

## LICENSEE EVENT REPORT (LER)

APPROVED NRC NO. 2100-0104  
EXPIRES 6/30/93

TITLE (1)

Limerick Generating Station - Unit 1

DOCKET NUMBER (2)

05000352

PAGE (3)

1 OF 1

TITLE (4)

Failure to Meet Hourly Fire Watch Requirements in Technical Specifications

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)							
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAME		DOCKET NUMBER (9)					
1	2	2	6	8	5	8	5	1	0	0	0	5	0	0	0	1
THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR 50.71 (Check one or more of the following) (11)																
OPERATING MODE (10)			20.400 (a)			20.400 (b)			20.730 (a) (1) (i)			20.730 (b)				
POWER LEVEL (10)			20.400 (a) (1) (i)			20.730 (a) (1) (i)			20.730 (a) (1) (i)			20.730 (b)				
			20.400 (a) (1) (i)			20.730 (a) (1) (i)			20.730 (a) (1) (i)			20.730 (b)				
			20.400 (a) (1) (i)			20.730 (a) (1) (i)			20.730 (a) (1) (i)			20.730 (b)				
			20.400 (a) (1) (i)			20.730 (a) (1) (i)			20.730 (a) (1) (i)			20.730 (b)				
			20.400 (a) (1) (i)			20.730 (a) (1) (i)			20.730 (a) (1) (i)			20.730 (b)				
			20.400 (a) (1) (i)			20.730 (a) (1) (i)			20.730 (a) (1) (i)			20.730 (b)				

LICENSEE CONTACT FOR THIS LER (12)

NAME	TELEPHONE NUMBER
John C. Nagle, Senior Engineer, Licensing Section	215 841-5184

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THE REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRC	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRC

SUPPLEMENTAL REPORT EXPECTED (14)

YES (If yes, complete EXPECTED SUBMISSION DATE)	NO	EXPECTED SUBMISSION DATE (15)	MONTH	DAY	YEAR
	X				

ABSTRACT (Limit to 1000 spaces, i.e., approximately 1000 characters) (16)

Abstract: 85-100

On December 26, 1985 with Unit 1 at 98 percent power, hourly fire watch inspections for four fire barriers located in the Reactor Enclosure were not performed within one hour of the previous inspections. The four inspection intervals exceeded the one-hour limitation of Technical Specification 3.7.7. The maximum inspection interval, which began at 1408 hours, was one hour and twenty-one minutes. The reactor enclosure supply fans had tripped and the exhaust fans continued to operate, which resulted in negative Reactor Enclosure pressure. As a result of the negative pressure in the Reactor Enclosure, the fire watch was unable to open the airlock doors and enter the posted areas.

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## LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

U.S. NUCLEAR REGULATORY COMMISSION

APPROVED OMS NO. 3198-0104

EXPIRES 6/31/85

FACILITY NAME (1)

DOCKET NUMBER (2)

LER NUMBER (3)

PAGE (3)

Limerick Generating Station  
Unit 1

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TEXT (if more space is required, use additional NRC Form 266a (1))

Unit Conditions Prior to the Event:

Mode 1 (Power Operation)  
Reactor Power 98%  
Power Ascension Testing in Progress

Description of the Event:

On December 26, 1985, hourly fire watch inspections for four fire barriers, which "separate safe shutdown fire areas or separate portions of redundant systems important to safe shutdown within a fire area", were not performed within one hour of the previous inspections. The maximum inspection interval, which began at 1408 hours, was one hour and twenty-one minutes. The intervals for the four inspections exceeded the one hour limitation of Technical Specification 3.7.7.

The three fire doors and one fire damper of concern are located in the Reactor Enclosure at the 177, 217, and 283 feet elevations.

The EIIS code for the affected system, Fire Protection, is KP.

Consequences of the Event:

The possible consequences of this event were minimal because smoke detectors in each of the four areas were operable and would have provided early detection of fires in those areas. The four areas have low combustible loadings (weight of combustible material/floor area) and one of the areas is partially protected by a pre-action sprinkler system.

## LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

FACILITY NAME (1) Limerick Generating Station Unit 1	DOCKET NUMBER (2) 0500035285	LER NUMBER (6)			PAGE (3)		
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER			
		85	100	00	03	OF	03

TEXT (If more space is required, use additional NRC Form 366A) (17)

Cause of the Event:

The four fire watch inspection intervals exceeded the one-hour limit because the fire watch was prevented from entering the posted areas because of a ventilation equipment failure. The reactor enclosure supply fans had tripped due to low inlet air temperature caused by the auxiliary boilers being out of service. The exhaust fans continued to operate, which resulted in negative Reactor Enclosure pressure. Consequently, the fire watch was unable to physically open the airlock doors and enter the posted areas.

Corrective Actions:

The operating shift was involved with returning the auxiliary boilers to service, when they were notified of the fire watch's inability to enter the Reactor Enclosure at 1456 hours. The operating shift continued to return the auxiliary boilers to service. At 1516 hours they manually removed the Reactor Enclosure exhaust fans from service, thereby causing an isolation of the Reactor Enclosure in order to reduce the negative pressure. The fire watch was able to enter the Reactor Enclosure by 1520 hours. The Standby Gas Treatment System and Reactor Enclosure Recirculation System started as designed when the Reactor Enclosure isolation signal was generated.

Previous Similar Occurrences:

Other Limerick LERs have reported failures to meet hourly fire watch requirements; however, none of these events were the result of negative Reactor Enclosure pressure.

PHILADELPHIA ELECTRIC COMPANY

2301 MARKET STREET

P.O. BOX 8699

PHILADELPHIA, PA. 19101

(215) 841-4000 January 24, 1986

Docket No. 50-352

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Washington, DC 20555

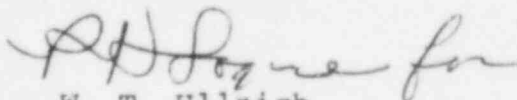
SUBJECT: Licensee Event Report  
Limerick Generating Station - Unit 1

This LER concerns failures to meet hourly fire watch requirements of the Technical Specifications.

Reference:	Docket No. 50-352
Report Number:	85-100
Revision Number:	00
Event Date:	December 26, 1985
Report Date:	January 24, 1986
Facility:	Limerick Generating Station P.O. Box A, Sanatoga, PA 19464

This LER is being submitted pursuant to the requirements of 10 CFR 50.73(a)(2)(i)(B).

Very truly yours,



W. T. Ullrich  
Superintendent  
Nuclear Generation Division

cc: Dr. Thomas E. Murley, Administrator, Region I, USNRC  
E. M. Kelly, Senior Resident Site Inspector  
See Service List

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Atomic Safety & Licensing Board Panel  
Docket & Service Section (3 Copies)  
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Timothy R. S. Campbell

September, 1985