

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1) H. B. Robinson Unit 2										DOCKET NUMBER (2) 0 5 0 0 0 2 6 1				PAGE (3) 1 OF 0 2		
TITLE (4) Reactor Trip - Steam Flow/Feed Flow Mismatch Coincident with Low Level "B" S/G																
EVENT DATE (5)			LER NUMBER (6)				REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)						
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES				DOCKET NUMBER(S)			
0 8	2 2	8 5	8 5	0 1 7	0 0	0 9	1 7	8 5					0 5 0 0 0			
OPERATING MODE (9)			THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: (Check one or more of the following) (11)													
POWER LEVEL (10) 1 0 0			20.402(b)				20.405(c)				<input checked="" type="checkbox"/> 50.73(a)(2)(iv)				73.71(b)	
			20.405(a)(1)(i)				50.38(c)(1)				50.73(a)(2)(v)				73.71(c)	
			20.405(a)(1)(ii)				50.36(c)(2)				50.73(a)(2)(vi)				OTHER (Specify in Abstract below and in Text, NRC Form 366A)	
			20.405(a)(1)(iii)				50.73(a)(2)(i)				50.73(a)(2)(vii)(A)					
			20.405(a)(1)(iv)				50.73(a)(2)(ii)				50.73(a)(2)(vii)(B)					
			20.405(a)(1)(v)				50.73(a)(2)(iii)				50.73(a)(2)(x)					
LICENSEE CONTACT FOR THIS LER (12)																
NAME Carson L. Wright										TELEPHONE NUMBER AREA CODE 8 0 3 3 8 3 - 4 5 2 4						
COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)																
CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPDs		CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPDs						
SUPPLEMENTAL REPORT EXPECTED (14)												EXPECTED SUBMISSION DATE (15)		MONTH	DAY	YEAR
<input type="checkbox"/> YES (If yes, complete EXPECTED SUBMISSION DATE)												<input checked="" type="checkbox"/> NO				

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

On August 22, 1985, the Plant tripped from 100% power due to a steam flow greater than feed flow (SF>FF) mismatch coincident with low steam generator (S/G) level. The trip was caused by performing the steps of a procedure out of sequence which resulted in isolating a steam pressure transmitter prior to transferring steam pressure control to a redundant instrument. The steam pressure transmitter provides a compensating signal to the steam flow instrumentation. The loss of the steam pressure signal caused the steam generator level control system to match actual feedwater flow to the false low steam flow signal by closing "B" Feedwater Regulating Valve. This resulted in the SF>FF coincident with a low S/G level reactor trip.

Corrective action is to provide improved procedural guidance for work on instrumentation that could result in a Plant trip.

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LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

U.S. NUCLEAR REGULATORY COMMISSION

APPROVED OMB NO. 3150-0104

EXPIRES 8/31/85

FACILITY NAME (1) H. B. Robinson Unit 2	DOCKET NUMBER (2) 0 5 0 0 0 2 6 1 8 5	LER NUMBER (6)			PAGE (3)		
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER			
		— 0 1 7	— 0 0 0 2	OF	0 2		

TEXT (If more space is required, use additional NRC Form 366A's) (17)

On August 22, 1985, the Plant was operating at 100% reactor power. At 0910 hours, the reactor tripped due to a steam flow greater than the feedwater flow (SF>FF) coincident with a low level signal from "B" Steam Generator (S/G). The "B" Steam Line pressure transmitter PT-485, was isolated to repair a leak on a sensing line. An Operating Work Procedure (OWP), used primarily for electrical maintenance of the transmitter, was used to remove PT-485 from service. A temporary change to the OWP added closing the transmitter sensing line isolation valve to isolate and repair the leak. The trip occurred because the temporary change, isolation of PT-485, was performed prior to the step which transferred steam pressure control from PT-485 to a redundant instrument. The steam flow signal to the S/G level control circuit is compensated for steam pressure. Isolating PT-485 resulted in a low steam flow signal. "B" Feedwater Regulating Valve (FWRV) started closing attempting to match actual feedwater flow to the false low steam flow coincident with an actual low steam generator level reactor trip.

Corrective Action is to provide improved procedural guidance for work on instrumentation that has a potential to affect the reactor trip or safeguards system in the Plant.

The Plant was returned on-line at 1829 hours on August 22, 1985.



Carolina Power & Light Company

ROBINSON NUCLEAR PROJECT DEPARTMENT
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HARTSVILLE, SOUTH CAROLINA 29550

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Serial: RNP/85-2992

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Washington, D.C. 20555

H. B. ROBINSON STEAM ELECTRIC PLANT, UNIT NO. 2
DOCKET NO. 50-261
LICENSE NO. DPR-23
LICENSEE EVENT REPORT 85-017

Dear Sir:

In accordance with 10CFR50.73, Licensee Event Report, the enclosed Licensee Event Report is submitted. This report fulfills the requirements for a written report within (30) days of a reportable event and is in accordance with the format set forth in NUREG-1022, September, 1983.

Very truly yours,

R. E. Morgan
General Manager
H. B. Robinson S. E. Plant

CLW/ac

Enclosure

cc: INPO
J. N. Grace
H. E. P. Krug

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