HAMPTON, CONN. 06424

February 5, 1986

U.S. Nuclear Regulatory Commission
Document Control Desk
Washington, D. C. 20555

Reference: Facility Operating License No. DPR-61
Docket No. 50-213
Reportable Occurrence LER 50-213/86-003-00

Gentlemen:

This letter forwards the Licensee Event Report 86-003-00, required to be submitted within two weeks, pursuant to the requirements of Connecticut Yankee Technical Specifications.

Very truly yours,

Richard H. Graves
Station Superintendent

RHG:MJR/lac
Attachment: LER 86-003-00

cc: Dr. T. E. Murley, Region I

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