LICENSEE EVENT REPORT (LER)

Facility Name (1)	Docket Number (2)	Page (3)
QUAD-CITIES, NUCLEAR POWER STATION, UNIT 1	0 5 0 0 0 2 5 4	1 of 0 3

Title (4) Leak Rate From All Valves and Penetrations in Excess of Technical Specification Limit

Event Date (5)			LER Number (6)				Report Date (7)			Other Facilities Involved (8)	
Month	Day	Year	Year	///	Sequential Number	Revision Number	Month	Day	Year	Facility Names	Docket Number(s)
											0 5 0 0 0
0 1	0 6	8 6	8 6	---	0 0 1	0 0	0 1	3 1	8 6		0 5 0 0 0

OPERATING MODE (9)		THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10CFR (Check one or more of the following) (11)			
	1	___ 20.402(b)	___ 20.405(c)	___ 50.73(a)(2)(iv)	___ 73.71(b)
POWER LEVEL (10)	0 0 0	___ 20.405(a)(1)(i)	___ 50.36(c)(1)	___ 50.73(a)(2)(v)	___ 73.71(c)
		___ 20.405(a)(1)(ii)	___ 50.36(c)(2)	___ 50.73(a)(2)(vii)	___ Other (Specify in Abstract below and in Text)
		___ 20.405(a)(1)(iii)	___ 50.73(a)(2)(i)	___ 50.73(a)(2)(viii)(A)	
		___ 20.405(a)(1)(iv)	X 50.73(a)(2)(ii)	___ 50.73(a)(2)(viii)(B)	
		___ 20.405(a)(1)(v)	___ 50.73(a)(2)(iii)	___ 50.73(a)(2)(x)	

LICENSEE CONTACT FOR THIS LER (12)

Name	TELEPHONE NUMBER
	AREA CODE
Nicos P. Digrindakis, Technical Staff Engineer Ext. 2158	3 0 9 6 5 4 - 2 2 4 1

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS		CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS	
X	J M			Y							

SUPPLEMENTAL REPORT EXPECTED (14)		Expected Submission Date (15)	Month	Day	Year
X Yes (If yes, complete EXPECTED SUBMISSION DATE)	___ NO		0 5	0 1	8 6

ABSTRACT (Limit to 1400 spaces, i.e. approximately fifteen single-space typewritten lines) (16)

On January 6, 1986, Quad Cities Unit One was shutdown for refueling. While performing refueling outage Local Leak Rate Testing, the measured combined leakage rate for all Penetrations and Valves, except Main Steam Isolation Valves, was found to leak in excess of 293.75 SCFH (Q60 L_a). A supplemental report will be submitted when all leak rate testing and repairs have been completed.

This report is being submitted to you in accordance with the requirements of 10 CFR 50.73(a)(2)(ii), which requires reporting of any event or condition that resulted in the condition of the nuclear power plant, including its principle safety barrier, being seriously degraded.

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PLANT AND SYSTEM IDENTIFICATION:

General Electric - Boiling Water Reactor - 2511 MWt rated core thermal power. Energy Industry Identification System (EIIS) codes are identified in the text as [XX].

IDENTIFICATION OF OCCURRENCE:

The leak rate from all valves and penetrations (excluding MSIVs) is in excess of the Technical Specification limit of $0.60L_a$ or 293.75 SCFH.

Discovery Date: 1-6-86

Report Date: 1-31-86

This report was initiated by Deviation Report D-4-1-86-5

CONDITIONS PRIOR TO OCCURRENCE:

SHUTDOWN Mode(1) - Rx Power 0% - Unit Load 0 MWe

SHUTDOWN Mode(1) - In this position, a reactor scram is initiated power to the control rod drives is removed and the reactor protection trip systems have been deenergized for 10 seconds prior to permissive for manual reset.

DESCRIPTION OF OCCURRENCE:

At 1800 hours, on January 6, 1986, Unit One was shutdown for End of Cycle Eight Refueling and Maintenance Outage. While performing refueling outage Local Leak Rate Testing, the measured combined leakage rate for all Penetrations and Valves, except Main Steam Isolation Valves, was found to leak in excess of 293.75 SCFH ($0.60\ L_a$).

This report is being submitted to comply with the requirements of 10 CFR 50.73(a)(2)(ii), which requires reporting of any event or condition that resulted in the condition of the nuclear power plant, including its principle safety barrier, being seriously degraded.

APPARENT CAUSE OF OCCURRENCE:

The probable cause of the excessive valve and penetration leakage that caused the total measured leakage to exceed the allowable limit (Technical Specification 4.7.A.2) is not known at this time.

ANALYSIS OF OCCURRENCE:

Local Leak Rate Testing is a conservative way of measuring containment leakage. During accident conditions actual leakage would be less than that determined by testing due to some lines being pressurized with water, and non-PCI valves performing as primary containment isolation valves. Also, Secondary Containment and Standby Gas Treatment system provide additional protection for the environs from radioactive releases.

CORRECTIVE ACTION:

No corrective action has been taken as of this date. A supplemental report will be submitted listing the necessary repairs and corrective actions taken to reduce the total leakage below the limits.

FAILURE DATE:

The last occurrence of total leakage in excess of Technical Specification limit occurred on March 18, 1985. It is documented in Licensee Event Report 85-007.

Commonwealth Edison
Quad Cities Nuclear Power Station
22710 206 Avenue North
Cordova, Illinois 61242
Telephone 309/654-2241

NJK-86-26

January 31, 1986

U.S. Nuclear Regulatory Commission
Document Control Desk
Washington, DC 20555

Reference: Quad-Cities Nuclear Power Station
Docket Number 50-254, DPR-29, Unit One

Enclosed please find Licensee Event Report (LER) 86-01, Revision 00, for Quad-Cities Nuclear Power Station.

This report is submitted to you in accordance with the requirements of the Code of Federal Regulations, Title 10, Part 50.73(a)(2)(ii), which requires reporting of any event or condition that resulted in the condition of the nuclear power plant, including its principle safety barrier, being seriously degraded.

Respectfully,

COMMONWEALTH EDISON COMPANY
QUAD-CITIES NUCLEAR POWER STATION

N. J. Kalivianakis
Station Manager

NJK/MSK/dak

Enclosure

cc: J. Wojnarowski
A. Madison
INPO Records Center
NRC Region III

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