

LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

APPROVED OMB NO. 3150-0104

EXPIRES 8/31/85

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TEXT (If more space is required, use additional NRC Form 365A's) (17)

On 01/01/86 at approximately 1500 with the unit in hot shutdown, a jumper was found in main control room panel 1H13-P623. It was determined that the jumper was not being used as part of an ongoing test and there was no apparent explanation for it being installed. The jumper was installed across the contacts on a Reactor Water Cleanup (RWCU) isolation relay and would have inhibited a Division I RWCU isolation. The jumper was removed at approximately 1600 without incident.

As a result of this incident, the Operations department conducted an inspection of all control room panels. The inspection and subsequent research by the Maintenance department identified one lifted lead not accounted for by plant records. This lifted lead was found in control room panel 1H13-P951 on 01/04/86 and would have prevented the automatic closure of Containment Building HVAC isolation valve 1HVR-AOV165 on a high radiation signal. It was immediately confirmed with Operations that the isolation valve was in the closed position, as required by Technical Specifications. The lead was relanded on 01/04/86 and applicable portions of the surveillance test procedure (STP) were successfully performed to test operability of the system.

Panel entry logs, maintenance work request records, clearance orders, the temporary alteration log and surveillance test procedure records were reviewed to determine what work had been performed in panels 1H13-P623 and 1H13-P951. The last identified work in panel 1H13-P623 was the performance of STP-207-4218 on

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12/27/85. The records in the procedure clearly identified, with verification, that the jumper was removed. The Assistant Plant Manager of Maintenance confirmed by discussion with the responsible instrument and controls technician that the jumper was removed during the restoration of the test. The last identified work involving the lifted lead in panel 1H13-P951 was the performance of STP-257-4201 on 11/12/85 which confirmed the lead to be landed. The investigation of the two instances resulted in no conclusive evidence as to why the jumper was installed in panel 1H13-P623 or the lead was lifted in panel 1H13-P951.

To determine whether these problems were isolated or indications of a generic problem, the Maintenance and Quality Assurance departments conducted independent inspections of randomly selected panels. Both inspections concluded that all circuit alterations were properly documented and no unidentified lifted leads or jumpers were noted.

In an effort to prevent recurrence, corrective actions were taken to establish additional administrative controls and practices for installing jumpers and lifting leads. On 01/07/86 a control room cabinet access and work monitoring program was implemented. Control room panel locks were also changed to limit access to only authorized individuals.

On 01/07/86 and 01/08/86, meetings were held by the Assistant Plant Manager of Maintenance with instrument and controls foremen and technicians to discuss the license violation and the proceduralized jumper and lifted lead program.

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U.S. NUCLEAR REGULATORY COMMISSION

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The requirements of GMP-0042, "Control Circuit Testing," were revised to include the use of a serialized tagging system for providing accountability for jumpers and lifted leads. Training was conducted by each foreman for his repairmen/technicians on the revised requirements of GMP-0042 and the events which led to it. Additionally GMP-0042 is included in the required reading list for new electrical repairmen and instrument and controls technicians.

A review of the system design confirmed that automatic isolation of the RWCU system was not disabled due to RWCU inboard isolation valves 1G33-F028, 1G33-F053, 1G33-F001 and 1G33-F040 remaining operable with the unauthorized jumper installed. Also a review of the Containment Building HVAC system design confirmed that an automatic system isolation on a high radiation alarm was not disabled due to a redundant isolation valve 1HVR*AOV123 remaining operable with lead lifted in panel 1H13-P951. Both systems were restored to meet the approved design configuration. For the reasons described above it is concluded that redundant safety features were still available to perform their intended safety function during this event.



GULF STATES UTILITIES COMPANY

RIVER BEND STATION POST OFFICE BOX 220 ST. FRANCISVILLE, LOUISIANA 70775
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U.S. Nuclear Regulatory Commission
Document Control Desk
Washington, D. C. 20555

Dear Sir:

River Bend Station - Unit 1
Docket No. 50-458

Please find enclosed Licensee Event Report No. 86-003 for River Bend Station - Unit 1. This report is submitted pursuant to 10CFR50.73.

Sincerely,

Eddie R. Grant

for J. E. Booker
Manager-Engineering,
Nuclear Fuels & Licensing
River Bend Nuclear Group

JEB
JEB/TFP/DRG/BEH/ebm

cc: U. S. Nuclear Regulatory Commission
611 Ryan Plaza, Suite 1000
Arlington, TX 76011

INPO Records Center
1100 Circle 75 Parkway
Atlanta, GA 30339-3064

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