NRC Form 366
(9-83)

U.S. NUCLEAR REGULATORY COMMISSION
APPROVED OMB NO. 3150-0104
EXPIRES: 8/31/85

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1)	DOCKET NUMBER (2)	PAGE (3)
Cooper Nuclear Station	0 5 0 0 0 2 9 8	1 OF 0 2

TITLE (4)
High Pressure Coolant Injection System Inoperability

EVENT DATE (5)			LER NUMBER (6)					REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)	
MONTH	DAY	YEAR	YEAR		SEQUENTIAL NUMBER		REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES	DOCKET NUMBER(S)
0 8	2 5	8 5	8 5	—	0 0 9	—	0 0	0 9	2 4	8 5		0 5 0 0 0
												0 5 0 0 0

OPERATING MODE (9)	N	THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: (Check one or more of the following) (11)			
POWER LEVEL (10)	0 1 1	20.402(b)	20.405(c)	50.73(a)(2)(iv)	73.71(b)
		20.405(a)(1)(i)	50.36(c)(1)	X 50.73(a)(2)(v)	73.71(c)
		20.405(a)(1)(ii)	50.36(c)(2)	50.73(a)(2)(vii)	OTHER (Specify in Abstract below and in Text, NRC Form 366A)
		20.405(a)(1)(iii)	50.73(a)(2)(i)	50.73(a)(2)(viii)(A)	
		20.405(a)(1)(iv)	50.73(a)(2)(ii)	50.73(a)(2)(viii)(B)	
		20.405(a)(1)(v)	50.73(a)(2)(iii)	50.73(a)(2)(x)	

LICENSEE CONTACT FOR THIS LER (12)

NAME	TELEPHONE NUMBER
	AREA CODE
E. M. Mace, Plant Engineering Supervisor	4 0 2 — 8 2 5 - 3 8 1 1

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS		CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS	
B	B J	V 2 0	A 3 9 5	Y							

SUPPLEMENTAL REPORT EXPECTED (14)		EXPECTED SUBMISSION DATE (15)	MONTH	DAY	YEAR
YES (If yes, complete EXPECTED SUBMISSION DATE)	X NO				

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

At 0200, August 25, 1985, the steam inlet valve to the High Pressure Coolant Injection (HPCI) System failed to open with the motor operator. The HPCI system had been declared inoperable eleven hours earlier due to failure of the turbine governor system (reference LER 85-008). In accordance with Technical Specifications, testing was already in progress to verify operability of the other coolant injection systems and the Automatic Depressurization System due to HPCI inoperability. Reactor pressure was approximately 930 psig and reactor power was approximately 11% of rated. The reactor had been critical for approximately 4.5 days. The valve was manually unseated and stroked several times with the motor operator. Amperage readings taken indicated no adjustments or repairs were necessary and that the operator and valve were operating freely. The valve was returned to service approximately two hours after first discovery of inoperability. The valve has been functionally tested satisfactorily on several occasions in the last 30 days. No Technical Specifications were violated and the event represented minimal safety consequence. One known factor may have contributed to the valve sticking in the seat. The valve disk (gate) had been restellited during the outage. The failure of the valve to open is not considered generic or recurrent in nature. No corrective action is planned.

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		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	
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TEXT (If more space is required, use additional NRC Form 366A's) (17)

At 0200, August 25, 1985, the steam inlet valve to the High Pressure Coolant Injection (HPCI) System failed to open with the motor operator. The HPCI system had been declared inoperable eleven hours earlier due to failure of the turbine governor system (reference LER 85-008). In accordance with Technical Specifications, testing was already in progress to verify operability of the other coolant injection systems and the Automatic Depressurization System due to HPCI inoperability. Reactor pressure was approximately 930 psig and reactor power was approximately 11% of rated. The reactor had been critical for approximately 4.5 days.

Maintenance personnel reported finding the valve stuck in its seat. The valve was then manually unseated and cycled full open and closed several times. Open, close, and running torque amperage readings were recorded and found to be within specifications. No repairs were made and the valve was returned to service.

The valve had been operated numerous times prior to the event and no anomalies had been noted. The cause of the failure of the valve to open cannot be conclusively determined. One factor is known which may have contributed to the failure. The valve disk was restellited during the outage. Another contributing factor may be that the valve operator control circuitry had been temporarily modified to allow throttling of the valve to aid in the troubleshooting of governor problems (reference LER 85-008).

The valve was returned to service approximately two hours after it was found to be inoperable. The valve has been functionally tested satisfactorily on several occasions in the last 30 days. No anomalies were observed.

A review of the equipment history file found no evidence of previous similar failure. Similar recurrence is not expected. No corrective action is planned.

Pertinent component data follows:

Valve

CNS Component Designation - HPCI-MOV-MO14
Mfr - Anchor Valve Co.
Size - 10 in.
Model - 772-3
Type - Gate
Matr - A216 WCB

Operator

Mfr - Limitorque Corp.
Model - SMB-1
Torque - 60 ft-lb
Speed - 20 sec. stroke
Volt - 250 VDC
Full Load - 16.5 amp
Locked Rotor - 95 amp

Nebraska Public Power District

COOPER NUCLEAR STATION
P.O. BOX 98, BROWNVILLE, NEBRASKA 68321
TELEPHONE (402) 825-3811

CNSS850557

September 24, 1985

U. S. Nuclear Regulatory Commission
Document Control Desk
Washington, D.C. 20555

Dear Sir:

Cooper Nuclear Station Licensee Event Report 85-009 is forwarded as an attachment to this letter.

Sincerely,

P. V. Thomason
Division Manager of
Nuclear Operations

PVT:lb
Attach.
cc: R. D. Martin
L. G. Kuncl
J. D. Weaver
L. R. Berry
INPO Records Center
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