

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1)
EDWIN I. HATCH, UNIT 1

DOCKET NUMBER (2)

0 5 0 0 0 3 2 1 1

PAGE (3)

1 OF 012

TITLE (4)

PUMP CASING CRACK CAUSES ACTUATION OF ENGINEERED SAFETY FEATURES (ESF)

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)								
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES		DOCKET NUMBER(S)						
1	0	2	9	8	5	8	5	0	3	8	0	5	0	0	0		
1	0	2	9	8	5	8	5	0	3	8	0	5	0	0	0		

THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR § (Check one or more of the following) (11)

OPERATING MODE (9)	20.402(b)	20.406(c)	50.73(a)(2)(iv)	73.71(b)
1	20.406(a)(1)(i)	50.36(e)(1)	X	73.71(c)
POWER LEVEL (10)	20.406(a)(1)(ii)	50.36(e)(2)		OTHER (Specify in Abstract below and in Text, NRC Form 366A)
1	20.406(a)(1)(iii)	50.73(a)(2)(i)		
0	20.406(a)(1)(iv)	50.73(a)(2)(ii)		
0	20.406(a)(1)(v)	50.73(a)(2)(iii)		

LICENSEE CONTACT FOR THIS LER (12)

NAME
Steven B. Tipps, Superintendent of Regulatory Compliance

TELEPHONE NUMBER

AREA CODE

917 12 316 17 + 17 181 511

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRC	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRC
X	WJ	PI	101715	N					

SUPPLEMENTAL REPORT EXPECTED (14)

YES (If yes, complete EXPECTED SUBMISSION DATE)	NO	EXPECTED SUBMISSION DATE (15)	MONTH	DAY	YEAR
	X				

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

On 10/29/85 at approximately 1729 CST, with the reactor mode switch in the run position and reactor power at 2436 MWt (100% power), maintenance personnel noted water on the floor of the 158 feet elevation of the Unit 1 Reactor Building. They immediately reported the leak to the main control room operator. At that time, SBT system "B" auto-started, and a group 2 isolation (including isolation of the H₂O₂ analyzers and Fission Product monitors) was received due to the reactor building radiation monitors reading approximately 20 millirem per hour. A Radiological Event was declared.

An investigation of the leak revealed that the radwaste's "Clean-up sludge discharge mix pump" (1G11-C028) (EIS identifier: WJ) had a casing crack. The pump was isolated and clean-up of the leak began.

On 10/29/85 at approximately 2330 CST, the Radiological Event was terminated and SBT system "B" was secured. On 10/30/85 at approximately 0015 CST, the group 2 isolation was reset and the H₂O₂ analyzers and Fission Product monitors were returned to service.

The SBT system and the group 2 isolation operated as required. Therefore, no actual or potential adverse safety consequences or implications resulted from this event. This event had no impact on any other Unit 1 system or on Unit 2. The health and safety of the public were not affected by this event. There are no known previous similar events.

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LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

APPROVED OMB NO. 3150-0104

EXPIRES: 8/31/85

FACILITY NAME (1)	DOCKET NUMBER (2)	LER NUMBER (6)			PAGE (3)		
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER			
EDWIN I. HATCH, UNIT 1	0 5 0 0 0 3 2 1 8 5	—	0 3 8	—	0 0 0	2	OF 0 2

TEXT (If more space is required, use additional NRC Form 366A's) (17)

This 30 day LER is required by 10CFR 50.73(a)(2)(iv) due to the actuation of the following Engineered Safety Features (ESFs): the Standby Gas Treatment system (SBGT) (EIIS identifier: BH) and a group 2 Primary Containment Isolation Valve (PCIV) isolation (EIIS identifier: JM).

On 10/29/85 at approximately 1729 CST, with the reactor mode switch in the run position and reactor power at 2436 MWt (100% power), maintenance personnel noted water on the floor of the 158 feet elevation of the Unit 1 Reactor Building. Maintenance personnel immediately reported the leak to the main control room operator. At that time, SBGT system "B" auto-started, and a group 2 isolation was received due to the reactor building radiation monitors reading approximately 20 millirem per hour. A Radiological Event was declared.

Upon receipt of the group 2 isolation, both Hydrogen-Oxygen (H₂O₂) Analyzers (EIIS identifier: BB) and the Fission Product Monitors (EIIS identifiers: IK) isolated. Thus, the plant could not meet the requirements of Tech. Specs. sections 3.7.A.6.c and 3.6.G.3.c, respectively. Plant operation was then continued per the 12 hour Limiting Condition for Operation (LCO) specified by Tech. Specs. sections 3.7.A.8 and 3.6.G.3.a and c.

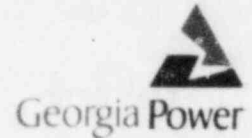
An investigation of the leak revealed that the radwaste's "Clean-up sludge discharge mix pump" (IG11-C028) (EIIS identifier: WJ) had a casing crack. The pump was isolated and clean-up of the leak began. Note: The pump failure is not a reportable item by itself. However, the results of that failure (i.e., high radiation resulting in the ESF actuations) is reportable.

On 10/29/85 at approximately 2330 CST, the Radiological Event was terminated and SBGT system "B" was secured. On 10/30/85 at approximately 0015 CST, the group 2 isolation was reset and the H₂O₂ analyzers and Fission Product monitors were returned to service.

The failed pump casing was replace (via a design change) with a stainless steel casing.

The SBGT system and the group 2 isolation operated as required. Therefore, no actual or potential adverse safety consequences or implications resulted from this event. This event had no impact on any other Unit 1 system or on Unit 2. The health and safety of the public were not affected by this event. There are no known previous similar events.

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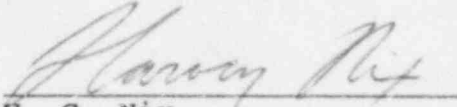
Edwin I. Hatch Nuclear Plant

November 26, 1985
LR-MGR-123-1185

PLANT E. I. HATCH
Licensee Event Report
Docket No. 50-321

United States Nuclear Regulatory Commission
Document Control Desk
Washington, D. C. 20555

Attached is Licensee Event Report No. 50-321/1985-038. This report is required by 10 CFR 50.73(a)(2)(iv).


H. C. Nix
General Manager

HCN/SBT/lmw

xc: NRC Region II
Document Control
Letter File (2)

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