

LICENSEE EVENT REPORT (LER)

APPROVED OMB NO. 3180-0107
EXPIRES - 8/31/83

FACILITY NAME (1) INDIAN POINT STATION, UNIT 2										DOCKET NUMBER (3) 0 5 0 0 0 2 4 7 1 OF 0 3										PAGE (3) 1															
TITLE (4) REACTOR TRIP - HIGH STEAM GENERATOR LEVEL																																			
EVENT DATE (8)						LER NUMBER (6)						REPORT DATE (7)						OTHER FACILITIES INVOLVED (8)																	
MONTH		DAY		YEAR		YEAR		SEQUENTIAL NUMBER		REVISION NUMBER		MONTH		DAY		YEAR		FACILITY NAME						DOCKET NUMBER (5)											
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1 0		2 4		8 5		8 5		0 1 4		0 0 1		1 2		2 8		5								0 5 0 0 0 1 1											
OPERATING MODE (9)		THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR 5: (Check one or more of the following) (11)																																	
POWER LEVEL (10)		0 1 1 3		20.402(a)										20.406(a)										80.73(a)(2)(iv)										73.71(a)	
		20.406(a)(1)(i)										80.38(a)(1)										80.73(a)(2)(v)										73.71(a)			
		20.406(a)(1)(ii)										80.38(a)(2)										80.73(a)(2)(vi)										OTHER (Specify in Abstract below and in Text, NRC Form 366A)			
		20.406(a)(1)(iii)										80.73(a)(2)(i)										80.73(a)(2)(vii)(A)													
		20.406(a)(1)(iv)										80.73(a)(2)(ii)										80.73(a)(2)(vii)(B)													
		20.406(a)(1)(v)										80.73(a)(2)(iii)										80.73(a)(2)(ix)													
LICENSEE CONTACT FOR THIS LER (12)																																			
NAME JOHN R. ELLWANGER										TELEPHONE NUMBER 9 1 4 5 2 6 - 5 1 8 2																									
COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)																																			
CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRC	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRC	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRC	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRC	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRC											
SUPPLEMENTAL REPORT EXPECTED (14)										EXPECTED SUBMISSION DATE (15)										MONTH		DAY		YEAR											
YES (If yes, complete EXPECTED SUBMISSION DATE)										X NO																									
ABSTRACT (Limit to 1400 words, i.e., approximately fifteen single-space typewritten lines) (16)																																			

On October 24, 1985, at 5:32 p.m. during restart from a planned outage, the reactor tripped due to high steam generator level. A high steam generator level signal resulted from a steam generator swell at the time of generator load increase just subsequent to bus synchronization. The cause of the event is attributed to operator error caused by a combination of factors. These factors include inadequate training and operator inexperience.

Both long-term and short-term actions have been planned or implemented to correct these circumstances. This includes evaluation of the training program.

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LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

U.S. NUCLEAR REGULATORY COMMISSION

APPROVED OMB NO. 3150-0104

EXPIRES 8/31/85

FACILITY NAME (1)

INDIAN POINT STATION, UNIT 2

DOCKET NUMBER (2)

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LER NUMBER (8)

YEAR

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PAGE (3)

TEXT (if more space is required, use additional NRC Form 366A) (17)

Plant and System Identification:

Westinghouse 4-loop pressurized water reactor; 900 MWe.

Identification of Occurrence:

Actuation of the Reactor Protection System due to high steam generator level.

Event Date:

October 24, 1985

Reportability Determination Date:

October 24, 1985

Report Due Date:

November 23, 1985

Similar Occurrences:

See LER 85-010

This report was initiated by SOR 85-469.

Description of Occurrence:

On October 24, 1985, at 5:32 p.m., while reactor power was at 13%, a high steam generator level occurred in steam generator number 23. This resulted in a main generator trip, causing a turbine and reactor trip. The generator had just been synchronized to the bus and load was being increased. The increase in the steam demand was sufficient to cause a swell in the steam generator which resulted in the high level trip. Just prior to the trip, increasing level in steam generator 23 was noted. The low flow feedwater regulating valve control was adjusted to the closed position and the speed of the turbine driven main boiler feedwater pump was reduced. Control Rod motion in the withdrawal direction was terminated. These actions were not successful and steam generator level continued to increase.

LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

U.S. NUCLEAR REGULATORY COMMISSION

APPROVED OMB NO. 3150-0104

EXPIRES 8/31/95

FACILITY NAME (1)

DOCKET NUMBER (2)

LER NUMBER (8)

PAGE (3)

INDIAN POINT STATION, UNIT 2

YEAR

SEQUENTIAL
NUMBERREVISION
NUMBER

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TEXT (If more space is required, use additional NRC Form 366A (17))

Analysis of Occurrence:

The event represented an unplanned actuation of the Reactor Protection System which functioned as designed. There were no abnormalities detected in the function of any safety related equipment. Thus, there was no impact on the safety of the plant.

Apparent Cause of Occurrence:

The cause of the incident was operator error due to a combination of adverse factors. Evaluation of steam generator level traces indicated oscillating behavior which is not observed in the response of the plant training simulator during this evolution. The response of the simulator to operator manipulation of the low flow feedwater regulating valve control during similar conditions of flow to the steam generator is more rapid than the design response of the plant. Secondly, the operator in control of feedwater flow was proceeding in his first attempt at this task. These factors contributed to the trip. The same operator successfully performed the subsequent start-up. The steam generator level traces showed reduced amplitudes and longer periods indicating the increased proficiency the operator had acquired from the experience.

Corrective Action:

Steps have already been taken to provide additional operator training on plant start-ups prior to each start-up. In addition, the current training program is being evaluated on the basis of operator recommendation to determine improvements.

On a broader scale the Company is also a participant in a Westinghouse Owners Group effort directed towards improving operator response in order to reduce the number of trips.

John D. O'Toole
Vice President

Consolidated Edison Company of New York, Inc.
4 Irving Place, New York, NY 10003
Telephone (212) 460-2533

November 22, 1985

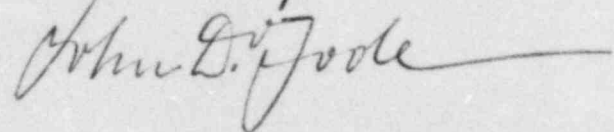
Re: Indian Point Unit No. 2
Docket No. 50-247
LER-85-014-00

Document Control Desk
U. S. Nuclear Regulatory Commission
Washington, D. C. 20555

Dear Sirs:

The attached Licensee Event Report LER-85-014-00 is hereby submitted in accordance with the requirements of 10 CFR Part 50.73.

Very truly yours,



attach.

cc: Dr. Thomas E. Murley,
Regional Administrator-Region I
U. S. Nuclear Regulatory Commission
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King of Prussia, Pa. 19406

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