

## LICENSEE EVENT REPORT (LER)

FACILITY NAME (1) Nine Mile Point Unit I										DOCKET NUMBER (2) 0 5 0 0 0 2 2 0										PAGE (3) 1 OF 2																							
TITLE (4) Automatic Actuation of Reactor Building Emergency Ventilation System																																											
EVENT DATE (5)						LER NUMBER (6)						REPORT DATE (7)						OTHER FACILITIES INVOLVED (8)																									
MONTH		DAY		YEAR		YEAR		SEQUENTIAL NUMBER		REVISION NUMBER		MONTH		DAY		YEAR		FACILITY NAMES																									
10		30		85		85		020		001		11		27		85		DOCKET NUMBER(S) 0 5 0 0 0																									
10		30		85		85		020		001		11		27		85		0 5 0 0 0																									
OPERATING MODE (9)		THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR 5: (Check one or more of the following) (11)																																									
N		20.402(b)										20.405(c)										X 50.73(a)(2)(iv)										73.71(b)											
POWER LEVEL (10)		0 9 9										20.405(a)(1)(i)										50.38(a)(1)										50.73(a)(2)(v)										73.71(c)	
		20.405(a)(1)(ii)										50.38(a)(2)										50.73(a)(2)(vi)										OTHER (Specify in Abstract below and in Text, NRC Form 366A)											
		20.405(a)(1)(iii)										50.73(a)(2)(i)										50.73(a)(2)(vii)(A)																					
		20.405(a)(1)(iv)										50.73(a)(2)(ii)										50.73(a)(2)(vii)(B)																					
		20.405(a)(1)(v)										50.73(a)(2)(iii)										50.73(a)(2)(ix)																					
LICENSEE CONTACT FOR THIS LER (12)																																											
NAME Robert G. Randall, Supervisor Technical Support																TELEPHONE NUMBER AREA CODE 3 1 5 3 4 9 - 2 4 4 5																											
COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)																																											
CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPDOS		CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPDOS																																	
SUPPLEMENTAL REPORT EXPECTED (14)																EXPECTED SUBMISSION DATE (15)		MONTH	DAY	YEAR																							
YES (If yes, complete EXPECTED SUBMISSION DATE)																X NO																											

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

## ABSTRACT

On October 30, 1985, with the reactor operating at 99% power, a momentary loss of output power from uninterruptible power supply motor-generator set #162 was experienced. This event caused the Reactor Building Ventilation System to shift to its emergency mode of operation when power was lost to its respective radiation monitoring detectors. The Control Room Emergency Ventilation System received an actuation signal as designed, but did not shift into the emergency ventilation mode as the system was administratively inoperative at the time and emergency fan control switches were in the OFF position. In addition, power was lost to Reactor Protection System Bus #11 and the Feedwater Control System transferred to its alternate power supply.

The event was initiated by a generator undervoltage relay sensing a spurious undervoltage which caused the opening of the output breaker. Monitoring of the Reactor Protection System Bus #11 has revealed considerable noise which may have contributed to the incident.

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## LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

U.S. NUCLEAR REGULATORY COMMISSION

APPROVED OMB NO. 3150-0104

EXPIRES 8/31/85

FACILITY NAME (1)  Nine Mile Point Unit I	DOCKET NUMBER (2)  0 5 0 0 0 2 2 0 8 5 — 0 2 0 — 0 0	LER NUMBER (5)			PAGE (3)		
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER			
		8 5	0 2 0	0 0	0 2	OF	0 2

TEXT (If more space is required, use additional NRC Form 306A's) (17)

TEXT

On October 30, 1985, with the reactor operating at 99% of rated power, the AC generator output breaker of the uninterruptible power supply motor-generator set #162 was tripped open by the actuation of its undervoltage relay 27-2. The output breaker remained open for approximately thirty seconds before the plant operators reclosed the breaker. The loss of power to the radiation monitoring detectors in the Reactor Building Ventilation System caused it to shift to the emergency ventilation mode as designed. The Control Room Emergency Ventilation System also received an actuation signal, but did not start as both emergency make up air fan control switches were in the OFF position, as permitted under Limiting Condition for Operation 3.4.5.e of the Nine Mile Point Nuclear Station Unit I Technical Specifications. This momentary loss of power also caused Reactor Protection System Bus #11 to deenergize (producing an automatic isolation of the Reactor Water Cleanup System) and the Feedwater Control System channel #11 to transfer to its alternate power supply.

Following the restoration of output power from motor-generator set #162, plant operators reset Reactor Protection System Bus #11 and transferred the Feedwater Control System to its normal power supply. The Reactor Water Cleanup System was placed back into service and the Reactor Building Ventilation System was restored to its normal lineup. There were no other engineered safety feature actuations and plant operating conditions were returned to normal without incident.

ASSESSMENT OF SAFETY CONSEQUENCES

There are no potential safety consequences resulting from this event as the plant engineered safety features operated as designed. The actuation of the Reactor Building Emergency Ventilation mode, isolation of the Reactor Water Cleanup System, and initiation of a half scram signal are all conservative actions. All engineered safety features that required Reactor Protection System power (energized to function), such as the Automatic Depressurization System, Liquid Poison initiation, and Emergency Cooling System Isolation, remained capable of performing their designed functions through the remaining operating Reactor Protection System motor-generator set. The Control Room Emergency Ventilation System was administratively inoperable at the time of the event to permit roof repairs near the ventilation intake, and was capable of performing its designed function should the need have arisen.

CORRECTIVE ACTION

The primary and alternate undervoltage relay trip and reset setpoints of motor-generator set #162 were tested by Meter and Test Group personnel and found to be satisfactory. A chart recorder was installed to trend the regulated voltage. To date, no unusual voltage output disturbances have occurred. However, there is considerable noise on the system which may have contributed to the incident.

## NIAGARA MOHAWK POWER CORPORATION

NIAGARA  MOHAWK300 ERIE BOULEVARD, WEST  
SYRACUSE, N. Y. 13202

November 27, 1985

United States Nuclear Regulatory Commission  
Document Control Desk  
Washington, DC 20555RE: Docket 50-220  
LER 85-20

Gentlemen:

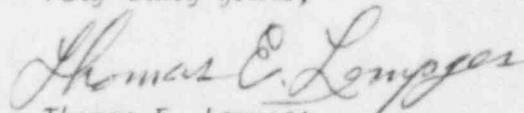
In accordance with 10 CFR 50.73, we hereby submit the following  
Licensee Event Report:

LER 85-20      Which is being submitted in accordance with 10 CFR 50.73  
(a)(2)(iv), "Any event or condition that resulted in manual  
or automatic actuation of any Engineered Safety Feature  
(ESF) including the Reactor Protection System (RPS). However,  
actuation of an ESF, including the RPS, that resulted from  
and was part of the preplanned sequence during testing or  
reactor operation need not be reported."

A 10 CFR 50.72 report was made at 1303 on October 30, 1985.

This report was completed in the format designated in NUREG-1022 dated  
September 1983.

Very truly yours,

Thomas E. Lempges  
Vice President  
Nuclear Generation

TEL/tg

cc: Dr. Thomas E. Murley  
Regional AdministratorTEL  
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