

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1) Waterford 3 Steam Electric Station										DOCKET NUMBER (2) 0 5 0 0 0 3 8 2 1 OF 0 3										PAGE (3) 1 OF 0 3						
TITLE (4) Reactor Trip As A Result of Deluge System																										
EVENT DATE (5)			LER NUMBER (6)				REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)																
MONTH	DAY	YEAR	YEAR	SEQUENTIA NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES				DOCKET NUMBER(S)													
									N/A				0 5 0 0 0													
1	0	2	8	8	5	8	5	0	4	7	0	0	1	1	2	6	8	5	N/A				0 5 0 0 0			
OPERATING MODE (U)		THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR 5. (Check one or more of the following) (11)																								
1		20.402(b)				20.405(c)				<input checked="" type="checkbox"/> 50.73(a)(2)(iv)				73.71(b)												
POWER LEVEL (10)		20.405(a)(1)(i)				50.36(e)(1)				50.73(a)(2)(v)				73.71(c)												
1 0 0		20.405(a)(1)(ii)				50.36(c)(2)				50.73(a)(2)(vi)				OTHER (Specify in Abstract below and in Text, NRC Form 766A)												
		20.405(a)(1)(iii)				50.73(a)(2)(i)				50.73(a)(2)(viii)(A)																
		20.405(a)(1)(iv)				50.73(a)(2)(ii)				50.73(a)(2)(viii)(B)																
		20.405(a)(1)(v)				50.73(a)(2)(iii)				50.73(a)(2)(ix)																
LICENSEE CONTACT FOR THIS LER (12)																										
NAME L. Myers, Operations Superintendent										TELEPHONE NUMBER AREA CODE 5 0 4 4 6 4 - 3 1 1 8																
COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)																										
CAUSE	SYSTEM	COMPONENT	MANUFAC- Turer	REPORTABLE TO NPDOS		CAUSE	SYSTEM	COMPONENT	MANUFAC- Turer	REPORTABLE TO NPDOS																
SUPPLEMENTAL REPORT EXPECTED (14)										EXPECTED SUBMISSION DATE (15)		MONTH	DAY	YEAR												
<input type="checkbox"/> YES (If yes, complete EXPECTED SUBMISSION DATE)										<input checked="" type="checkbox"/> NO																

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

ABSTRACT

At 1920 hours on October 28, 1985 Waterford 3 Steam Electric Station was at 100% reactor power when an uncomplicated reactor trip occurred due to an auxiliary trip from the Core Protection Calculator (EIIIS System Code JD) on high Pressurizer pressure. Plant conditions were subsequently stabilized in mode 3 (hot standby).

The trip occurred when a steam leak from the suction flange on the B Main Feedwater Pump (EIIIS System Code SJ) actuated the deluge system (EIIIS System Code KP) located directly above the feedwater pump. The water from the deluge system sprayed the feedwater pump control cabinet causing a trip of the B Main Feedwater Pump. Operations personnel could not reduce reactor power fast enough, therefore, Pressurizer pressure increased above the range of the Core Protection Calculator.

Maintenance personnel repaired the flange leak and dried the components within the feedwater pump control cabinet.

8512060003 851126
PDR ADOCK 05000382
S PDRIE 22
1/1

LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

APPROVED OMB NO. 3150-0104

EXPIRES: 8/31/85

FACILITY NAME (1)	DOCKET NUMBER (2)	LER NUMBER (6)			PAGE (3)		
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER			
Waterford 3 Steam Electric Station	0 5 0 0 0 3 8 2	8 5	— 0 4 7	— 0 0	0 2	OF	0 3

TEXT (If more space is required, use additional NRC Form 366A's) (17)

NARRATIVE

At 1920 hours on October 28, 1985 Waterford 3 Steam Electric Station was at 100% reactor power when an uncomplicated reactor trip occurred due to an auxiliary trip from the Core Protection Calculator (EIIIS System Code JD) on high Pressurizer pressure. Upon receiving the trip Operations personnel immediately entered procedures OP-902-000, "Emergency Entry Procedure" and OP-902-001, "Uncomplicated Reactor Trip Recovery". Plant conditions were subsequently stabilized in mode 3 (hot standby).

Just prior to the trip, Maintenance personnel were attempting to repair a steam leak on the B Main Feedwater Pump (EIIIS System Code SJ) suction piping. The steam was emanating from the pump suction flange. Before the leak could be completely repaired, the steam actuated the feedwater pump deluge system (EIIIS System Code KP) located directly above the feedwater pump control cabinet causing a trip of the B Main Feedwater Pump. Operations personnel immediately attempted to reduce load to within the capacity of the operating feedwater pump. However, reactor power could not be decreased fast enough, and therefore, Pressurizer pressure increased above the range of the Core Protection Calculator. (An auxiliary trip is generated whenever Pressurizer pressure falls out of the range of the Core Protection Calculator ie., less than or equal to 1845 psia and greater than 2355 psia.)

SAFETY CONSEQUENCES AND IMPLICATIONS

The above event resulted in an uncomplicated reactor trip in which no primary design parameters were exceeded. Since the Control Element Assemblies and Reactor Protective System functioned as designed, the subject event in no way placed Waterford 3 in a degraded safety condition.

LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

APPROVED OMB NO. 3150-0104

EXPIRES: 8/31/85

FACILITY NAME (1)	DOCKET NUMBER (2)	LER NUMBER (6)			PAGE (3)		
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER			
Waterford 3 Steam Electric Station	0 5 0 0 0 3 8 2 8 5 — 0 4 7 — 0 0				0 3	OF	0 3

TEXT (If more space is required, use additional NRC Form 366A's) (17)

CORRECTIVE ACTION

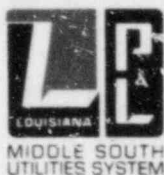
Maintenance personnel repaired the flange leak and dried the components within the feedwater pump control cabinet. The feedwater pump has been running satisfactorily since Waterford 3 returned to power.

SIMILAR EVENTS

NONE

PLANT CONTACT

L. Myers, Operations Superintendent, 504/464-3118



LOUISIANA
POWER & LIGHT

142 DELARONDE STREET
P. O. BOX 6008 • NEW ORLEANS, LOUISIANA 70174 • (504) 366-2345

November 26, 1985

W3P85-3286
A4.05
QA

Director, Office of Nuclear Reactor Regulation
ATTENTION: Document Control Desk
U.S. Nuclear Regulatory Commission
Washington, D.C. 20555

Subject: Waterford 3 SES
Docket No. 50-382
License No. NPF-38
Reporting of Licensee Event Report

Dear Sirs:

Attached is Licensee Event Report Number LER-85-047-00 for the Waterford 3 Steam Electric Station. This Licensee Event Report is submitted per 10CFR50.73(a)(2)(iv).

Very truly yours,

K.W. Cook
Nuclear Support & Licensing Manager

KWC:GEW:sms

Attachment

cc: R.D. Martin, G.W. Knighton, J.H. Wilson, NRC Resident Inspectors
Office, INPO Records Center (J.T. Wheelock), B.W. Churchill,
W.M. Stevenson

IE22
11