

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1): Oyster Creek, Unit 1										DOCKET NUMBER (2): 0 5 0 0 0 2 1 9 1										PAGE (3): 1 OF 4																																																						
TITLE (4): Loss of Power to 480 Volt Unit Substations																																																																										
EVENT DATE (5):						LER NUMBER (6):						REPORT DATE (7):						OTHER FACILITIES INVOLVED (8):																																																								
MONTH			DAY			YEAR			YEAR			SEQUENTIAL NUMBER			REVISION NUMBER			MONTH			DAY			YEAR			FACILITY NAMES						DOCKET NUMBER(S)																																									
0			9			1			0			8			4			8			4			-			0			2			0			-			0			1			0			9			1			1			8			5			0 5 0 0 0						0 5 0 0 0					
OPERATING MODE (9): N		THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR 5: (Check one or more of the following) (11)																																																																								
POWER LEVEL (10): 0 1 0 0		20.402(b)						20.406(a)						X 80.73(a)(2)(iv)						73.71(b)																																																						
		20.406(a)(1)(i)						80.36(a)(1)						80.73(a)(2)(v)						73.71(e)																																																						
		20.406(a)(1)(ii)						80.36(a)(2)						X 80.73(a)(2)(vi)						OTHER (Specify in Abstract below and in Text, NRC Form 365A)																																																						
		20.406(a)(1)(iii)						80.73(a)(2)(i)						80.73(a)(2)(vii)(A)																																																												
		20.406(a)(1)(iv)						80.73(a)(2)(ii)						80.73(a)(2)(vii)(B)																																																												
		20.406(a)(1)(v)						80.73(a)(2)(iii)						80.73(a)(2)(x)																																																												
LICENSEE CONTACT FOR THIS LER (12)																																																																										
NAME: Arthur C. Dickinson, Engineer										TELEPHONE NUMBER: 610 9 917 11 - 146 216																																																																
COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)																																																																										
CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPDOS		CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPDOS																																																																
B	ELD	0 0 5 1	G 0 8 0	Y																																																																						
SUPPLEMENTAL REPORT EXPECTED (14)										EXPECTED SUBMISSION DATE (15)						MONTH DAY YEAR																																																										
YES (If yes, complete EXPECTED SUBMISSION DATE)										X NO																																																																

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

While transferring from parallel to single unit substation operation (480 VAC), power was lost to 1A2 and 1B2 buses, resulting in a full scram, reactor primary containment and secondary containment isolation, and loss of both Standby Gas Treatment System trains. The bus tie breaker US2T was closed. When unit substation breaker 1A2M was opened, unit substation breaker 1B2M tripped on short time overcurrent. Breaker 1B2M was reclosed and reloaded and assumed load on both buses. When power was restored, systems were lined up and returned to normal. The Standby Gas Treatment System was without power for approximately 37 seconds.

8509190630 850911
PDR ADOCK 05000219
S PDR

IE22
11

LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

APPROVED OMB NO 3150-0104

EXPIRES 8/31/85

FACILITY NAME (1)	DOCKET NUMBER (2)	LER NUMBER (6)			PAGE (3)		
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER			
Oyster Creek, Unit 1	0 5 0 0 0 2 1 9	8 4	— 0 2 0	— 0 1	0 2	OF	0 4

TEXT (If more space is required, use additional NRC Form 365A's) (17)

IDENTIFICATION OF OCCURRENCE:

Loss of power to redundant 480 volt unit substations caused inoperability of required safety systems and actuation of engineered safety feature systems. This event is considered to be a reportable occurrence per 10CFR 50.73 (a) (2) (iv) and (a)(2)(vi).

CONDITIONS PRIOR TO OCCURRENCE:

The reactor mode switch was in the SHUTDOWN position. Reactor coolant temperature was less than 212°F. The reactor was vented to primary containment. Plant systems were being lined up for a Primary Containment Integrated Leak Rate Test.

DESCRIPTION OF OCCURRENCE:

In preparation for testing of primary unit substation relays, the 480V unit substation 1A2 was being lined up to be powered from the redundant unit substation 1B2 through tie breaker US2T so normal supply breaker 1A2M could be opened. As directed by a plant procedure, which has been used without incident both prior to and subsequent to this event, the tie breaker US2T was closed and the normal supply breaker 1A2M was opened. When breaker 1A2M was opened, breaker 1B2M tripped immediately causing loss of power to both unit substations 1A2 and 1B2. To regain power, tie breaker US2T was opened, supply breaker 1B2M was reset and closed and tie breaker US2T was reclosed. Power was restored to unit substations 1A2 and 1B2 in approximately 37 seconds. The remaining equipment which is supplied by these buses was restored.

The loss of power caused a reactor scram, reactor isolation, primary containment isolation, and secondary containment isolation. While power was lost to unit substation 1A2 and 1B2, AC powered isolation valves and safety systems were inoperable.

LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

APPROVED: OMB NO. 3150-0104
EXPIRES: 8/31/85

FACILITY NAME (1)	DOCKET NUMBER (2)	LER NUMBER (6)			PAGE (3)		
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER			
Oyster Creek, Unit 1	0 5 0 0 0 2 1 9	8 4	- 0 2 0	- 0 1	0 3	OF	0 4

TEXT (If more space is required, use additional NRC Form 366A's) (17)

APPARENT CAUSE:

Breaker 1B2M was removed for maintenance the next day (September 11, 1984). The overcurrent device (G.E. EC-1B) was found to trip at a current 10 percent of the normal short time setting.

The apparent cause of this event was a faulty overcurrent device which caused breaker 1B2M to trip on overload due to the current surge when breaker 1A2M was opened.

ANALYSIS OF OCCURRENCE AND SAFETY SIGNIFICANCE:

The safety significance of the immediate consequences of this event are considered minimal due to the following:

- a) The reactor was in cold shutdown.
- b) Due to plant conditions, and work in progress at the time, most safety related systems affected by the loss of power were not required to be operable or fully operable.
- c) Due to plant conditions, and work in progress at the time, neither primary nor secondary containment integrity were required to be maintained.
- d) The loss of power was of short duration and was corrected by operation of normal plant controls.
- e) The main core spray pumps were available (without booster pumps) and with manual valve operation had core spray systems been required.

The full scram and isolation occurred because both reactor protection systems' power supplies were tied together. During normal reactor operation, the buses would not be connected as required by plant procedure.

Similarly, the loss of redundant safety equipment would not have occurred during normal plant operation. The redundant electrical systems are restricted from being tied together in accordance with plant procedure.



GPU Nuclear Corporation
Post Office Box 388
Route 9 South
Forked River, New Jersey 08731-0388
609 971-4000
Writer's Direct Dial Number:

September 11, 1985

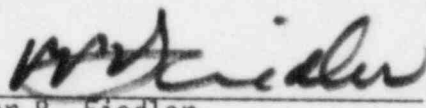
U.S. Nuclear Regulatory Commission
Document Control Desk
Washington, DC 20555

Dear Sir:

Subject: Oyster Creek Nuclear Generating Station
Docket No. 50-219
Licensee Event Report Revision

This letter forwards one (1) copy of Licensee Event Report (LER) No. 84-20-01.

Very truly yours,


Peter B. Fiedler
Vice President and Director
Oyster Creek

PBF:PC:dam (#0741V)
Enclosures

cc: Dr. Thomas E. Murley, Administrator
Region I
U.S. Nuclear Regulatory Commission
631 Park Avenue
King of Prussia, PA 19406

NRC Resident Inspector
Oyster Creek Nuclear Generating Station
Forked River, NJ 08731

IE22
11