



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

SEP 25 1985

Docket No.: 50-458

Mr. William J. Cahill, Jr.  
Senior Vice President  
River Bend Nuclear Group  
Gulf States Utilities Company  
Post Office Box 2951  
Beaumont, Texas 77704  
ATTN: Mr. J. E. Booker

Dear Mr. Cahill:

SUBJECT: REQUEST FOR INFORMATION AND COMMENTS RE: FULL POWER OPERATING  
LICENSE FOR RIVER BEND STATION

On August 29, 1985, the U. S. Nuclear Regulatory Commission (NRC) issued a low power Facility Operating License (NPF-40) together with Technical Specifications and Environmental Protection Plan for the River Bend Station, Unit 1. License No. NPF-40 authorizes operation of River Bend, Unit 1 at reactor power levels not in excess of 2894 megawatts thermal (100% rated power). However, pending Commission approval, operation is restricted to power levels not to exceed five percent of rated power (144.7 megawatts thermal).

Enclosed for your review comment is a draft of the full power operating license for River Bend Station, Unit 1. Please review this draft license and provide your written comments 14 days in advance of your anticipated need for this license.

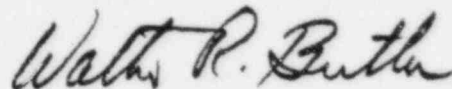
For the NRC to issue a full power license, GSU must satisfy all conditions of the low power license which specify resolution prior to exceeding 5% rated power as well as those commitments made by GSU which are to be accomplished prior to exceeding 5% rated power. In order to effectively schedule the staff's review efforts to support your licensing schedule, we need to have status reports relative to your progress on these license conditions and other commitments. Accordingly, we request that you forward a weekly report commencing on September 27, 1985, identifying those license conditions and other commitments that have been completed during the previous week and cumulatively since issuance of the low power license.

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Please send copies of the progress report to Stephen M. Stern, the staff's Project Manager for the River Bend Application, as well as the NRC's Region IV office and the NRC's Resident Inspector at River Bend.

Please contact Mr. Stern for any clarification of this request.

Sincerely,

A handwritten signature in cursive script that reads "Walter R. Butler".

Walter R. Butler, Chief  
Licensing Branch No. 2  
Division of Licensing

Enclosure:  
As stated

cc: w/enclosure  
See next page

Mr. William J. Cahill, Jr.  
Gulf States Utilities Company

River Bend Nuclear Plant

cc:

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Office of Executive Director  
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UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

GULF STATES UTILITIES COMPANY AND  
CAJUN ELECTRIC POWER COOPERATIVE  
DOCKET NO 50-458  
RIVER BEND STATION, UNIT 1  
FACILITY OPERATING LICENSE

License No. NPF-40

1. The Nuclear Regulatory Commission (the Commission or the NRC) has found that:
  - A. The application for license filed by Gulf States Utilities Company, acting on behalf of itself and Cajun Electric Power Cooperative, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's regulations set forth in 10 CFR Chapter I, and all required notifications to other agencies or bodies have been duly made;
  - B. Construction of the River Bend Station, Unit 1 (the facility) has been substantially completed in conformity with Construction Permit No. CPPR-145 and the application, as amended, the provisions of the Act and the regulations of the Commission;
  - C. The facility will operate in conformity with the application, as amended, the provisions of the Act, and the regulations of the Commission;
  - D. There is reasonable assurance: (i) that the activities authorized by this operating license can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations set forth in 10 CFR Chapter I;
  - E. Gulf States Utilities Company\* is technically qualified to engage in the activities authorized by this operating license in accordance with the Commission's regulations set forth in 10 CFR Chapter I;
  - F. Gulf States Utilities Company and Cajun Electric Power Cooperative have satisfied the applicable provisions of 10 CFR Part 140, "Financial Protection Requirements and Indemnity Agreements," of the Commission's regulations;
  - G. The issuance of this license will not be inimical to the common defense and security or to the health and safety of the public;

\*Gulf States Utilities Company is authorized to act as agent for Cajun Electric Power Cooperative and has exclusive responsibility and control over the physical construction, operation and maintenance of the facility.

- H. After weighing the environmental, economic, technical, and other benefits of the facility against environmental and other costs and considering available alternatives, the issuance of Facility Operating License No. NPF-40, subject to the conditions for protection of the environment set forth herein, is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied; and
  - I. The receipt, possession, and use of source, byproduct and special nuclear material as authorized by this license will be in accordance with the Commission's regulations in 10 CFR Parts 30, 40 and 70.
2. Based on the foregoing findings Facility Operating License NPF-40 is hereby issued to Gulf States Utilities Company and Cajun Electric Power Cooperative (the licensees), to read as follows:
- A. This license applies to the River Bend Station, Unit 1, a boiling water nuclear reactor and associated equipment, owned by Gulf States Utilities Company and Cajun Electric Power Cooperative. The facility is located approximately 2 miles east of the Mississippi River in West Feliciana Parish, Louisiana, approximately 2.7 miles southeast of St. Francisville, Louisiana and approximately 18 miles northwest of the city limits of Baton Rouge, Louisiana, and is described in the licensees' "Final Safety Analysis Report," as supplemented and amended, and in the licensees' Environmental Report-Operating License Stage, as supplemented and amended.
  - B. Subject to the conditions and requirements incorporated herein, the Commission hereby licenses:
    - (1) Gulf States Utilities Company (GSU) and Cajun Electric Power Cooperative to possess the facility at the designated location in West Feliciana Parish, Louisiana, in accordance with the procedures and limitations set forth in this license;
    - (2) GSU, pursuant to Section 103 of the Act and 10 CFR Part 50, to use and operate the facility at the above designated location in accordance with the procedures and limitations set forth in this license;
    - (3) GSU, pursuant to the Act and 10 CFR Part 70, to receive, possess and to use at any time special nuclear material as reactor fuel, in accordance with the limitations for storage and amounts required for reactor operation, as described in the Final Safety Analysis Report, as supplemented and amended;

- (4) GSU, pursuant to the Act and 10 CFR Parts 30, 40 and 70, to receive, possess, and use at any time any byproduct, source and special nuclear material as sealed neutron sources for reactor startup, sealed sources for reactor instrumentation and radiation monitoring equipment calibration, and as fission detectors in amounts as required;
  - (5) GSU, pursuant to the Act and 10 CFR Parts 30, 40 and 70, to receive, possess, and use in amounts as required any byproduct, source or special nuclear material without restriction to chemical or physical form, for sample analysis or instrument calibration or associated with radioactive apparatus or components; and
  - (6) GSU, pursuant to the Act and 10 CFR Parts 30, 40 and 70, to possess, but not separate, such byproduct and special nuclear materials as may be produced by the operation of the facility.
- C. This license shall be deemed to contain and is subject to the conditions specified in the Commission's regulations set forth in 10 CFR Chapter I and is subject to all applicable provisions of the Act and to the rules, regulations and orders of the Commission now or hereafter in effect; and is subject to the additional conditions specified or incorporated below:
- (1) Maximum Power Level  
GSU is authorized to operate the facility at reactor core power levels not in excess of 2894 megawatts thermal (100% rated power) in accordance with the conditions specified herein. The items identified in Attachment 1 to this license shall be completed as specified. Attachment 1 is hereby incorporated into this license.
  - (2) Technical Specifications and Environmental Protection Plan  
The Technical Specifications contained in Appendix A and the Environmental Protection Plan contained in Appendix B, both of which are attached hereto, are hereby incorporated into this license. GSU shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.
  - (3) Antitrust Conditions  
GSU shall comply with the antitrust conditions in Appendix C attached hereto, which is hereby incorporated into this license.



(4) Turbine System Maintenance Program (Section 3.5.1 and 3.3, SER)\*

GSU shall submit a turbine system maintenance program by November 1, 1987. Prior to review and approval of that program by the NRC staff, GSU shall volumetrically inspect all low pressure turbine rotors at the second refueling outage and every other (alternate) refueling outage thereafter.

(5) Seismic and Dynamic Qualification of Seismic Category 1 Mechanical and Electrical Equipment (Section 3.10, SER and SSER 3)

GSU shall complete the requirements of the seismic and dynamic qualification of mechanical and electrical equipment as specified in Attachment 2. Attachment 2 is hereby incorporated into this license.

(6) Equipment Qualification (Section 3.11, SER and SSER 3)

All electrical equipment within the scope of 10 CFR 50.49 shall be environmentally qualified by November 30, 1985.

(7) Mark III Related Issues (Section 6.2.1.9, SER and SSER 2)

- a. GSU shall not use the residual heat removal system in the steam condensing mode.
- b. Prior to startup following the first refueling outage, GSU shall furnish the outstanding information identified in Appendix K of SSER 2 addressing the Mark III containment related issues.

(8) Inservice Inspection Program (Section 5.2.4.3 and 6.6.3, SER and SSER 3)

GSU shall submit the inservice inspection program for NRC staff review and approval by one year from date of this license.

(9) Bypassed and Inoperable Status Indication (Section 7.5.2.2, SER and SSER 3)

Prior to startup following the first refueling outage, GSU shall implement design modifications to improve the capabilities of existing bypassed and inoperable status indication used to monitor the status of safety related systems. The specific design changes to be implemented are identified in a GSU letter dated December 3, 1984 as clarified in a GSU letter dated March 5, 1985.

\*The parenthetical notation following the title of many license conditions denotes the section of the Safety Evaluation Report and/or its supplements wherein the license condition is discussed.

(10) TDI Diesel Engines (Section 8.3.1 SSER 3)

GSU shall complete the TDI diesel requirements as specified in Attachment 3. Attachment 3 is hereby incorporated into this license.

(11) Ultimate Heat Sink (Section 9.2.5, SER and SSER 3)

Prior to startup following the first refueling outage GSU shall have installed and operational in the ultimate heat sink a permanent temperature monitoring system acceptable to the NRC staff and Technical Specification modifications as required.

(12) Fire Protection (Section 9.5.1, SER and SSER 3)

GSU shall comply with the requirements of the fire protection program as specified in Attachment 4. Attachment 4 is hereby incorporated into this license.

(13) Operating Staff Experience Requirements (Section 13.1.2.1, SSER 2)

GSU shall have a licensed senior operator on each shift who has had at least six months of hot operating experience on a plant comparable to River Bend Station, including at least six weeks at power levels greater than 20% of full power, and who has had startup and shutdown experience. For those shifts where such an individual is not available on the plant staff, an advisor shall be provided who has had at least four years of power plant experience, including two years of nuclear plant experience, and who has had at least one year of experience on shift as a licensed senior operator at a comparable facility or its equivalent as approved by the staff. Use of advisors who were licensed only at the RO level will be evaluated on a case by case basis. Advisors shall be trained on plant procedures, Technical Specifications and plant systems, and shall be examined on these topics at a level sufficient to assure familiarity with the plant. For each shift, the remainder of the shift crew shall be trained as to the role of the advisors.

(14) Post-Fuel-Loading Initial Test Program (Section 14, SER and SSER 3)

Any changes to the initial test program described in Section 14 of the FSAR made in accordance with the provisions of 10 CFR 50.59 shall be reported in accordance with 50.59(b) within one month of such change.



(15) Partial Feedwater Heating (Section 15.1, SER)

The facility shall not be operated with partial feedwater heating for the purpose of extending the normal fuel cycle.

(16) Emergency Response Capabilities (Generic Letter 82-33, Supplement 1 to NUREG-0737, Section 7.5.2.4, SER and SSER 3, Section 18, SER, SSER 2 and SSER 3)

GSU shall complete the requirements of NUREG-0737 Supplement #1 as specified in Attachment 5. Attachment 5 is hereby incorporated into this license.

(17) Salem ATWS Event, Generic Letter 83-28

GSU shall submit responses to and implement the requirements of Generic Letter 83-28 on a schedule which is consistent with that given in its letters dated August 3, 1984 and May 20, 1985.

(18) Emergency Planning (Section 13.3, SER, SSER 2 and SSER 3)

In the event the NRC staff finds that the lack of progress in completion of the procedures in the Federal Emergency Management Agency's final rule, 44 CFR Part 350, is an indication that a major substantive problem exists in achieving or maintaining an adequate state of emergency preparedness, the provisions of 10 CFR Section 50.54(s)(2) will apply.

- D. GSU shall fully implement and maintain in effect all provisions of the Commission-approved physical security, guard training and qualification, and safeguards contingency plans, including all amendments and revisions made pursuant to the authority of 10 CFR 50.90 and 10 CFR 50.54(p), which are part of the license. These plans, which contain safeguards information protected under 10 CFR 73.21, are entitled: "River Bend Station Physical Security Plan," "River Bend Station Security Training and Qualification Plan" and "River Bend Station Safeguards Contingency Plan."

- E. Except as otherwise provided in the Technical Specifications or Environmental Protection Plan, GSU shall report any violations of the requirements contained in Section 2.C of this license in the following manner: initial notification shall be made within 24 hours to the NRC Operations Center via the Emergency Notification System with written followup within thirty days in accordance with the procedures described in 10 CFR 50.73(b), (c), and (e).
- F. The licensees shall have and maintain financial protection of such type and in such amounts as the Commission shall require in accordance with Section 170 of the Atomic Energy Act of 1954, as amended, to cover public liability claims.
- G. This license is effective as of the date of issuance and shall expire at midnight on August 29, 2025

FOR THE NUCLEAR REGULATORY COMMISSION

Harold R. Denton, Director  
Office of Nuclear Reactor Regulation

Enclosures:

- 1. Attachments 1-5
- 2. Appendix A - Technical Specifications (NUREG-1142)
- 3. Appendix B - Environmental Protection Plan
- 4. Appendix C - Antitrust Conditions

Date of Issuance:

- E. Except as otherwise provided in the Technical Specifications or Environmental Protection Plan, GSU shall report any violations of the requirements contained in Section 2.C of this license in the following manner: initial notification shall be made within 24 hours to the NRC Operations Center via the Emergency Notification System with written followup within thirty days in accordance with the procedures described in 10 CFR 50.73(b), (c), and (e).
- F. The licensees shall have and maintain financial protection of such type and in such amounts as the Commission shall require in accordance with Section 170 of the Atomic Energy Act of 1954, as amended, to cover public liability claims.
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FOR THE NUCLEAR REGULATORY COMMISSION

Harold R. Denton  
Office of Nuclear Reactor Regulation

Enclosures:

- 1. Attachments 1-5
- 2. Appendix A - Technical Specifications (NUREG-1142)
- 3. Appendix B - Environmental Protection Plan
- 4. Appendix C - Antitrust Conditions

Date of Issuance:

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|---------|---------|-----------|-----------|-----------|---------|
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| WButler | TNovak  | FMiraglia | HThompson | DEisenhut | HDenton |
| 9/ /85  | 9/ /85  | 9/ /85    | 9/ /85    | 9/ /85    | 9/ /85  |

ATTACHMENT 1  
To-NPF 40

Prior to achieving the condition indicated on or before the date indicated, the following items shall be completed to the satisfaction of Region IV:

1. OUTSTANDING ITEMS TO BE ACCOMPLISHED PRIOR TO COMPLETION OF THE INITIAL TEST PROGRAM
  - a. Verify the station electric distribution voltage analyses are in accordance with the guidelines of Branch Technical Position PSB-1, Position 4.
  - b. Perform an engineering evaluation and complete modifications on the battery powered lighting system used in areas of the plant outside the main control room required for safe shutdown and personnel evacuation to upgrade those areas identified as deficient with regard to the requirements contained in FSAR Table 9.5-2
2. OUTSTANDING ITEM TO BE COMPLETED PRIOR TO THE FIRST REFUELING OUTAGE
  - a. Complete and have operational the fuel building sampling system. Off-loading of irradiated fuel prior to the first refueling outage shall be performed as described in GSU's letter dated June 13, 1985.
3. OUTSTANDING ITEMS TO BE COMPLETED PRIOR TO FIRST DESIGN USE OR PRIOR TO STARTUP FOLLOWING THE FIRST REFUELING OUTAGE (WHICHEVER IS FIRST)
  - a. Complete a load test which meets all the OSHA requirements for load handling capability on all remaining cranes and hoists not previously tested.
  - b. Install and have operational the chlorination systems for the normal and standby service water systems for Asiatic clam control prior to first introduction of Mississippi River water into these systems.
4. OUTSTANDING ITEMS TO BE ACCOMPLISHED PRIOR TO STARTUP FOLLOWING THE FIRST REFUELING OUTAGE
  - a. Rework fuse block connectors as delineated in the 10 CFR 50.55(e) report identified as DR-287.
  - b. Repair or replace the control valves on HVAC chillers as delineated in the 10 CFR 50.55(e) report identified as DR-314.
  - c. Verify that adequate radio communication capability exists from all appropriate plant areas.

ATTACHMENT 2

To-NPF 40

SEISMIC DYNAMIC QUALIFICATION OF SEISMIC CATEGORY 1 MECHANICAL AND  
ELECTRICAL EQUIPMENT

GSU shall complete the following requirements for seismic and dynamic qualification on the schedule noted below:

1. GSU shall, prior to startup following the second refueling outage, have completed modifications to the hydraulic control units to install an additional brace as used in the qualification testing of the hydraulic control unit as described in GSU's letter dated May 15, 1985.
2. GSU shall complete the seismic qualification of the in-vessel rack prior to its use.

ATTACHMENT 3  
To-NPF 40  
TDI DIESEL ENGINES REQUIREMENTS

GSU shall comply with the following requirements related to the TDI diesel engines.

1. Changes to the maintenance and surveillance program for the TDI diesel engines, as identified and approved by the NRC staff in Supplement 3 to the SER, shall be subject to the provisions of 10 CFR 50.59.

2. Crankshafts shall be inspected as follows:

SD 1B: During the first refueling outage, inspect the fillets and oil holes of the three most heavily loaded crankpin journals (Nos. 5, 6, and 7) with fluorescent liquid penetrant and ET as appropriate.

SD 1A and 1B: During the second and subsequent refueling outages, inspect the fillets and oil holes of two of the three most heavily loaded crankpin journals in the manner just mentioned.

SD 1A and 1B: During each major engine overhaul, inspect the fillets and oil holes of the two main bearing journals between crankpin Nos. 5, 6, and 7, using fluorescent liquid penetrant and ET as appropriate. This inspection is in addition to the crankpin inspections.

3. Cylinder blocks shall be inspected at intervals calculated using the cumulative damage index (CDI) model and using inspection methodologies described by Failure Analysis Associates, Inc. in Design Review of TDI R-4 and RV-4 Series Emergency Diesel Generator Cylinder Blocks (FaAA-84-9-11-1). Liquid penetrant inspection of cylinder liner landing area shall be performed any time cylinder liners are removed. Visual daily inspection between adjacent cylinder heads and the general block top during any period of continuous operation following automatic diesel generator startup.
4. GSU shall roll the engines over with the air start system prior to any planned starts, unless that planned start occurs within four hours of a shutdown. In addition, after engine operation, the engines shall be rolled over on air after four hours but not more than eight hours after engine shutdown and then rolled over once again approximately 24 hours after each shutdown. In the event an engine is removed from service for any reason other than the rolling over procedure prior to expiration of the eight hour or 24 hour periods noted above, that engine need not be rolled over while it is out of service. Once the engine is returned to service, GSU shall roll it over with air once at the time that it is returned to service. Any head which leaks due to a crack shall be replaced.



5. If inspection of diesel generators 1A and/or 1B should reveal cracks in the crankshaft or in the cylinder block between stud holes of adjacent cylinders, this condition shall be reported promptly to the NRC staff and the affected engine(s) shall be considered inoperable. The engine(s) shall not be restored to "operable" status until the proposed disposition and/or corrective actions have been approved by the NRC staff.
6. The following actions are required if SD 1A or SD 1B is operated in excess of 3130 KW<sup>(1)</sup>:
  - a) For indicated engine loads in the range of 3130 KW to 3200 KW for a period less than two hours<sup>(2)</sup>, no additional action shall be required.
  - b) For indicated engine loads in the range of 3130 KW to 3200 KW for a period equal to or exceeding two hours<sup>(2)</sup>, a crankshaft inspection pursuant to Item d below shall be performed at the next refueling outage.
  - c) For indicated engine loads in the range of 3200 KW to 3500 KW for a period less than 1 hour<sup>(2)</sup>, a crankshaft inspection pursuant to item d below shall be performed for the affected engine at the next refueling outage.
  - d) For indicated engine loads in the range of 3200 KW to 3500 KW for periods equal to or exceeding one hour<sup>(2)</sup>, and for engine loads exceeding 3500 KW for any period of time, (1) the engine shall be removed from service as soon as safely possible, (2) the engine shall be declared inoperable, and (3) the crankshaft shall be inspected. The crankshaft inspection shall include crankpin journal numbers 5, 6, and 7 (the most heavily loaded) and the two main journals in between using fluorescent liquid penetrant and eddy current as appropriate.
7. Operation beyond the first refueling outage is subject to NRC staff approval based on the staff's final review of the Owners Group generic findings and of the overall design review and quality revalidation program at River Bend.

- 
- (1) Momentary transients (not exceeding 5 seconds) due to changing of bus loads need not be considered as an overload.
  - (2) If there are multiple overload events within a given load range since the previous crankshaft inspection, then the time period criterion applies to the total accumulated time in that load range.

ATTACHMENT 4  
To-NPF 40  
FIRE PROTECTION PROGRAM REQUIREMENTS

GSU shall comply with the following requirements of the fire protection program:

1. GSU shall implement and maintain in effect all provisions of the approved fire protection program as described in the Final Safety Analysis Report for the facility through Amendment 21 and as approved in the SER dated May 1984 and Supplement 3 dated August 1985 subject to provisions 2 and 3 below.
2. GSU may make no change to the approved fire protection program which would significantly decrease the level of fire protection in the plant without prior approval of the Commission. To make such a change GSU must submit an application for license amendment pursuant to 10 CFR 50.90.
3. GSU may make changes to features of the approved fire protection program which do not significantly decrease the level of fire protection without prior Commission approval provided (a) such changes do not otherwise involve a change in a license condition or technical specification or result in an unreviewed safety question (see 10 CFR 50.59), and (b) such changes do not result in failure to complete the fire protection program approved by the Commission prior to license issuance. GSU shall maintain, in an auditable form, a current record of all such changes, including an analysis of the effects of the change on the fire protection program, and shall make such records available to NRC inspectors upon request. All changes to the approved program shall be reported to the Director of the Office of Nuclear Reactor Regulation, along with the FSAR revisions required by 10 CFR 50.71(e).

ATTACHMENT 5  
To-NPF 40  
EMERGENCY RESPONSE CAPABILITIES

GSU shall complete the following requirements of NUREG-0737 Supplement #1 on the schedule noted below:

1. The Safety Parameter Display System shall be installed and operational prior to March 1, 1986.
2. Actions and schedules for correcting all human engineering discrepancies (HEDs) identified in the "Detailed Control Room Design Review Summary Report dated October 31, 1984 and Supplements dated May 14, June 12, 1985, and July 31, 1985, shall be implemented in accordance with the schedule committed to by GSU in the summary report and supplements and accepted by the NRC staff in SER Section 18.1.
3. Prior to startup following the first refueling outage, GSU shall implement modifications (installation or upgrade) for those items listed below consistent with the guidance of Regulatory Guide 1.97, Revision 2 unless prior approval of an alternate design of these items is granted by the NRC staff. These items as listed in GSU's letter of June 24, 1985 are:
  - a) neutron flux;
  - b) coolant level in the reactor;
  - c) suppression pool water level;
  - d) drywell atmosphere temperature;
  - e) primary system safety relief valve position;
  - f) standby liquid control system storage tank level;
  - g) emergency ventilation damper position; and
  - h) airborne radiohalogens and particulates.

Please send copies of the progress report to Stephen M. Stern, the staff's Project Manager for the River Bend Application, as well as the NRC's Region IV and the NRC's Resident Inspector at River Bend.

Please contact Mr. Stern for any clarification of this request.

Sincerely,

**Original signed by:**

Walter R. Butler, Chief  
Licensing Branch No. 2  
Division of Licensing

Enclosure:  
As stated

cc: w/enclosure  
See next page

DISTRIBUTION

Docket File

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