



SEABROOK STATION  
Engineering Office

Public Service of New Hampshire

New Hampshire Yankee Division

February 5, 1986

SBN- 940

T.F. B7.1.3

United States Nuclear Regulatory Commission  
Washington, DC 20555

Attention: Mr. Vincent S. Noonan, Project Director  
PWR Project Directorate No. 5

Reference: (a) Construction Permits CPPR-135 and CPPR-136, Docket  
Nos. 50-443 and 50-444  
(b) Westinghouse Electric Corporation letter (NS-EPR-2895),  
dated March 14, 1984, "Special Rod Tests," E. P. Rahe, Jr.  
to C. H. Berlinger

Subject: Seabrook Startup Test Program

Dear Sir:

Recently, members of your staff requested clarification of the review/approval process in our startup test program. More specifically, in our discussions, your staff questioned our review/approval process with respect to programs used by other applicants.

To clarify our program, startup test results at Seabrook will undergo a three step review process. The initial review will be performed by the individual test director, who will insure that all required analyses have been performed, that the acceptance criteria have been met and/or that additional action is undertaken to resolve discrepancies. When this review is completed, the entire test package will receive a secondary review by the reactor startup supervisor and by the test group manager (Phases 2-6). After all testing is completed for a given plateau, the reactor startup supervisor and the test group manager (Phases 2-6) will present the results to the Station Operations Review Committee (SORC) for final review and approval. The power level will not be escalated until SORC approval is obtained.

All analyses associated with a given plateau will be completed prior to escalating to the next plateau with the exception of those tests that are listed on the attached Table A-1, which is part of our Startup Test Procedure 1-ST-1. The tests indicated in Table A-1 will undergo a preliminary analysis and review by the reactor startup supervisor, the test group manager (Phase 2-6) and SORC as described in the above paragraph. The attached table indicates those specific tests and the maximum power levels allowed prior to completion of the analyses, review and final SORC approval. This assures that our program does not allow prior escalation without thorough and extensive review of test results.

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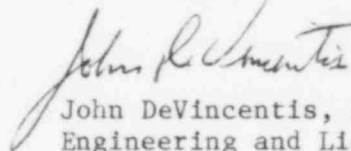
United States Nuclear Regulatory Commission  
Attention: Mr. Vincent S. Noonan

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The attached Table A-1 also reflects the deletion of the Negative Rate Trip Test described in FSAR Table 14.2-5 (Sheet 35). The justification for the deletion of this test is provided in Reference (b). This change to Table 14.2-5 (i.e., see Attachment 1) will be incorporated into the FSAR by a future amendment.

Should you or your staff have any questions, please do not hesitate to contact us. We do request that the acceptability of this test deletion be reflected in the next supplement to Seabrook Station's SER.

Very truly yours,

A handwritten signature in dark ink, appearing to read "John DeVincentis", is written over the typed name.

John DeVincentis, Director  
Engineering and Licensing

Enclosure

cc: Atomic Safety and Licensing Board Service List

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TABLE A-1Test Analysis Schedule Exceptions

<u>Test Results Required Prior To Exceeding</u>	<u>Test Description</u>	<u>Test Performed At</u>	<u>Test Procedure Number</u>
5%	RCS Flow Measurement	Hot Standby	1-ST-11
50%	Boron Endpoint Measurement	HZP	1-ST-17
	Control Bank Worth	HZP	1-ST-20
	Flux Map (Low Power)	HZP	1-ST-19
	Power Coefficient	30%	1-ST-30
75%	RCS Flow Coastdown	Hot Standby	1-ST-12
	Pseudo Rod Ejection	HZP	1-ST-21
	Load Swing	30%, 50%	1-ST-34
	Power Coefficient	50%	1-ST-30
	Flux Map (Rod Drop)	50%	1-ST-31
90%	Rod Drop Test	50%	1-ST-31
	Test		
	Power Coefficient	75%	1-ST-30
	Load Swing	75%	1-ST-34
	Large Load Reduction	75%	1-ST-35

SBN-940

Attachment 1

FSAR Table 14.2-5 Revisions

Seabrook Station

# Attachment 1

SB 1 & 2  
FSAR

Amendment 48  
January 1983

TABLE 14.2-5  
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# Attachment 1

SB 1 & 2  
FSAR

TABLE 14.2-5  
(Sheet 35 of 53)

## 32. NEGATIVE RATE TRIP TEST

### Objective

To demonstrate operation of the negative rate trip circuitry and to monitor plant response following a trip from 50% reactor power.

### Plant Conditions/Prerequisites

The plant will be stable at the 50% power plateau.

### Test Method

Two RCCA's, selected based on their location and worth as the most difficult to detect, will be dropped causing a negative rate trip of the Reactor Protection System. The plant response during the trip will be monitored. The drop time of a selected RCCA will also be monitored.

### Acceptance Criteria

The following criteria will be utilized to determine acceptability:

1. The reactor shall have tripped as a result of a negative rate trip.
2. All full length RCCA's shall be released and shall have bottomed.
3. The pressurizer safety valves shall not have lifted.
4. The steam generator safety valves shall not have lifted.
5. Safety injection was not initiated.

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