

OPERATING DATA REPORT

DOCKET NO. 50-409
 DATE 12/06/85
 COMPLETED BY Loree Malin
 TELEPHONE 608-689-2331

OPERATING STATUS

1. Unit Name: La Crosse Boiling Water Reactor
2. Reporting Period: 0000, 11/01/85 to 2400, 11/30/85
3. Licensed Thermal Power (MW_t): 165
4. Nameplate Rating (Gross MW_e): 65.3
5. Design Electrical Rating (Net MW_e): 50
6. Maximum Dependable Capacity (Gross MW_e): 50
7. Maximum Dependable Capacity (Net MW_e): 48
8. If Changes Occur in Capacity Ratings (Items 3 Through 7) Since Last Report, Give Reasons: _____

NOTES:

9. Power Level To Which Restricted, If Any (Net MW_e): _____
10. Reasons for Restrictions, If Any: _____

	<u>This Month</u>	<u>Yr.-to-Date</u>	<u>Cumulative</u>
11. Hours In Reporting Period	720	8,016	140,979
12. Number Of Hours Reactor Was Critical	720	7,013.2	95,336.9
13. Reactor Reserve Shutdown Hours	0	0	478
14. Hours Generator On-Line	720	6,853.6	88,756.9
15. Unit Reserve Shutdown Hours	0	0	79
16. Gross Thermal Energy Generated (MWH)	110,529.1	996,039.1	12,343,441.0
17. Gross Electrical Energy Generated (MWH)	34,738	307,327	3,702,938
18. Net Electrical Energy Generated (MWH)	32,840	288,559	3,434,395
19. Unit Service Factor	100	85.5	63.0
20. Unit Availability Factor	100	85.5	63.0
21. Unit Capacity Factor (Using MDC Net)	95.0	75.0	50.8
22. Unit Capacity Factor (Using DER Net)	91.2	72.0	48.7
23. Unit Forced Outage Rate	0	3.4	9.8

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):

Refueling, March 2, 1986, 5-6 weeks

25. If Shut Down At End Of Report Period, Estimated Date of Startup: NA
 26. Units In Test Status (Prior to Commercial Operation):
- | | <u>Forecast</u> | <u>Achieved</u> |
|----------------------|-----------------|-----------------|
| INITIAL CRITICALITY | _____ | _____ |
| INITIAL ELECTRICITY | _____ | _____ |
| COMMERCIAL OPERATION | _____ | _____ |

AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO. 50-409

UNIT LACBWR

DATE 12/06/85

COMPLETED BY L.S. Goodman

TELEPHONE 608-689-2331

MONTH November 1985

DAY AVERAGE DAILY POWER LEVEL
(MWe-Net)

1	<u>48</u>
2	<u>48</u>
3	<u>48</u>
4	<u>48</u>
5	<u>48</u>
6	<u>49</u>
7	<u>49</u>
8	<u>49</u>
9	<u>49</u>
10	<u>48</u>
11	<u>48</u>
12	<u>49</u>
13	<u>49</u>
14	<u>48</u>
15	<u>39</u>
16	<u>27</u>

DAY AVERAGE DAILY POWER LEVEL
(MWe-Net)

17	<u>36</u>
18	<u>25</u>
19	<u>35</u>
20	<u>45</u>
21	<u>48</u>
22	<u>48</u>
23	<u>49</u>
24	<u>49</u>
25	<u>49</u>
26	<u>49</u>
27	<u>48</u>
28	<u>49</u>
29	<u>48</u>
30	<u>48</u>
31	<u>NA</u>

INSTRUCTIONS

On this format, list the average daily unit power level in MWe-Net for each day in the reporting month. Compute to the nearest whole megawatt.

UNIT SHUTDOWNS AND POWER REDUCTIONS

DOCKET NO. 50-409
 UNIT NAME LACBWR
 DATE 12/06/85
 COMPLETED BY L. S. Goodman
 TELEPHONE 608-689-2331

REPORT MONTH November 1985

No.	Date	Type ¹	Duration (Hours)	Reason ²	Method of Shutting Down Reactor ³	Licensee Event Report #	System Code ⁴	Component Code ⁵	Cause & Corrective Action to Prevent Recurrence
85-13	11-15-85	F	0	A	4	NA	CB	INSTRU	Power decreased when the 1A Forced Circulation Pump Dis- charge Valve closed.
85-14	11-18-85	F	0	A	4	NA	CB	INSTRU	Power decreased when the 1A Forced Circulation Pump Dis- charge Valve closed.

1

F: Forced
 S: Scheduled

2

Reason:
 A-Equipment Failure (Explain)
 B-Maintenance or Test
 C-Refueling
 D-Regulatory Restriction
 E-Operator Training & License Examination
 F-Administrative
 G-Operational Error (Explain)
 H-Other (Explain)

3

Method:
 1-Manual
 2-Manual Scram
 3-Automatic Scram
 4-Other (Explain)

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Exhibit G-Instructionse
 for Preparation of Data
 Entry Sheets for Licensee
 Event Report (LER) File
 (NUREG-0161)

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Exhibit 1 - Same Source

REFUELING INFORMATION REQUEST

1. Name of Facility

La Crosse Boiling Water Reactor (LACBWR)

2. Scheduled Date for Next Refueling Shutdown

The tentative date for the next refueling shutdown (EOC-IX) is March 2, 1986.

3. Scheduled Date for Restart Following Refueling

The tentatively scheduled date for subsequent reactor startup is approximately April 7, 1986

4. Will Refueling or Resumption of Operation Thereafter Require a Technical Specification Change or Other License Amendment?

If answer is yes, what, in general, will these be?

If answer is no, has the reload fuel design and core configuration been reviewed by your Plant Safety Review Committee to determine whether any unreviewed safety questions are associated with the core reload Reference 10 CFR Section 50.59?

If no such review has taken place, when is it scheduled?

A license amendment will be needed for the new control rods which will be installed.

5. Scheduled Date(s) for Submitting Proposed Licensing Action and Supporting Information.

December 1985

6. Important Licensing Considerations Associated with Refueling, e.g., New or Different Fuel Design or Supplies, Unreviewed Design or Performance Analysis Methods, Significant Changes in Fuel Design, New Operating Procedure

Some new control rods will be utilized.

REFUELING INFORMATION REQUEST - (Continued)

7. The Number of Fuel Assemblies

(a) in the Core and

(b) in the Spent Fuel Pool.

Core Loading: 72 Fuel Assemblies

Spent Fuel Storage Pool Loading: 237 Irradiated Fuel Assemblies

8. The Present Licensed Spent Fuel Pool Storage Capacity and the Size of any Increase in Licensed Storage Capacity that has been Requested or is Planned, in Number of Fuel Assemblies

440 Fuel Assemblies

9. The Projected Date of the Last Refueling that can be Discharged to the Spent Fuel Pool Assuming the Present Licensed Capacity

1992

December 6, 1985

In reply, please
refer to LAC-11307

DOCKET NO. 50-409

Office of Management Information
and Program Control
U. S. Nuclear Regulatory Commission
Washington, D. C. 20555

SUBJECT: DAIRYLAND POWER COOPERATIVE
LA CROSSE BOILING WATER REACTOR (LACBWR)
PROVISIONAL OPERATING LICENSE NO. DPR-45
MONTHLY OPERATING DATA REPORT FOR NOVEMBER 1985

Reference: (1) NRC Letter, Reid to Madgett,
dated September 19, 1977.
(2) NRC Letter, Reid to Madgett,
dated December 29, 1977

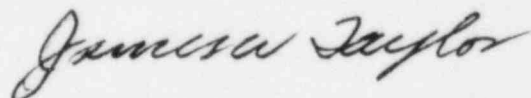
Gentlemen:

In accordance with instructions contained in Reference (1) and Technical Specification Amendments contained in Reference (2), we are submitting information concerning operability and availability of the La Crosse Boiling Water Reactor (LACBWR) for the Month of November 1985.

Please contact us if there are any questions concerning this report.

Very truly yours,

DAIRYLAND POWER COOPERATIVE



James W. Taylor, General Manager

JWT:LSG:sks

cc: J. G. Keppler, Regional Administrator, NRC-DRO III
John Stang, LACBWR Project Manager
NRC Resident Inspector
D. Sherman (ANI Library)
INPO

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NARRATIVE SUMMARY OF OPERATING EXPERIENCE

NOVEMBER 1985

At the onset of the November 1985 reporting period, power generation continued at approximately 98% Rated Thermal Power (48 MWe-net).

At 1603 on November 15, 1985, the 1A Forced Circulation Pump Discharge Valve closed. Reactor power was decreased to approximately 33% power and the valve was reopened at 1928. Power escalation then began and continued until November 18, reaching approximately 86% power. At 0156 on November 18, the 1A Forced Circulation Pump Discharge Valve closed. Reactor power was decreased to approximately 34%. At 0400, the valve was opened and power re-escalation commenced.

Reactor power reached 98% (48-49 MWe-net) on November 20. Power generation continued at this level through the remainder of the November reporting period.

During the month of November, maintenance was performed on the 1A Shutdown Condenser Steam Inlet Valve Positioner Gauge Manifold. The 1A Shutdown Condenser steam inlet manual valve was isolated while the manifold gasket was replaced. A surveillance test was then performed on the 1A Shutdown Condenser Steam Inlet (Automatic) Valve. No other major safety-related maintenance was performed during the November reporting period.

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