

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1) Duane Arnold Energy Center										DOCKET NUMBER (2) 0 5 0 0 0 3 3 1 1										PAGE (3) 1 OF 0 2																													
TITLE (4) Secondary Containment Violations																																																	
EVENT DATE (5)										LER NUMBER (6)										REPORT DATE (7)										OTHER FACILITIES INVOLVED (8)																			
MONTH			DAY			YEAR			YEAR			SEQUENTIAL NUMBER			REVISION NUMBER			MONTH			DAY			YEAR			FACILITY NAMES										DOCKET NUMBER(S)												
0 9			0 6			8 5			8 5			0 3			8			0 1			0 1			3 0			8 6			None										0 5 0 0 0									
0 9			0 6			8 5			8 5			0 3			8			0 1			0 1			3 0			8 6													0 5 0 0 0									
OPERATING MODE (9) N										THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR § (Check one or more of the following) (11)																																							
POWER LEVEL (10) 0 9 0										20.402(b)										20.406(a)										50.73(a)(2)(iv)										73.71(b)									
										20.406(a)(1)(i)										50.36(a)(1)										50.73(a)(2)(v)										73.71(a)									
										20.406(a)(1)(ii)										50.36(a)(2)										50.73(a)(2)(vi)										OTHER (Specify in Abstract below and in Text, NRC Form 305A)									
										20.406(a)(1)(iii)										50.73(a)(2)(i)										50.73(a)(2)(vii)(A)																			
										20.406(a)(1)(iv)										50.73(a)(2)(ii)										50.73(a)(2)(vii)(B)																			
										20.406(a)(1)(v)										50.73(a)(2)(iii)										50.73(a)(2)(x)																			
LICENSEE CONTACT FOR THIS LER (12)																																																	
NAME Kenneth S. Putnam, Technical Support Engineer																				TELEPHONE NUMBER																													
																				AREA CODE 3 1 9 8 5 1 - 7 4 5 6																													
COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)																																																	
CAUSE		SYSTEM		COMPONENT		MANUFACTURER		REPORTABLE TO NRC				CAUSE		SYSTEM		COMPONENT		MANUFACTURER		REPORTABLE TO NRC																													
E		NIG		SOL				Yes																																									
E		NIG		SOL				Yes																																									
SUPPLEMENTAL REPORT EXPECTED (14)																																																	
YES (If yes, complete EXPECTED SUBMISSION DATE)																				X NO										EXPECTED SUBMISSION DATE (15)																			
																														MONTH DAY YEAR																			
ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)																																																	
<p>On 09/06/85 at approximately 1400 hours, while in normal power operation, both doors of an airlock between the Reactor and Turbine Buildings were inadvertently opened simultaneously, resulting in a momentary violation of secondary containment. The failure occurred due to a personnel error in wiring of the door interlocks the previous day. Testing to ensure operability was not required due to the classification of the system as "non-safety related".</p> <p>On 09/12/85 at approximately 1356 hours, both doors of the Standby Gas Treatment Room were opened simultaneously, resulting in a momentary violation of secondary containment as a result of a loose wire on a door interlock.</p> <p>Technical Specification 3.7.C.1 requires secondary containment integrity during power operation. These events, therefore, although momentary, constitute a condition prohibited by our Technical Specifications and are being reported accordingly. There were no effects upon the health and safety of the public.</p> <p>Both events were the result of an inoperative solenoid in the interlock mechanism. In each event the door interlocks were promptly repaired. Long-term corrective actions being undertaken include upgrading of the quality level designation of secondary containment door interlocks and installation of improved quality interlocking mechanisms.</p>																																																	

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LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

U.S. NUCLEAR REGULATORY COMMISSION

APPROVED OMB NO. 3150-0104

EXPIRES: 8/31/85

FACILITY NAME (1)	DOCKET NUMBER (2)	LER NUMBER (5)			PAGE (3)		
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER			
Duane Arnold Energy Center	05000331	85	038	01	02	of	02

TEXT (If more space is required, use additional NRC Form 386A's) (17)

On 09/06/85 at approximately 1400 hours, both doors of an airlock between the Reactor and Turbine Buildings were inadvertently opened simultaneously, resulting in a momentary violation of secondary containment. Security personnel were posted to prevent further violations until the interlock mechanism was repaired the same day.

As part of corrective actions in an attempt to correct previous secondary containment violations, tests were being conducted to determine the effectiveness of a new interlock design. While reconnecting the old interlock device, workers inadvertently crossed the solenoid wires which disabled the interlock.

Administrative controls on plant maintenance activities failed to require adequate post-maintenance testing to detect the wiring error largely due to the classification of the secondary containment door interlocks as Quality Level IV (non-safety related) equipment. In response to NRC Inspection Report 85-029, Duane Arnold Energy Center has begun treating secondary containment door interlocks as Quality Level II equipment.

Quality Level II applies to those structures, systems, components, and related activities that are not safety-related but are governed by regulation and for which certain quality assurance controls are established. A Quality Level II designation will ensure that appropriate Post-Maintenance testing and other Quality Controls are considered when dealing with secondary containment door interlocks. In addition, a task force review is being initiated to identify systems currently designated as non-safety related (Quality Level IV) that are necessary to support safety-related functions or regulatory requirements. The task force will conduct a review of plant systems. Any systems identified that merit a higher level of attention to quality than is normally associated with Quality Level IV equipment will be upgraded to Quality Level II.

On 09/12/85 at approximately 1356 hours, both doors of the Standby Gas Treatment Room were opened simultaneously, resulting in a momentary violation of secondary containment. Both doors were verified closed and a loose wire was reattached to the solenoid, thus returning it's secondary containment function.

The second event was also the result of an inoperative solenoid in the interlock mechanism. As corrective action, the solenoid was repaired the same day of the failure. In addition to the actions noted above, new interlock mechanisms, expected to be more reliable, are being installed.

Technical Specification 3.7.C.1 required secondary containment integrity during power operation. These events, therefore, constituted a condition which was prohibited by our Technical Specifications and are being reported accordingly. A Technical Specification change has been issued which provides an action statement for reestablishing secondary containment integrity. The Standby Gas Treatment System (EIS Code BH) was operable during each event but was not required to be operating to provide an elevated filtered effluent pathway. (See LER's 84-30, 84-34, 84-38, 84-45, 85-03, 85-16, and 85-30 for prior similar events.) There was no effect upon the health and safety of the public.

Iowa Electric Light and Power Company

January 30, 1986

DAEC-86-0067

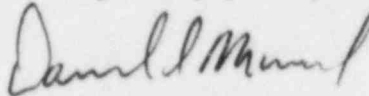
U. S. Nuclear Regulatory Commission
Document Control Desk
Washington, D. C. 20555

Subject: Duane Arnold Energy Center
Docket No. 50-331
Op. License DPR-49
Licensee Event Report No. 85-038, Rev. 1

Gentlemen:

In accordance with 10 CFR 50.73 please find attached a copy of the
subject Licensee Event Report.

Very truly yours,



Daniel L. Mineck
Plant Superintendent - Nuclear
Duane Arnold Energy Center

DLM/KSP/kp

Attachment - LER 85-38, Rev. 1

cc: Mr. James G. Keppler
Regional Administrator
Region III
U. S. Nuclear Regulatory Commission
799 Roosevelt Road
Glen Ellyn, IL 60137

NRC Resident Inspector - DAEC

File A-118a

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