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SACRAMENTO MUNICIPAL UTILITY DISTRICT 6201 S Street, P.O. Box 15830, Sacramento CA 95852-1830, (516) 452-3211
AN ELECTRIC SYSTEM SERVING THE HEART OF CALIFORNIA

RJR 86-26

January 24, 1986

J B MARTIN REGIONAL ADMINISTRATOR
REGION V OFFICE OF INSPECTION AND ENFORCEMENT
ATTENTION DOCUMENT CONTROL DESK
U S NUCLEAR REGULATORY COMMISSION
WASHINGTON D C 20555

DOCKET NO. 50-312
LICENSE NO. DPR-54
SPECIAL REPORT NO. 86-01, DIESEL DRIVEN FIRE PUMP

The Sacramento Municipal Utility District hereby submits Special Report 86-01 in accordance with Rancho Seco Nuclear Generating Station Technical Specifications, Sections 3.14.2.2 and 6.9.5E.

On January 7, 1986, it was reported by the District's Nuclear Operations Fire Protection Coordinator that the diesel driven fire pump (P-996) located in the raw water chemical mix building had been inoperable during the period between December 21, 1985 and December 31, 1985. The diesel driven fire pump provides a backup capability for the motor driven fire pump (P-440) in the Fire Suppression Water System. It should be noted that because P-440 was operational, the Fire Suppression Water System remained functional during this period of concern. The period of inoperability exceeded the seven (7) days allowed by Technical Specification 3.14.2.2 and requires the submittal of this report.

P-996 was observed to be overheating on December 21, 1985, during an automatically initiated weekly test of the pump. The pump was declared inoperable by Operations, a clearance tag was attached, and a work request was written to investigate a suspected outboard bearing problem.

As a result of the work request, the outboard bearings of the pump were inspected, the pump was operated for thirty (30) minutes, and the bearing temperatures were measured and found to be within specifications. No problems were found and the clearance tag was removed from P-996 on December 24, 1985.

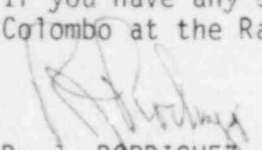
At this time, the only item required to return P-996 to an operable status was the successful completion of surveillance procedure SP 201.03B. However, SP 201.03B was not completed until December 31, 1985, at which time the seven days for returning the pump to an operable status allowed by Technical Specification 3.14.2.2 had been exceeded.

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An investigation was made to determine why the allowed time was exceeded. The investigation showed (1) that the time requirement was explicitly stated on the work request and was reflected in the priority assigned to the task, (2) that the pump status was properly tracked during the Control Room shift turnovers, and (3) that the focus of attention on activities surrounding the December 26, 1985 trip was a major contributor in missing the required surveillance. In addition to the investigation, the District's Incident Analysis Group will perform a root cause analysis of this concern to determine if and what procedural changes are needed to prevent a recurrence. This analysis is expected to be completed by March 31, 1986.

If you have any questions concerning this report, please contact Mr. Ron W. Colombo at the Rancho Seco Nuclear Generating Station Unit No. 1.



R. J. RODRIGUEZ
ASSISTANT GENERAL MANAGER,
NUCLEAR

cc: Director NRR