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ComEd

April 9, 1997

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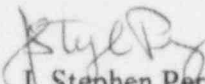
Director, Nuclear Reactor Regulation
U. S. Nuclear Regulatory Commission
ATTN: Document Control Desk
Washington, DC 20555

Subject: 1997 First Quarter 10 CFR 50.59 Report
Dresden Nuclear Power Station
Docket Nos. 50-010, 50-237, and 50-249

Enclosed is the first quarter Report of completed Changes, Tests, and Experiments per 10 CFR 50.59 for Dresden Nuclear Power Station. These evaluations correspond to the conditions identified in 10 CFR 50.59(a)(2) for determining whether a proposed change, test, or experiment shall be determined to involve an unreviewed safety question.

If there are any questions regarding this report, please contact Mr. Frank Spangenberg, Dresden Station Regulatory Assurance Manager, at (815) 942-2920, ext. 3800.

Sincerely,


J. Stephen Perry
Site Vice President
Dresden Station

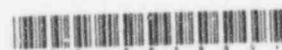
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Enclosure

cc: A. Bill Beach, Regional Administrator, Region III
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Office of Nuclear Facility Safety - IDNS

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Safety Evaluation Summary Report

Safety Evaluation Number: 1997-01-003

Type of Safety Evaluation: Exempt Change

Evaluation Reference Number: E12-3-95-259

Title: Unit 3 Corner Room Structural Steel

Description: This exempt change provides reinforcement details for the Unit 3 Corner Room structural steel. The LPCI corner room structural steel framing was originally designed in 1966 using the weight of the heat exchanger with area loadings to account for piping loads. During an evaluation of pipe support load changes in 1991, it was determined that the original analysis had never been updated to include as-built piping loads. Walkdowns of all four LPCI corner rooms were conducted in 1993 to identify all the attached loads on the structural steel. A preliminary assessment was made in January 1994 indicating stress levels in the main support steel and connections may exceed UFSAR limits unless a detailed analysis with refined models was performed. In addition, it was determined that piping nozzle loads on the LPCI Heat Exchanger were not accounted for in the structural design calculations. The effect of these nozzle loads on the structural steel is significant. A ENC-QE-40.1 evaluation was performed (CHRON No. 0123866, dated 1/6/94) which found the structural steel to be operable. At the request of the NRC, a more detailed operability evaluation was made and documented in supplements to the original ENC-QE-40.1 evaluation (see Document No. 4912523, dated 4/2/96, and Document No. 4921117, dated 4/4/96). The supplementary information confirmed the conclusion of the original QE-ENC-40.1 evaluation.

The expectation of the NRC, as outlined in Generic Letter 91-18, is that design stresses found to be outside design allowables be returned to within UFSAR allowables. In March 1994, ComEd met with the Region III NRC Inspector and presented the operability determination and the plan to restore margin to the LPCI corner room support steel. Comments were addressed and resolved with the plan showing a completion date of December 31, 1994.

In May 1994, based in part on the LPCI corner room support steel being operable, work was postponed to focus Station resources on addressing emergent Core Shroud issues. At that time the corner room steel plan was revised to show a completion date of December 1996. In September 1995, this issue was presented to Dresden's Issues Review Board and the technical approach was approved. In October 1995, the Dresden Business Review Committee authorized funding Engineering for work to proceed. The formal evaluation and design for the Unit 3 Corner Rooms was started in April 1996.

The reinforcement details issued under this exempt change increase the capacity of the beams and connections to carry design basis loading and allow them to meet UFSAR requirements. The main purpose of the reinforcement details is to resist seismic loads from the LPCI Heat Exchanger and associated piping nozzle loads. The Unit 3 Corner Room structural steel will meet UFSAR requirements when the installation of the reinforcement is completed.

Result: This evaluation determined that an unreviewed safety question did not exist.

Safety Evaluation Summary Report

Safety Evaluation Number: 1997-01-004

Type of Safety Evaluation:

Exempt Change

Evaluation Reference Number: E12-3-95-258

Title: Unit 3 Corner Room Structural Steel

Description: This exempt change provides reinforcement details for the Unit 3 Corner Room structural steel. The LPCI corner room structural steel framing was originally designed in 1966 using the weight of the heat exchanger with area loadings to account for piping loads. During an evaluation of pipe support load changes in 1991, it was determined that the original analysis had never been updated to include as-built piping loads. Walkdowns of all four LPCI corner rooms were conducted in 1993 to identify all the attached loads on the structural steel. A preliminary assessment was made in January 1994 indicating stress levels in the main support steel and connections may exceed UFSAR limits unless a detailed analysis with refined models was performed. In addition, it was determined that piping nozzle loads on the LPCI Heat Exchanger were not accounted for in the structural design calculations. The effect of these nozzle loads on the structural steel is significant. A ENC-QE-40.1 evaluation was performed (CHRON No. 0123866, dated 1/6/94) which found the structural steel to be operable. At the request of the NRC, a more detailed operability evaluation was made and documented in supplements to the original ENC-QE-40.1 evaluation (see Document No. 4912523, dated 4/2/96, and Document No. 4921117, dated 4/4/96). The supplementary information confirmed the conclusion of the original QE-ENC-40.1 evaluation.

The expectation of the NRC, as outlined in Generic Letter 91-18, is that design stresses found to be outside design allowables be returned to within UFSAR allowables. In March 1994, ComEd met with the Region III NRC Inspector and presented the operability determination and the plan to restore margin to the LPCI corner room support steel. Comments were addressed and resolved with the plan showing a completion date of December 31, 1994.

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The reinforcement details issued under this exempt change increase the capacity of the beams and connections to carry design basis loading and allow them to meet UFSAR requirements. The main purpose of the reinforcement details is to resist seismic loads from the LPCI Heat Exchanger and associated piping nozzle loads. The Unit 3 Corner Room structural steel will meet UFSAR requirements when the installation of the reinforcement is completed.

Result: This evaluation determined that an unreviewed safety question did not exist.

Safety Evaluation Number: 1997-01-005

Type of Safety Evaluation:

FSAR Change

Evaluation Reference Number: UFSAR Change 97-006

Title: Remove Turbine Rating from UFSAR

Description: Remove turbine rating of 832,000 KW from UFSAR Section 10.2. The KW output is not a design limit, but rather a function of turbine-generator efficiency.

Result: This evaluation determined that an unreviewed safety question did not exist.

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Safety Evaluation Summary Report

Quarter: 1997-01

Safety Evaluation Number: 1997-01-006

Type of Safety Evaluation: FSAR Change

Evaluation Reference Number: DFL 97-018

Title: Information on Station Batteries

Description: Revise UFSAR Section 8.3.2.2 to add information addressing the qualified life of the 125Vdc normal and alternate station batteries. This was an item identified during the NRC ISI.

Result: This evaluation determined that an unreviewed safety question did not exist.

Safety Evaluation Number: 1997-01-007

Type of Safety Evaluation: FSAR Change

Evaluation Reference Number: DFL 97-012

Title: UFSAR Changes Dealing with the LOCA Analysis

Description: This 50.59 addresses the changes to UFSAR dealing with the LOCA analysis incorporating the newly identified ECCS leakages and a conservative ECCS fluid temperature performed by Siemens Power Corporation using NRC approved methods used for previous LOCA analyses of record. Siemens Power Corporation, in order to support the identification of new ECCS leakages and a new limiting ECCS fluid temperature has performed an analysis to determine the peak cladding temperature, local cladding oxidation and core wide metal water reaction for the limiting break and ECCS single failure. These values are all within the acceptance criteria of 10 CFR 50.46. Siemens Power Corporation uses NRC approved methodology to support their LOCA analyses.

Result: This evaluation determined that an unreviewed safety question did not exist.

Safety Evaluation Number: 1997-01-008

Type of Safety Evaluation: Temporary Alteration

Evaluation Reference Number: III-53-96

Title: Aux Computer Room

Description: The purpose for this Temporary Alteration is to isolate the Aux Computer room from the Control Room ventilation system by closing the inlet damper 2/3-5741-054D and installing a blank off plate at the exhaust. Two portable air handling units will also be located in the Aux computer room to provide cooling to this room as a stand alone system.

Result: This evaluation determined that an unreviewed safety question did not exist.

Safety Evaluation Number: 1997-01-009

Type of Safety Evaluation: Exempt Change

Evaluation Reference Number: E12-3-97-200

Title: Pipe Nipple Downstream of FCV-3-8501-3A

Description: The proposed change will allow installation of 1" schedule 80 stainless steel pipe nipple with a threaded end to an existing matching socket weld pipe coupling downstream of valve FCV-3-8501-3A. This modification is located inside Unit 3 Drywell at penetration X-115 for Air Sample Return Line No. 3-8510-1"-AK at Elevation 517' near mezzanine catwalk. This will facilitate performing Local Leak Rate Testing (LLRT) of the valve in an accident direction as required per Appendix J to 10 CFR 50.

Result: This evaluation determined that an unreviewed safety question did not exist.

Safety Evaluation Summary Report

Safety Evaluation Number: 1997-01-010

Type of Safety Evaluation:

Exempt Change

Evaluation Reference Number: E12-2-96-216

Title: Yarway Impulse Steam Trap

Description: The design basis for the HPCI Standby (Normal) mode is that the system is maintained in a standby condition to permit rapid initiation while minimizing the effects of thermal stress and water hammer within all components which are necessary for HPCI initiation. The HPCI steam supply line drain pot, drain piping and steam trap were designed to provide a drainage path for the HPCI steam supply line condensate and thus mitigating the likelihood of water hammer in the steam line. The existing HPCI steam supply drain pot and the impulse trap 2(3)-2301-1 in the HPCI drain line was designed to store condensate until a certain amount of condensate was accumulated in the trap and then to discharge the stored condensate via the down stream drain piping to the main condenser. The HPCI supply steam drain line is automatically diverted from the main condenser to the suppression pool upon initiation of the HPCI system.

The existing Yarway impulse steam trap 2(3)-2301-1 in the HPCI steam supply drain line was replaced with an Accuflo choke trap to ensure that the HPCI steam line drains operate reliably and the periodic (approximately every 45 minutes) nuisance alarms occurring in the existing system are eliminated.

Result: This evaluation determined that an unreviewed safety question did not exist.

Safety Evaluation Number: 1997-01-011

Type of Safety Evaluation:

Exempt Change

Evaluation Reference Number: E12-3-95-257

Title: Yarway Impulse Steam Trap

Description: The design basis for the HPCI Standby (Normal) mode is that the system is maintained in a standby condition to permit rapid initiation while minimizing the effects of thermal stress and water hammer within all components which are necessary for HPCI initiation. The HPCI steam supply line drain pot, drain piping and steam trap were designed to provide a drainage path for the HPCI steam supply line condensate and thus mitigating the likelihood of water hammer in the steam line. The existing HPCI steam supply drain pot and the impulse trap 2(3)-2301-1 in the HPCI drain line was designed to store condensate until a certain amount of condensate was accumulated in the trap and then to discharge the stored condensate via the down stream drain piping to the main condenser. The HPCI supply steam drain line is automatically diverted from the main condenser to the suppression pool upon initiation of the HPCI system.

The existing Yarway impulse steam trap 2(3)-2301-1 in the HPCI steam supply drain line was replaced with an Accuflo choke trap to ensure that the HPCI steam line drains operate reliably and the periodic (approximately every 45 minutes) nuisance alarms occurring in the existing system are eliminated.

Result: This evaluation determined that an unreviewed safety question did not exist.

Safety Evaluation Number: 1997-01-012

Type of Safety Evaluation:

Technical Specification Change

Evaluation Reference Number: CCSW & DGCW

Title: Tech Spec Bases for CCSW and DGCW

Description: TS Bases for CCSW has a clarifying clause added to identify that only Unit 2 CCSW supplies water to the CR HVAC Refrigerant Condenser Unit (RCU). Also, the last sentence in Bases 3/4.8.A was inadvertently duplicated in 3/4.8.B for the D/G Cooling Water. It is being removed.

Result: This evaluation determined that an unreviewed safety question did not exist.

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Safety Evaluation Summary Report

Quarter: 1997-01

Safety Evaluation Number: 1997-01-013

Type of Safety Evaluation: FSAR Change

Evaluation Reference Number: DFL 96-143

Title: Five Minute Damper Closure Time Linked to Tech Spec SBTGS Removal Efficiency

Description: Rewrite UFSAR Section 15.7.3.4.3.4 using the same information to make it more understandable and add information linking the five minute damper closure time to the Technical Specification SBTGS removal efficiency (penetration) of 90%. The present UFSAR Section 15.7.3.4.3.4 does not state this linkage. The added information explicitly states that the release, for the refueling accident are well within 10 CFR 100 limits at the Exclusion Area Boundary (EAB) and GDC-19 dose rate limits in the control room are not exceeded with SBTGS filter efficiency of 90% and a maximum closure time of 5 minutes for the Reactor Building Isolation Damper.

Remove the second paragraph which references an AEC analysis which is superseded by the S & L analysis which is the basis for the change.

Result: This evaluation determined that an unreviewed safety question did not exist.

Safety Evaluation Number: 1997-01-014

Type of Safety Evaluation: Exempt Change

Evaluation Reference Number: E12-3-96-234 & E12-3-96-235

Title: 3A & 3B Reactor Recirc Pump Motor Lower Bearing Oil Reservoirs

Description: This Excluded Plant Design Change (EPDC) modifies the oil level detection system on the 3A & 3B Reactor Recirc Pump Motor Lower bearing oil reservoirs. This change will add a vent pipe in the sensing lines to the level switches which provide annunciation in the control room. Entrapped air in these lines causes leakage and false high level alarms. The proposed installation of a vent line will provide a vent path for the entrapped air. This design concept has been proven successful on 2B Pump Motor lower oil line in Unit 2 which was installed by Exempt Plant Change, EPC P12-2-94-217.

Result: This evaluation determined that an unreviewed safety question did not exist.

Safety Evaluation Number: 1997-01-015

Type of Safety Evaluation: Temporary Alteration

Evaluation Reference Number: III-36-96

Title: Continued Operation Below <45% RCTP With 1 Turbine CV Closed

Description: This evaluation assures that there are no unreviewed safety questions with operating Unit 2 below the Reactor Protection System Interlock for Main Turbine Control Valve Closure Scram at 45% Power with (1) One Main Turbine Control Valve closed.

Result: This evaluation determined that an unreviewed safety question did not exist.

Safety Evaluation Number: 1997-01-016

Type of Safety Evaluation: Set Point Changes

Evaluation Reference Number: SPC 03-96-96

Title: TSUP Range for LPCI and Core Spray Low Pressure ADS Permissive

Description: TSUP range for the LPCI and Core Spray low pressure ADS permissive is "100 less than or equal to Discharge Pressure less than or equal to 150 psig". This necessitates changing the setpoint of the instruments to 112 +/-2 psig for Core Spray and 106 +/-2 psig for LPCI. This is a new addition to Tech Specs as part of TSUP.

Result: This evaluation determined that an unreviewed safety question did not exist.

10 CFR 50.59
Safety Evaluation Summary Report

Quarter: 1997-01

Safety Evaluation Number: 1997-01-017

Type of Safety Evaluation: Set Point Changes

Evaluation Reference Number: SPC 02-96-95

Title: TSUP Range for LPCI and Core Spray Low Pressure ADS Permissive

Description: TSUP range for the LPCI and Core Spray low pressure ADS permissive is "100 less than or equal to Discharge Pressure less than or equal to 150 psig". This necessitates changing the setpoint of the instruments to 112 +/-2 psig for Core Spray and 106 +/-2 psig for LPCI. This is a new addition to Tech Specs as part of TSUP.

Result: This evaluation determined that an unreviewed safety question did not exist.

Safety Evaluation Number: 1997-01-018

Type of Safety Evaluation: FSAR Change

Evaluation Reference Number: DFL 97-005

Title: DFL 97-005

Description: Revised assorted sections to be consistent with other UFSAR sections and to reflect the previously installed ISCO Diesel M-U Pumps. Clarified the 90,000 gal. level in the CCST to be consistent with Section 9.2.

Result: This evaluation determined that an unreviewed safety question did not exist.

Safety Evaluation Number: 1997-01-019

Type of Safety Evaluation: Modification

Evaluation Reference Number: M12-2-87-025

Title: Drywell Cooling System (Existing Plant Condition)

Description: A new system had been designed under this modification to augment the original Drywell Cooling System by installing a new fan and associated ductwork in the upper elevations of the drywell to provide an additional mixing of hotter air from the upper elevations with cooler air from below. In order to provide power to the new fan, a new electrical penetration was installed in a spare containment penetration sleeve.

Subsequent to the installation of the electrical penetration, a decision was made to cancel the remaining work for this modification. This decision had been made based on repairs of mechanical insulation and repairs made to the original drywell cooling system which allowed the system to function as initially designed.

This Safety Evaluation is being performed to evaluate the existing condition of the incomplete installation.

Result: This evaluation determined that an unreviewed safety question did not exist.

Safety Evaluation Number: 1997-01-020

Type of Safety Evaluation: Miscellaneous

Evaluation Reference Number: DCR 970011

Title: Errors on Electrical Drawings

Description: As a result of closing modifications M12-2(3)-85-032, several errors were identified on electrical drawings. The errors were mainly minor in nature affecting only references and notes on the prints. There are no major changes nor changes to the battery system design.

Result: This evaluation determined that an unreviewed safety question did not exist.

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Safety Evaluation Summary Report

Quarter: 1997-01

Safety Evaluation Number: 1997-01-021

Type of Safety Evaluation: FSAR Change

Evaluation Reference Number: DFL 96-110

Title: UFSAR Clarification Changes

Description: The proposed UFSAR changes are a result of an independent review of various parts of Chapter 13 of the UFSAR. All changes are for clarification only and do not constitute a change to the facility as described in the UFSAR. They do not result in any changes to the way the plant is operated or maintained.

Result: This evaluation determined that an unreviewed safety question did not exist.

Safety Evaluation Number: 1997-01-022

Type of Safety Evaluation: FSAR Change

Evaluation Reference Number: DFL 96-114

Title: DFL 96-114

Description: The proposed changes are a result of a review between the Dresden UFSAR and Operating Procedures. These changes maintain conformance between the design and licensing basis. These changes do not result in changes to the plant. There are no changes in the method of plant operation or how it is maintained.

Result: This evaluation determined that an unreviewed safety question did not exist.

Safety Evaluation Number: 1997-01-023

Type of Safety Evaluation: FSAR Change

Evaluation Reference Number: DFL 97-003

Title: Scram Bypass Setting \leq 45% RCTP

Description: The load reject/turbine trip scram bypass is currently set at \leq 45% of rated turbine 1st stage pressure (\leq 395 psig) which corresponds to \leq 45% of rated steam flow and is in compliance with current Tech Spec Table 3.1.1, UFSAR Sections 15.2.2.2 and 15.2.3.1, and RPS Design Specifications 257HA388. The scram bypass setting will be changed to 45% of rated core thermal power (\leq 300 psig, 1st stage pressure) as required per TSUP Table 3.1.A-1 and recommended per GE SIL no. 423. This change will be reflected in FSAR Sections 15.2.2.2 and 15.2.3.1. Turbine bypass capacity listed in FSAR Section 15.2.2.2 will also be changed to 40% of rated to be consistent with other FSAR sections and plant design. Also, reference to load was removed from annunciator tile 902(3)-5 A12 (stop valves closed) to eliminate confusion.

Result: This evaluation determined that an unreviewed safety question did not exist.

Safety Evaluation Number: 1997-01-024

Type of Safety Evaluation: Modification

Evaluation Reference Number: M12-0-91-019C, Addendum 3

Title: Uses of Station Blackout Diesel Generator

Description: Uses of the Station Blackout (SBO) Diesel Generator for purposes other than an SBO event. This provides the operators with additional flexibility and capacity for energizing the ECCS Busses (23-1, 24-1, 33-1, 34-1) and Auxiliary Busses (23,24,33,34) during a loss of offsite power.

Result: This evaluation determined that an unreviewed safety question did not exist.

Safety Evaluation Summary Report

Safety Evaluation Number: 1997-01-025

Type of Safety Evaluation:

Temporary Alteration

Evaluation Reference Number: III-59-96

Title: RWCU A HX Room ventilation Control

Description: When the dampers, registers, and ductwork in the RWCU A HX Room are covered with plastic to allow asbestos abatement and when the block wall is removed to facilitate HX removal during D3R14 (Modification M12-3-91-018A), the differential pressure between the RWCU A HX Room and the refueling floor will be affected. If the differential pressure is decreased too much, the control system will act to close dampers and possibly result in a trip of the reactor building fans on low flow. Another possibility is that portable ventilation units that will be used to remove air and filter contaminants from the RWCU A and B HX rooms may cause the differential pressure too much. If so, the dampers in the reactor building ventilation system may open too much and possibly result in poor contamination control in other areas of the reactor building.

To prevent excessive damper movements that may occur when the differential pressure in the RWCU A HX Room is affected, this temporary alteration will install a pressure regulator between instrument air and the tubing at the output of differential pressure transmitter dPT 3-5703-15A (located in Local Panel P-24). This pressure regulator should be capable of regulating output pressure between 9.0 and 12.0 psig. The output pressure will be adjusted by a value as directed by the Reactor Building Ventilation System Engineer to ensure that the reactor building dampers remain at a position that will prevent a low flow trip of the fans but also ensure areas with a high potential for contamination remain at a negative pressure for contamination control.

Result: This evaluation determined that an unreviewed safety question did not exist.

Safety Evaluation Number: 1997-01-026

Type of Safety Evaluation:

Temporary Alteration

Evaluation Reference Number: III-58-96

Title: Unit 3 RWCU A HX Room

Description: This temporary alteration seals ventilation pathways and floor drains in the Unit 3 RWCU A HX Room to provide a containment area for asbestos abatement. This safety evaluation assumes ventilation openings, registers, dampers, and ducts are sealed with plastic (or filters) and taped as required by DHP 0130-08.

Result: This evaluation determined that an unreviewed safety question did not exist.

**10 CFR 50.59
Safety Evaluation Summary Report**

Quarter: 1997-01

Safety Evaluation Number: 1997-01-027

Type of Safety Evaluation:

Procedure

Evaluation Reference Number: DCP 2103-02, Rev. 7

Title: Closed Cooling Water

Description: Provide allowable Turbine Building Closed Cooling Water (TBCCW) radioactivity concentrations and designate activity levels for the various concentrations. These levels will be added to procedure DCP 2103-02 and will provide a range of concentrations and station guidance and/or actions required to address/resolve the potential findings and concerns. These actions help to ensure that releases from the plant do not exceed the limits in 10 CFR 20 and 10 CFR 50. In addition, NRC I.E. 80-10 requires a 50.59 evaluation if any nonradioactive system becomes contaminated. The TBCCW system has the potential to become contaminated because it services contaminated systems within the plant and potential cobalt particles from the various valves and components of systems may become activated. Contamination sources other than cobalt also have the potential to be initiated into the TBCCW system, with cesium being the worst case contamination.

The procedure levels will identify the level of station commitment required and the anticipated actions to address the issue. Current contamination levels have been identified and are significantly lower (10^{-8} uCi/ml or not detected although positive confirmation is observed on spectrum) than the level of 3×10^{-6} uCi/ml set forth in 10 CFR 20. The identification of these low level contaminations is in part a result of the measuring equipment and process now being utilized at Dresden station which provides a better detectability of contamination in lower ranges. This evaluation addresses and justifies contamination levels up to 7×10^{-5} uCi/ml in TBCCW, based on a 200 gpm leak from TBCCW into Service Water. This assumes 15,000 gpm Service Water flow which mixes completely with the 200 gpm leak (2 pumps at 100 gpm). The concentration in service effluent would still be within 10 CFR 20 limits. This evaluation satisfies the need to provide a safety evaluation per NRC I.E. 80-10 for levels up to 7×10^{-5} uCi/ml. At the 7×10^{-5} uCi/ml point, the procedure direction is to implement a plan for contamination reduction. Continued operation of the TBCCW system at and above this contamination level will require a new 10 CFR 50.59 evaluation in accordance with NRC I.E. 80-10.

Result: This evaluation determined that an unreviewed safety question did not exist.

Safety Evaluation Number: 1997-01-028

Type of Safety Evaluation:

FSAR Change

Evaluation Reference Number: DFL 97-016

Title: DFL 97-016

Description: The UFSAR changes associated with this 50.59 evaluation are the results of a review between the UFSAR, the Design Basis Documents (DBDs) and the Dresden Tech Spec Update (TSUP) that has been approved by the NRC and issued for use. The TSUP approval was issued under NRC SER Amendment No. 144 for Tech Spec DPR-19 and Amendment No. 138 for Tech Spec DPR-29. Changes to the UFSAR and DBD were not issued at the time of the TSUP approval. The changes to the UFSAR addressed in this package reflect current TSUP parameters which take an exception to the Control Room air filtration unit (AFU) charcoal efficiency testing requirements as set forth in NRC Regulatory Guide 1.52. The current methyl iodide penetration limit is set at less than 0.175%. The exception which was approved by the NRC sets the new penetration limit at less than 0.50%. This change is strictly for laboratory testing requirements and does not affect the operation of the Control Room AFU or any associated components or systems. This does not result in a change to the way the plant is operated or maintained. There are no modifications, alterations or other facility changes associated with this review.

Result: This evaluation determined that an unreviewed safety question did not exist.

10 CFR 50.59
Safety Evaluation Summary Report

Quarter: 1997-01

Safety Evaluation Number: 1997-01-029

Type of Safety Evaluation:

Temporary Alteration

Evaluation Reference Number: III-31-95

Title: Install Clamp around 3A FW Reg Vlv Body-to-Bonnet Flange Joint

Description: The proposed change is to install a clamp around the 3A feedwater regulating valve body-to-bonnet flange joint, and inject a sealant material into the chamber between the body and bonnet flange. The quantity of sealant material will be limited to ensure that, based on the volume of the injection chamber and compression ratio needed, the potential quantity of sealant that may be injected into the feedwater system is minimized. The process will include at least two injections, but will be governed by the limit on the volume of sealant that can be injected. The reason for this change is to stop a body-to-bonnet leak.

Result: This evaluation determined that an unreviewed safety question did not exist.

Safety Evaluation Number: 1997-01-030

Type of Safety Evaluation:

FSAR Change

Evaluation Reference Number: DFL 97-007

Title: DFL 97-007

Description: The change to Section 12.3.2.4 regarding concrete aging testing is a result of an independent review of the UFSAR. An operability evaluation was performed (OSR #96-298, Offsite Review 12-96-128) and it was determined that the UFSAR should be changed to revise the requirement that 'certain key locations are to be tested annually for the effects of concrete aging on shielding integrity'. Upon review of the UFSAR for this operability evaluation, it was noticed that the concrete wall surrounding the Drywell was being referred to as the Biological Shield Wall in other sections. All the other changes to the UFSAR addressed by this evaluation are the result of changing 'Biological Shield' to 'Reactor Shield' or 'Containment Shield' to differentiate between the two walls.

Result: This evaluation determined that an unreviewed safety question did not exist.

Safety Evaluation Number: 1997-01-031

Type of Safety Evaluation:

FSAR Change

Evaluation Reference Number: DFL 96-140

Title: DFL 96-140

Description: The proposed change is to impose limits on 1) the service water inlet temperature such that it must be less than or equal to 75 degrees F when the unit is in operation and 2) the torus bulk temperature such that it must be less than or equal to 75 degrees F. This ensures that the rate of heat removal from containment is maintained, adequate ECCS pump NPSH is provided, and the open operability evaluations are addressed.

This UFSAR change maintains the heat removal capability of the Containment Cooling System by limiting the service water inlet temperature thus ensuring the containment cooling function. The containment cooling function a) limits the peak containment and suppression pool temperature and pressure b) ensures the Mark I and Torus attached piping loading is within the current analysis and c) ensures that the 20 psi differential is maintained between the service water and LPCI side of the heat exchanger with reduced CCSW flow thus ensuring no radioactive release to the environment. By taking this administrative action, the containment and ECCS design parameters are met ensuring the containment cooling systems can perform their intended function.

Result: This evaluation determined that an unreviewed safety question did not exist.

10 CFR 50.59
Safety Evaluation Summary Report

Quarter: 1997-01

Safety Evaluation Number: 1997-01-032

Type of Safety Evaluation: FSAR Change

Evaluation Reference Number: DFL 96-141 and TSUP 3/4.7K

Title: DFL 96-141 and TSUP 3/4.7K

Description: During the Dresden Independent Safety Inspection, a concern was raised that under DBA LOCA conditions with LPCI and CS pumps at above rated flow, it is possible under some circumstances to have less than required NPSH. Operability evaluation ID 96-18 was initiated to evaluate this concern. Subsequently a calculation of the ECCS torus suction strainer head loss was discovered that changed the loss from 1 foot at 10,000 gpm to 5.8 feet. As a result operability evaluation 96-68 was performed. The purpose of this change is to revise UFSAR Sections 6.2.2.3.2 and 6.3.3.4.3 and clarify Technical Specification (TSUP) Basis 3/4.7K regarding the impact from these concerns on NPSH for the LPCI and Core Spray pumps. These calculations of NPSH utilize the reconstituted head loss for the ECCS torus suction strainer and the torus water and CCSW water temperature administrative controls presented in pending UFSAR change DFL 96-140. This will allow operability assessments 96-18 and 96-68 to be closed.

The change to UFSAR Section 6.2.2.3.2 revised the torus suction strainer head loss.

The change to UFSAR Section 6.3.3.4.3 documents the new NPSH calculations for short term (injection phase, no manual action) and long term (containment cooling phase, manual action).

Result: This evaluation determined that an unreviewed safety question did not exist.

Safety Evaluation Number: 1997-01-033

Type of Safety Evaluation: FSAR Change

Evaluation Reference Number: DFL 96-145

Title: DFL 96-145

Description: Section 6.3.2.1.3.3, para 2: The other instrumentation listed is both local and remote (in the control room), thus the word "local" is removed. The discharge pressure was never monitored locally. The percent flow may have been indicated as a secondary function of the Flow Indication Switch that controlled the minimum flow valve. Neither of these indications have operator action associated with them. These clarifications do not change the original design.

Section 6.3.4.1, para 1.b: Remove the word "test" from leak-off drain lines and drainline which were added incorrectly during rebaselining of FSAR. These lines are test provisions and are not themselves to be tested. This correction returns the description to reflect original design.

Section 6.3.4.1, para 2.B.2: Change nomenclature from "startup valve" to the correct "injection valve". This has no affect on design.

Section 6.3.4.1, para 3: Delete the end of the last sentence since it can imply that the pump running in test, shuts off and then restarts upon a receipt of an actuation signal. The pump continues to run per design as stated in the UFSAR Section 6.3.2.1.4. This reflects original design.

Result: This evaluation determined that an unreviewed safety question did not exist.

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Quarter: 1997-01

Safety Evaluation Number: 1997-01-035

Type of Safety Evaluation: Procedure

Evaluation Reference Number: DEP 0500-06

Title: Revision to DEP 0500-6

Description: This Reactor Protection System (RPS) logic is jumpered out to prevent a full SCRAM when deenergizing one of the RPS Buses. With the reactor in mode 4 or 5 there is an automatic bypass around all of the MSIV closure and condenser low vacuum trip relays so that these conditions do not hold the RPS in a tripped condition. The design of the bypass relay logic is such that loss of power to either RPS bus will result in the removal of the Bypass in both trip systems. This results in a full SCRAM. During those periods when RPS bus maintenance activities are being performed the Bypass relay contacts are jumpered so the bypass is not removed.

Result: This evaluation determined that an unreviewed safety question did not exist.

Safety Evaluation Number: 1997-01-036

Type of Safety Evaluation: FSAR Change

Evaluation Reference Number: DFL 96-026

Title: FSAR Section 7.6.1.5.3.2, Figure 7.6-17

Description: FSAR Section 7.6.1.4.3.2 states incorrectly that the reference APRM downscale trip provides a rod block setpoint. Instead, a rod block is provided if the channel reading falls below the downscale trip setpoint which is specific to RBM circuitry.

FSAR Figure 7.6-17 needs a generic change for the note to reference the COLR (Core Operating Limits Report) for RBM setpoint guidance, as already specified in the Tech Specs.

Result: This evaluation determined that an unreviewed safety question did not exist.

Safety Evaluation Number: 1997-01-037

Type of Safety Evaluation: FSAR Change

Evaluation Reference Number: DFL 96-022

Title: Sections 5.4.6.2 & 7.3.4

Description: Change UFSAR sections 5.4.6.2 and 7.3.4 by deleting reference to 1070 psig as pressure at which the Iso Condenser automatically initiates.

Result: This evaluation determined that an unreviewed safety question did not exist.

Safety Evaluation Number: 1997-01-038

Type of Safety Evaluation: FSAR Change

Evaluation Reference Number: DFL 96-020

Title: Administrative Controls for Manually-Operated Valves

Description: Revise text concerning the administrative controls for manually-operated valves associated with process piping which penetrates Primary Containment. As a result of the present text being vague, Dresden Station developed a technical position for defining which manually-operated valves are Primary Containment Isolation Valves (PCIVs) and which of these valves are required to be locked in order to safeguard containment integrity.

Result: This evaluation determined that an unreviewed safety question did not exist.

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Safety Evaluation Number: 1997-01-039

Type of Safety Evaluation: FSAR Change

Evaluation Reference Number: DFL 96-018

Title: Reference to HPCI Low Steam Pressure Setpoint

Description: Delete UFSAR reference to HPCI low steam pressure setpoint.

Result: This evaluation determined that an unreviewed safety question did not exist.

Safety Evaluation Number: 1997-01-040

Type of Safety Evaluation: FSAR Change

Evaluation Reference Number: DFL 96-010

Title: Volume 5, Section 9.1.4.2.2

Description: Add the following statement to Volume 5 Section 9.1.4.2.2 page 9.1-18, "As an alternative to the digital load limiter, station procedures require supervising personnel to ensure load hangups do not occur during reactor building crane operation." This statement is being included to update the UFSAR with the current design requirements of the Reactor Building overhead crane.

Result: This evaluation determined that an unreviewed safety question did not exist.

Safety Evaluation Number: 1997-01-041

Type of Safety Evaluation: FSAR Change

Evaluation Reference Number: DFL 96-006

Title: FSAR Sections 8.3 and 9.5

Description: The change in description is to 1) correct the description to match the field (original design) and 2) to allow for tolerances in respective setpoints. FSAR 8.3 and 9.5

Result: This evaluation determined that an unreviewed safety question did not exist.

Safety Evaluation Number: 1997-01-042

Type of Safety Evaluation: FSAR Change

Evaluation Reference Number: DFL 96-002, NTS 237-200-95-494

Title: HPCI Normal Suction

Description: Change the UFSAR description of the HPCI normal suction from 2/3B CST to either the 2/3B or the 2/3A CST. This change will allow the flexibility of being aligned to either tank. P&ID M35-1 and the procedures in paragraph 2 above already reflect this flexibility by specifying that valves be aligned to take water from the 2/3 A CST.

Result: This evaluation determined that an unreviewed safety question did not exist.

Safety Evaluation Number: 1997-01-043

Type of Safety Evaluation: Nuclear Work Request

Evaluation Reference Number: 950018646-02

Title: Movement of 3A Core Spray Pump Motor

Description: This safety evaluation reviews the affect on Unit 3 Plant Operations during movement of the 3A Core Spray Pump Motor. All other design issues and affects are considered in Temporary Rigging Permit 95-191. This Safety Evaluation and its precautions apply during removal and installation of the motor.

Result: This evaluation determined that an unreviewed safety question did not exist.

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Safety Evaluation Number: 1997-01-044

Type of Safety Evaluation: Procedure

Evaluation Reference Number: SPI 97-03-02

Title: Unit 1 Service Air Supply

Description: The normal Service Air supply to Unit 1 will be temporarily replaced by a portable air compressor, located outside the Station Sewage Ejector House. This Special Procedure will install, operate and then remove the temporary air compressor to supply Unit 1 Service air (cross-tied to Instrument Air) while the normal feed from Unit 2 Service Air is out of service for maintenance.

Result: This evaluation determined that an unreviewed safety question did not exist.

Safety Evaluation Number: 1997-01-045

Type of Safety Evaluation: Set Point Changes

Evaluation Reference Number: SPC 2/3-96-016

Title: High Flow Setpoint for FS 2(3)-2354

Description: This change revised the high flow setpoint of FS 2(3)-2354 to ensure the system design limit contained in the DBD (1200 GPM) for minimum flow valve 2(3) 2301-14 closure is met.

Result: This evaluation determined that an unreviewed safety question did not exist.

Safety Evaluation Number: 1997-01-049

Type of Safety Evaluation: FSAR Change

Evaluation Reference Number: DFL 97-024

Title: J.L. Manta painting procedure JLM/QWP-09.3, Rev. 11

Description: This procedure controls the touch up painting activities in the torus and drywell in support of maintenance and modification work at Dresden Station. The coating systems have been qualified to design basis accident conditions and have been approved by ComEd's System Materials Analysis Level III Coating Specialist. The painting procedure is required to perform the remedial coating activities for the drywell.

Result: This evaluation determined that an unreviewed safety question did not exist.

Safety Evaluation Number: 1997-01-050

Type of Safety Evaluation: Procedure

Evaluation Reference Number: DES 8000-01

Title: Reactor Protection M-G set output Breaker Trip Device Test

Description: This revision to procedure DES 8000-01 will install temporary power to the radiation monitoring equipment located in Main Control Room Panel 903-10 to preclude auto-initiation of the Standby Gas Treatment System during the surveillance testing of the RPS MG set output breakers. The previously used temporary power from revision 2 of this procedure did not meet the requirements for operability of the effected rad monitors. The new temporary power source will maintain the operable status of the radiation monitors, which will allow fuel transfer activities to continue during the testing of the RPS breakers. The use of this temporary source of power is expected to last for the duration of the Reactor Protection System testing which is normally about two days.

Result: This evaluation determined that an unreviewed safety question did not exist.

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Quarter: 1997-01

Safety Evaluation Number: 1997-01-051

Type of Safety Evaluation: FSAR Change

Evaluation Reference Number: DFL 96-112, Rev. 1

Title: DFL 96-112, Rev. 1

Description: The UFSAR changes associated with this 50.59 evaluation are the result of an independent review of the UFSAR and includes changes initiated under DFL 96014 which is being superseded so that all RWCU changes can be performed at the same time. These changes are already marked up in the pending change pages of the next revision to the UFSAR (not issued) and reflect the design document initiating the change. The changes previously addressed under DFL 95014 were initiated as a result of Mod M12-2-90-018 for RWCU piping components and the 50.59 evaluation addresses these changes. An additional change is added to this evaluation to address the abandonment of the RWCU filters which was performed under M12-2-86-037 and P12-3-90-644. The remainder of the changes are the result of the UFSAR review and provide typographical corrections and minor technical enhancements and/or correction to the descriptions of the design basis. These changes do not alter the original intent of the design basis of the station and only reflect the current plant configuration.

This is revision 1 to this Safety Evaluation. Also see the original, 1996-04-286, dated 10/5/96.

Result: This evaluation determined that an unreviewed safety question did not exist.

Safety Evaluation Number: 1997-01-052

Type of Safety Evaluation: FSAR Change

Evaluation Reference Number: DFL 96-040

Title: DFL 96-040

Description: The UFSAR will be updated to include a section 9.5.10 for the Station Blackout system description and licensing basis. The Table of Contents will be changed to reflect the addition of section 9.5.10. Sections 8.1.3 and 8.3 are revised to discuss the addition of Station Blackout to the station auxiliary power systems. Figure 8.2.1 is revised to show the addition of the Station Blackout generators in the station single-line electrical diagram. Table 8.3-1 has been updated to describe the SBO generator capacity and size.

Result: This evaluation determined that an unreviewed safety question did not exist.

Safety Evaluation Number: 1997-01-055

Type of Safety Evaluation: FSAR Change

Evaluation Reference Number: DFL 96-098

Title: DFL 96-098

Description: The UFSAR changes resulting from this review are for minor technical clarifications and do not change the intent of the UFSAR and do not constitute a change to the facility. One technical change in Section 12.3.2.1 does delete reference to 10 CFR 50, Appendix I and the rules of the Illinois Department of Public Health which do not address occupational exposure limits which are annotated in 10 CFR 20.

This is revision 1 to this Safety Evaluation. Also see the original, 1996-04-281, dated 10/7/96.

Result: This evaluation determined that an unreviewed safety question did not exist.

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Safety Evaluation Number: 1997-01-056

Type of Safety Evaluation:

Set Point Changes

Evaluation Reference Number: SPC# 03-96-049

Title: SPC# 03-96-049

Description: The SBGTS Inlet Damper 7505B tripped while being cycled. Based on a walkdown of the 480Vac feed circuit at MCC 39-2B1, it has been determined that the Thermal Overload relay currently installed was not ambient compensated but should be based on the service application. The heater is also marginally sized and will be increased. This setpoint change will specify the appropriate ambient compensated TOL (relay and heater) to be installed in the 7505B SBGTS Inlet Damper at MCC 39-2 Compartment B1.

Result: This evaluation determined that an unreviewed safety question did not exist.

Safety Evaluation Number: 1997-01-058

Type of Safety Evaluation:

Miscellaneous

Evaluation Reference Number: OOS 950011754 & 960003741

Title: OOS 950011754 & 960003741

Description: This evaluation addresses the permanent abandonment in place and the installation of the Out of Service Tags under OOS 950011754 & 960003741 for the drywell pneumatic air operated supply valves 2(3)-4720 and 2(3)-4721. This drywell pneumatic compressor abandonment was performed at an earlier date and is documented in the UFSAR and on design drawing (P&ID) M-37, Sheet 2 and M-367, Sheet 2. As part of this abandonment, the listed valves were placed in the closed position and tagged out-of-service (OOS). These valves were previously used to control supply of compressed drywell atmosphere for the drywell pneumatic supply system. Drawing M-37, Sheet 2, Revision RH and M-367, Sheet 2, Revision AJ shows that this equipment is abandoned in place or not in use. Additionally section 9.3.1.5 states that, "The drywell pneumatic compressors have been permanently removed from service. The cross tie to the pump back system supplies the pressurized drywell gas to this system." However, the valve status as designated in Table 3.2-9, Sheet 5 was not revised to reflect the permanent out of service. This evaluation addresses the change to the table and designates that the valves normal status is in the closed position. The original safety evaluation for this abandonment was not found. Therefore, this evaluation addresses the OOS of the valves and along with OOS 950014864 is an evaluation performed for drywell pneumatic compressor abandonment design change.

Result: This evaluation determined that an unreviewed safety question did not exist.

Safety Evaluation Number: 1997-01-059

Type of Safety Evaluation:

Exempt Change

Evaluation Reference Number: EC E12-3-96-212

Title: EC E12-3-96-212

Description: The scope of the change is to add two safety related check valves in series to each reactor recirculation (RR) pump seal purge line to prevent the potential bypass of secondary containment. Additionally, test taps are added to each line to facilitate check valve leakage testing. The RR seal purge water is directed through the drywell penetrations to the RR pump seal cavities via the seal cavity pressure instrument lines.

Result: This evaluation determined that an unreviewed safety question did not exist.

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Quarter: 1997-01

Safety Evaluation Number: 1997-01-060

Type of Safety Evaluation: FSAR Change

Evaluation Reference Number: DFL 96-125

Title: DFL 96-125

Description: Engineering was requested to resolve discrepancies between UFSAR temperature values for the condensate system and those obtained by field measurement. The results are documented in letter 5210516. The discrepancies are to be resolved by removal from UFSAR Section 10-4 of Tables 10.4-2, Condensate Pump Characteristics, and Table 10.4-3, Condensate Booster Pump Characteristics. Said tables list operating characteristics of non-safety related pumps. Tables are meant to provide general information only. However, values have turned out to be too restrictive and do not coincide with actual operating values. Because tables lead the reader to believe that the values listed are strict design limits (which they are not), then it is best to delete the tables to avoid future misinterpretation. Current UFSAR text description of said pumps has sufficient detail to adequately describe the pumps and their design function.

Result: This evaluation determined that an unreviewed safety question did not exist.

Safety Evaluation Number: 1997-01-061

Type of Safety Evaluation: Modification

Evaluation Reference Number: M12-2(3)-94-004

Title: Core Shroud Horizontal Welds

Description: This modification is a mechanical repair to the Dresden Unit 2 core shroud horizontal welds H1 through H7. Its purpose is to structurally replace the horizontal welds should the horizontal welds eventually crack through wall.

Result: This evaluation determined that an unreviewed safety question did not exist.

Safety Evaluation Number: 1997-01-062

Type of Safety Evaluation: Modification

Evaluation Reference Number: DCP 9600318 DCN 001049M

Title: Modification to Torus Penetrations Going to the ECCS Pump Suction Ring Header

Description: The modification to the Torus penetrations going to the ECCS pump suction ring header is required in order to support the installation of the new ECCS pump suction ring header suction strainers. The new strainers will be bolted to the flanges of the existing penetration piping and will be supported by the flange and piping. The larger strainers result in increased loads on the penetrations, which have been evaluated as part of this Design Change Package. The evaluation of the adequacy of the penetration and flange are documented in calculations DRE97-0007 and DRE 97-0019. The evaluations include applicable seismic and pool hydrodynamic loads and demonstrate that all applicable UFSAR acceptance limits are met. Dresden Station committed to installation of larger suction strainers in their interim response to NRC Bulletin 96-03 and their request for deferral of full NRC Bulletin 96-03 compliance.

Result: This evaluation determined that an unreviewed safety question did not exist.

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Quarter: 1997-01

Safety Evaluation Number: 1997-01-063

Type of Safety Evaluation:

Modification

Evaluation Reference Number: DCP 9600318 ECN 001050M

Title: Install New Torus ECCS Ring Header Suction Strainers

Description: The purpose in performing this modification is to increase the surface area of the strainers and thus increase the flow area of the strainer. NRC Bulletin 96-03, "Potential Plugging of Emergency Core Cooling Suction Strainers by Debris in Boiling-Water Reactors", describes ECCS suction strainer clogging events and required responses to satisfy the Bulletin. This modification will address at least a portion of the actions required by the bulletin, but at this time it is not assumed that the modification or the associated 50.59 will disposition all issues associated with Bulletin 96-03. Compliance with 96-03 will be fully addressed, separate from this modification, to ensure full compliance with the requirements of the NRC bulletin. Structural qualification of the strainers and flange bolting is documented in calculations DRE97-0016 and DRE97-0019, which conclude that the new configuration meets the applicable UFSAR allowable stress limits for all load combinations, including pool hydrodynamic and seismic loads.

Result: This evaluation determined that an unreviewed safety question did not exist.

Safety Evaluation Number: 1997-01-064

Type of Safety Evaluation:

Exempt Change

Evaluation Reference Number: E12-3-95-251

Title: HPCI Oil Pump Pressure Switches

Description: This exempt change will replace three HPCI Oil Pump Pressure Switches PS-003-2303-PS1, PS3, PS4. The function of the pressure switches is safety related. These switches are being replaced due to potential internal diaphragm leakage. The existing switches are obsolete, and replacement parts are not available. A test valve will be added in-line with PS1 for calibration purposes.

Result: This evaluation determined that an unreviewed safety question did not exist.

Safety Evaluation Number: 1997-01-066

Type of Safety Evaluation:

Exempt Change

Evaluation Reference Number: E12-2-96-202

Title: Hydrogen Supply Line

Description: This change adds a cross connect pipe in the Unit 2 Turbine Building between the hydrogen supply line to the Generator and the hydrogen supply line to the Hydrogen Water Chemistry system, and cuts and caps the existing Generator hydrogen supply line near the floor of the Turbine Building. This change also de-terminates the cables associated with the controls and instrumentation for the abandoned hydrogen supply line to Unit 2.

This change is required due to a failure in the existing buried hydrogen supply piping from the hydrogen tank farm to the Unit 2 Generator.

Result: This evaluation determined that an unreviewed safety question did not exist.

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Quarter: 1997-01

Safety Evaluation Number: 1997-01-067

Type of Safety Evaluation:

Exempt Change

Evaluation Reference Number: E12-2-95-262

Title: Unit 2 CRD SDV Galleries

Description: This change is based on the results of an evaluation of the Unit 3 CRD-SDV galleries for additional piping reactions from proposed permanent lead shielding on the Unit 3 CRD-SDV piping. The Unit 3 evaluation identified discrepancies between the as-built configuration and the drawing configuration of the galleries, and also identified modeling and design discrepancies in the Unit 3 CRD-SDV gallery design basis calculations. A QE-ENC-40.1 evaluation was performed and the Unit 3 CRD-SDV galleries were found to be operable. A commitment was made to the NRC (LER 3-95-022) to evaluate the Unit 2 CRD-SDV galleries prior to restarting Unit 2 from D2R14. In order to expedite work, the engineering evaluation of the Unit 2 CRD-SDV galleries will be performed in parallel with the reinforcement installation. Therefore, reinforcement installation details must be issued prior to calculation approval. The final installed configuration will be qualified prior to Unit 2 restart.

Result: This evaluation determined that an unreviewed safety question did not exist.

Safety Evaluation Number: 1997-01-068

Type of Safety Evaluation:

Exempt Change

Evaluation Reference Number: E12-2-96-200

Title: Replace Valve Body of MOV 2/3-3099-60A

Description: The scope of this exempt change is to install a 3" Velan gate valve body with a reducer assembly to replace the existing 2.5" Walworth gate valve body of MOV 2/3-3099-60A located on the piping from the Liquid Radwaste Reboiler to the Unit 2 Main Condenser. The change is proposed as a result of corrective maintenance work performed to replace corroded sections of piping around MOV 2/3-3099-60A. The subject valve body was inspected and found to be severely degraded. A like-for-like replacement was not feasible due to long lead time. A 3" Velan gate valve available at the site storeroom, equipped with a mounting plate to hook up the existing Limitorque Operator shall be installed. To facilitate the installation of the subject and reducer assembly, existing support located above manual gate valve 2/3-3099-71 shall be removed and in addition, existing support at the bottom of the riser supporting the subject MOV and the manual gate valve shall be modified.

Result: This evaluation determined that an unreviewed safety question did not exist.

Safety Evaluation Number: 1997-01-069

Type of Safety Evaluation:

FSAR Change

Evaluation Reference Number: DFL 97-010

Title: DFL-97-010

Description: The subject change is to remove reference to the continuous in-process monitors as described in UFSAR Sections 9.3.2.4, 11.3.2.1.6, Figures 11.3-14 & 15 and on drawings identified in DCR 970038. These monitors perform a grab sample at the discharge point of the charcoal absorbers with continuous main control room alarm of high radiation levels in the sample. These monitors are currently not installed in the plant, therefore, a review of the system has been performed and was documented per DOC ID 5182094. Upon conclusion, it was recommended that an As-Built DCR and a UFSAR revision be generated and tracked per NTS 2372019616203. Doc Id 5182094 also stated as an assumption that the stack monitor fulfills the function that this in line sample monitor was intended to perform. Since that time, this assumption has been confirmed and is documented in the UFSAR package DFL 97-010.

Result: This evaluation determined that an unreviewed safety question did not exist.

Safety Evaluation Summary Report

Safety Evaluation Number: 1997-01-070

Type of Safety Evaluation:

Exempt Change

Evaluation Reference Number: P12-2-94-209

Title: Replace Flexible Metallic Bellows

Description: The purpose of this change is to replace the two-ply flexible metallic bellows over the isolation condenser steam supply drywell penetration X-108A (U2).

Result: This evaluation determined that an unreviewed safety question did not exist.

Safety Evaluation Number: 1997-01-071

Type of Safety Evaluation:

Exempt Change

Evaluation Reference Number: P12-2-94-208

Title: Replace Flexible Metallic Bellows

Description: The purpose of this change is to replace the two-ply flexible metallic bellows over the LPCI pump discharge drywell penetration X-116A (U2).

Result: This evaluation determined that an unreviewed safety question did not exist.

Safety Evaluation Number: 1997-01-072

Type of Safety Evaluation:

Modification

Evaluation Reference Number: M12-2-94-008

Title: Replace Existing Evaporator Cooler (2-5711)

Description: At the present time, the cooling of the various areas inside the reactor building is provided by the evaporative cooler of the reactor building ventilation system (ref. M-239). The proposed modification replaces the existing evaporator cooler (2-5711) with new chilled water cooling coils to be served by a new chilled water system (ref. Drawing 2-CWS-M01). Additionally, the existing roll-type filter (2-5717) is also being replaced with new bag-type filter. The replacement cooling coils and replacement air filter are to be installed inside the existing plenums at the same location as the existing equipment. Existing drains in the turbine building will be used to remove condensate from the chilled water coils. The existing coolers, filters and other associated installations will be demolished and removed.

Result: This evaluation determined that an unreviewed safety question did not exist.

Safety Evaluation Number: 1997-01-073

Type of Safety Evaluation:

Modification

Evaluation Reference Number: M12-2-90-066

Title: SBTG Suction Isolation Valve

Description: The SBTG system can draw suction from Unit 2 and Unit 3 reactor buildings. The 2-7503 and 3-7503 are the isolation valves on this suction duct work (spiral wound pipe). In the original design configuration, the Unit 3 SBTG suction isolation valve would close on high radiation signal from Unit 2 and the Unit 2 SBTG suction isolation valve would close on a high radiation signal from Unit 3. This configuration infers that the SBTG can operate on a unit specific basis. However, the open refueling floor is common to both Unit 2 and Unit 3 reactor buildings. Because of the failure history of this SBTG suction isolation valve and/or the associated isolation logic, this modification was implemented to electrically disconnect valve 2-7503 in the open position.

Result: This evaluation determined that an unreviewed safety question did not exist.

Safety Evaluation Summary Report

Safety Evaluation Number: 1997-01-074

Type of Safety Evaluation:

Modification

Evaluation Reference Number: M12-2-90-057D

Title: Replace Motor on Iso Condenser Makeup Valve 2-4399-74

Description: This partial modification provides for the replacement of the ac motor on isolation condenser makeup valve 2-4399-74 with a dc motor. To accommodate the motor change, a dc power supply will be provided to the motor. The gearing and springpack will also be changed in the MOV. These changes are part of a larger modification to upgrade the capability of the isolation condenser to operate, if needed, during the loss of offsite ac power. Modification M12-2-90-057C provided two diesel driven isolation condenser makeup pumps that do not require ac power to start or operate. Valve 2-4399-74 is a normally closed valve that is remote manually opened from the control room when the pumps start. Providing a dc motor for this valve is the final step required to get makeup water to the isolation condenser during loss of ac power.

Result: This evaluation determined that an unreviewed safety question did not exist.

Safety Evaluation Number: 1997-01-075

Type of Safety Evaluation:

Modification

Evaluation Reference Number: M12-0-91-019E

Title: Switchgear Bus 24

Description: This partial modification consists of Switchgear Bus 24 circuit breaker and secondary stationary contact (auxiliary) switch replacement, and the addition of two new cubicles. The new circuit breakers will increase the bus rating from 250 MVA to 350 MVA. The ampere rating of the circuit breaker will not be changed by this modification. The presently installed air magnetic circuit breakers will be replaced with Sulfur Hexafluoride gas insulated circuit breakers which will be a "roll in fit". Circuit breakers will be furnished complete and ready for installation. The circuit breaker replacement will increase the interrupting rating of Bus 24 from 250 MVA to 350 MVA to envelop the installed system requirements. The replacement of the secondary stationary contacts and associated linkage with new design components will improve the operability and reduce maintenance. There will be no control system changes associated with the auxiliary switch as the circuitry remains the same. New switches at the DCS panels will control the SBO tie-in circuit breaker. The two new cubicle additions will serve as a third source of power to Bus 24 (from SBO Bus 61) and as a spare for a future RAT. This third source of power is in order to meet the requirements of Station Blackout Rule 10 CFR 50.63, "Loss of all Alternating Current (ACC) Power".

Result: This evaluation determined that an unreviewed safety question did not exist.

Safety Evaluation Number: 1997-01-076

Type of Safety Evaluation:

FSAR Change

Evaluation Reference Number: DFL 95-019

Title: DFL 95-019

Description: Change the position of valves 2(3)-1501-35 to eliminate an operator work-around.

Result: This evaluation determined that an unreviewed safety question did not exist.