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January 23, 1986

Director
Office of Nuclear Reactor Regulation
U S Nuclear Regulatory Commission
Washington, DC 20555

PRAIRIE ISLAND NUCLEAR GENERATING PLANT
Docket Nos. 50-282 License Nos. DPR-42
50-306 DPR-60

Unit 2 Inservice Inspection Summary Report

Attached for your review are four copies of the report. "Prairie Island Nuclear Generating Plant - Unit 2, Inservice Inspection - Examination Summary, September 9, 1985 to October 21, 1985". This is the first inservice inspection conducted for inspection period one of the second ten year interval.

This report identifies the components examined, the examination methods used, the examination number and summarizes the examination results of each of the following areas:

1. Pressure retaining components and supports of the reactor coolant and associated systems classified as ASME Class 1 and ASME Class 2.
2. Reactor pressure vessel remote tool examinations.
3. Eddy current examination of Steam Generator tubing. Seven tubes in No. 21 steam generator and twelve tubes in No. 22 steam generator were mechanically plugged.
4. Safety injection line replacement.

The evaluation of the results from the inservice examinations indicated that integrity of the systems has been maintained.

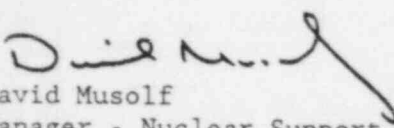
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Dir of NRR
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This report is being submitted in accordance with the Prairie Island ASME Code Section XI Inservice Inspection and Testing Program. As noted in that program, submittal of the attached summary is intended to satisfy the inspection reporting requirements contained in IWA-6220 of Section XI of the ASME Code.


David Musolf
Manager - Nuclear Support Services

DMM/EFE/efe

- c: Regional Administrator-III, NRC (2 copies of attachment)
- NRR Project Manager, NRC (1 copy of attachment)
- Resident Inspector, NRC (w/o attachment)
- G Charnoff (w/o attachment)
- H Baron, Chief Boiler Insp., State of MN (1 copy of attachment)
- S Tact, Hartford Ins. (1 copy of attachment)

Attachment