

FPL NUCLEAR DIVISION

TURKEY POINT NUCLEAR STATION
ST. LUCIE NUCLEAR STATION

MONTHLY PERFORMANCE MONITORING MANAGEMENT INFORMATION REPORT

November 1995

BBB/46

Issued: December 18, 1995

FPL NUCLEAR DIVISION

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Inter-Office Correspondence



FPL

JNE-NBS-95-217

TO: Page Z-1 in Attachment

DATE: December 18, 1995

FROM: R. L. Wade

LOCATION: Nuclear Division/JB

SUBJECT: PERFORMANCE MONITORING MANAGEMENT INFORMATION

Enclosed is the Nuclear Division Performance Monitoring Management Information Report (PMMI) for **November 1995**.

Equivalent Availability Factor (EAF). The combined Y-T-D Equivalent Availability Factor for both Turkey Point units through November was 92.4% which was higher than the 87.9% target. St. Lucie's combined Y-T-D Equivalent Availability for both units through November was 76.4% which was below the 89.9% target. For the Division, EAF year-end projection is 83.5%.

O&M expenditures through November 1995 were \$242.9 million which represented a budget underrun of \$11.9 million (or 4.7%). The variance was primarily due to: underruns in materials and contracted services at Turkey Point due to less trouble and breakdown than planned; Nuclear Division O&M Contingency not yet needed; and over-accrual of estimated 1994 Year-End liabilities.

Capital expenditures through November 1995 were \$37.5 million. This represented a budget underrun of \$19.4 million (or 34.0%). The variance was primarily due to: project cancellations, scope/estimate reductions, and schedule changes for various projects; underrun in Contractor Wrap-Up Insurance; and, delay in support services and staffing under budget for the Steam Generator Replacement Project at St. Lucie.

Fuel expenditures through November 1995 were \$120.3 million representing a budget underrun of \$0.3 million (or 0.3%). The underrun was primarily due to lower than planned electrical generation partially offset by a higher than planned amortization rate.

OSHA Recordable Injuries. St. Lucie reported seven OSHA Recordable Injuries in November. Year-to-date, the Nuclear Division reported a rate of 2.08, which is below the Corporate Target of 3.10.

Unplanned Automatic Trips. No Unplanned Automatic Trips were reported in November. Year-to-date, St. Lucie has experienced two auto trips.

Any questions or comments concerning this report may be addressed to John Pizzutelli at 694-4245 or to me at 694-3786.

A handwritten signature in cursive script that reads 'R. L. Wade'.

R. L. Wade
Manager, Business Services

RLW/JGP/skb

Attachment

FOREWORD

The Nuclear Division Performance Monitoring Management Information Report presents a monthly compilation of performance indicators which provide a quantitative indication of plant performance. Specific areas of focus include nuclear and personnel safety, plant reliability, and economic performance.

The specific indicators included in this report have been selected by senior management as key indicators of operating performance. Summaries of NRC indicator and INPO indicator performance have been incorporated in this report.

Data contained herein will be refined on the basis of feedback from data providers, of continuing quality control efforts, and of comparisons to other data sources. Each monthly report will reflect the best available data.

TABLE OF CONTENTS

	PAGE		PAGE
Forward	i		
Table of Contents	ii		
Executive Summary	iii	B. COST PERFORMANCE	
Nuclear Division Business Plan Indicator Overview	v	O&M Budget Variance - Division	B-1
NRC Indicator Performance Overview	vi	Capital Budget (Non-Fuel) Variance - Division	B-2
INPO Overall Indicator Performance Overview	vii	Fuels Variance - PTN	B-3
INPO Overall Indicator Performance Overview - Discussion of FPL Performance Unfavorable to Industry Mean	viii	Fuels Variance - PSL	B-4
		Inventory Levels - PTN	B-5
		Inventory Levels - PSL	B-6
A. OPERATIONS		C. CHEMISTRY & HEALTH PHYSICS	
Operations Highlights	A-1	Collective Radiation Exposure - PTN	C-1
Equivalent Availability - PTN Unit 3	A-2	Collective Radiation Exposure - PSL	C-2
Equivalent Availability - PTN Unit 4	A-3	Secondary Chemistry Performance - PTN	C-3
Equivalent Availability - PSL Unit 1	A-4	Secondary Chemistry Performance - PSL	C-4
Equivalent Availability - PSL Unit 2	A-5	Solid Waste Disposed - PTN	C-5
Forced Outage Rate - PTN Unit 3	A-6	Solid Waste Disposed - PSL	C-6
Forced Outage Rate - PTN Unit 4	A-7		
Forced Outage Rate - PSL Unit 1	A-8	D. LICENSING	
Forced Outage Rate - PSL Unit 2	A-9	NRC Violations - PTN	D-1
Unplanned Auto Trips While Critical - PTN Units 3 & 4	A-10	NRC Violations - PSL	D-2
Unplanned Auto Trips While Critical - PTN Units 1 & 2	A-11	Licensee Event Reports - PTN Unit 3	D-3
Safety System Failures - PTN	A-12	Licensee Event Reports - PTN Unit 4	D-4
Safety System Failures - PSL	A-13	Licensee Event Reports - PSL Unit 1	D-5
Unplanned Safety System Actuations - PTN Units 3 & 4	A-14	Licensee Event Reports - PSL Unit 2	D-6
Unplanned Safety System Actuations - PTN Units 1 & 2	A-15	INPO Assessment Ratings	D-7
Safety System Performance: Safety		SALP Category Ratings	D-8
Injection System - PTN	A-16		
Safety System Performance: High Pressure Safety		E. TRAINING	
Injection System - PSL	A-17	Operator Examination Performance - PTN	E-1
Safety System Performance: Auxiliary Feedwater		Operator Examination Performance - PSL	E-2
System - PTN	A-18		
Safety System Performance: Auxiliary Feedwater		F. EMPLOYEE SAFETY	
System - PSL	A-19	Industrial Safety Statistics - PTN	F-1
Safety System Performance: Emergency Diesel		Industrial Safety Statistics - PSL	F-2
Generator Unavailability - PTN Unit 3	A-20		
Safety System Performance: Emergency Diesel			
Generator Unavailability - PTN Unit 4	A-21		
Safety System Performance: Emergency Diesel			
Generator Unavailability - PSL Unit 1	A-22		
Safety System Performance: Emergency Diesel			
Generator Unavailability - PSL Unit 2	A-23		
Fuel Reliability - PTN Unit 3	A-24		
Fuel Reliability - PTN Unit 4	A-25		
Fuel Reliability - PSL Unit 1	A-26		
Fuel Reliability - PSL Unit 2	A-27		
Fuel Utilization - PTN Unit 3	A-28		
Fuel Utilization - PTN Unit 4	A-29		
Fuel Utilization - PSL Unit 1	A-30		
Fuel Utilization - PSL Unit 2	A-31		
Capacity Factor (MDC Net) - PTN	A-32		
Capacity Factor (MDC Net) - PSL	A-33		
Thermal Performance (Gross Heat Rate) - PTN	A-34		
Thermal Performance (Gross Heat Rate) - PSL	A-35		
		Appendix	
		Distribution List	Z-1

EXECUTIVE SUMMARY

DIVISION

- **O&M expenditures** through November 1995 were \$242.9 million which represented a budget underrun of \$11.9 million (or 4.7%). The variance was primarily due to: underruns in materials and contracted services at Turkey Point due to less trouble and breakdown than planned; Nuclear Division O&M Contingency not yet needed; and over-accrual of estimated 1994 Year-End liabilities.

YTD O&M budget performance variances are stratified as follows:

<u>Turkey Point Site Specific</u>	\$ 5.7 million (or 5.0%) below budget
<u>St. Lucie Site Specific</u>	\$ 0.9 million (or 0.9%) below budget
<u>Other Nuclear Division</u>	\$ 5.3 million (or 15.9%) below budget

- **Capital expenditures** through November 1995 were \$37.5 million. This represented a budget underrun of \$19.4 million (or 34.0%). The variance was primarily due to: project cancellations, scope/estimate reductions, and schedule changes for various projects; underrun in Contractor Wrap-Up Insurance; and, delay in support services and staffing under budget for the Steam Generator Replacement Project at St. Lucie.

Y-T-D Capital budget performance variances are stratified as follows:

<u>Turkey Point Site Specific</u>	\$ 3.0 million (or 27.7%) below budget
<u>St. Lucie Site Specific</u>	\$ 7.8 million (or 20.6%) below budget
<u>Other Nuclear Division</u>	\$ 8.6 million (or 101.5%) below budget

PLANT & STAFF

- Turkey Point reported no NRC Violations in November. Year-to-date, Turkey Point has received one NRC Violation.
- St. Lucie reported three NRC Violations for November:
 - #95-18-01 - Failure to Follow Procedure and Not Properly Log Locked Key-Switch Deviations.
Exit Meeting Date: 11/1/95.
 - #95-18-02 - Failure to Follow Procedure and Issue a Clearance, Which Resulted in Personnel Injury.
Exit Meeting Date: 11/1/95.
 - #95-18-03 - Inadequate Design of Unit 2 EDG Resulting in Reverse Powering of the EDG During ESFAS Testing. Exit Meeting Date: 11/1/95.

Year-to-date, St. Lucie has received 17 NRC Violations.

Other plant statistics are as follows:

	<u>PTN Unit 3</u>		<u>PTN Unit 4</u>	
	<u>November</u>	<u>Y-T -D</u>	<u>November</u>	<u>Y-T -D</u>
Gross Generation (MWh)	507,903	4,918,932	500,532	5,548,918
Net Generation	484,808	4,687,455	477,225	5,288,128
Net Heat Rate	10806.6	11057.6	10947.7	11042.3
Equivalent Availability	99.7%	87.6%	96.8%	97.2%
Capacity Factor	101.1%	87.8%	99.5%	99.1%
Auto Trips	0	0	0	0
Forced Outage Rate	0.0%	0.6%	0.0%	1.6%

	<u>PSL Unit 1</u>		<u>PSL Unit 2</u>	
	<u>November</u>	<u>Y-T -D</u>	<u>November</u>	<u>Y-T -D</u>
Gross Generation (MWh)	576,770	5,176,380	0	5,606,960
Net Generation	545,144	4,879,055	- 1,761	5,294,188
Net Heat Rate	11068.4	11003.0	0.0	11127.1
Equivalent Availability	90.9%	73.1%	0.0%	79.7%
Capacity Factor	90.2%	72.6%	0.0%	78.7%
Auto Trips	0	1	0	1
Forced Outage Rate	4.3%	23.8%	0.0%	2.4%

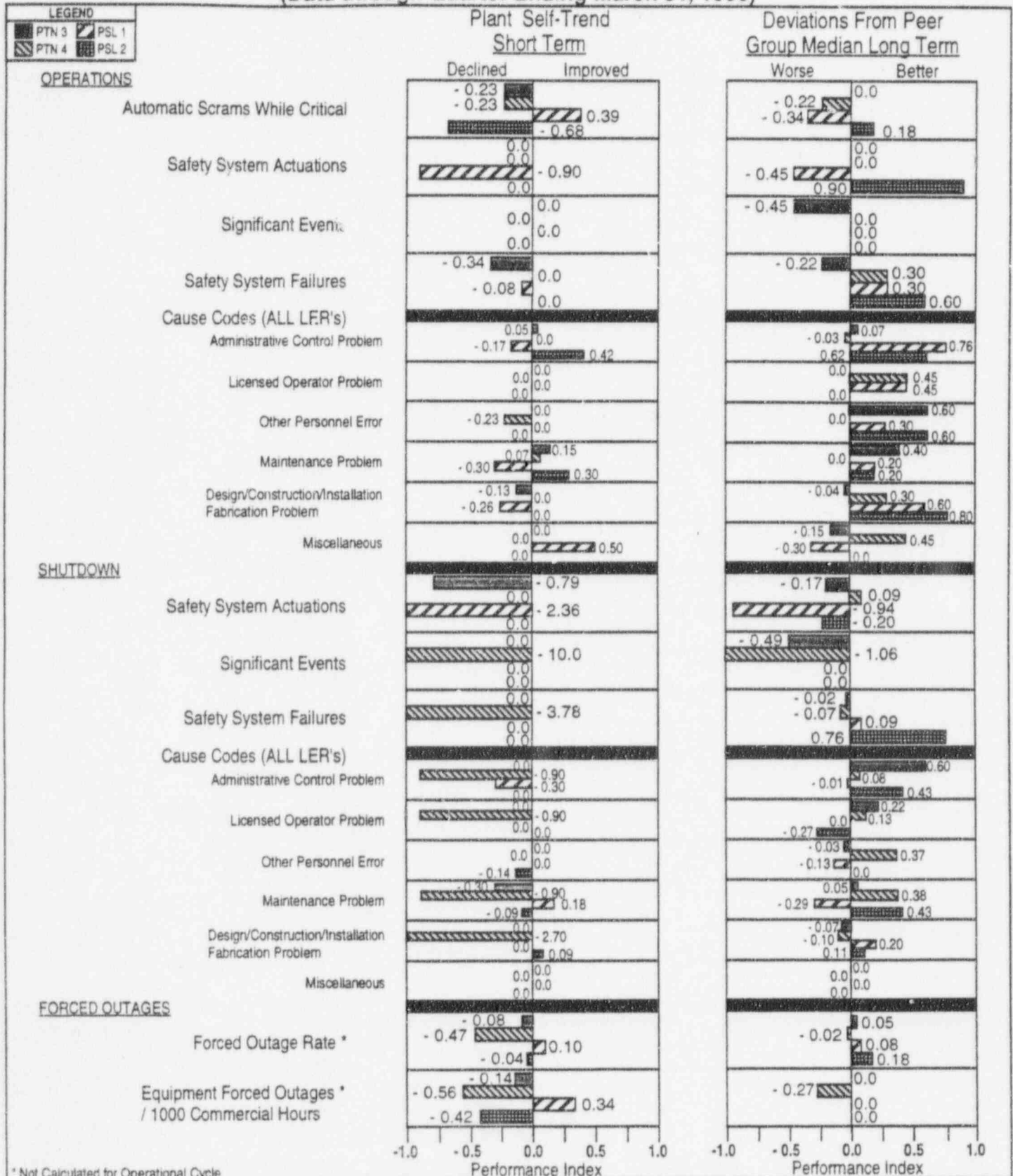
NUCLEAR DIVISION BUSINESS PLAN INDICATOR OVERVIEW

(DATA THROUGH NOVEMBER 1995)

Indicator		Plant	1995 Y-E Target	1995 Y-T-D Target	1995 Y-T-D Actuals	Comments
Collective Radiation Exposure (Man-Rem)		PTN PSL	≤ 275 ≤ 240	270.0 231.0	212.6 352.2	St. Lucie Man-Rem for November exceeded the Y-T-D target.
NRC SALP Ratings		PTN PSL	1.00 1.25		1.00 1.00	The SALP period for St. Lucie ends Jan. 6, 1996; for PTN, the period ends Aug. 31, 1996.
NRC Violations		PTN PSL	≤ 7 ≤ 7		1 17	PTN reported no NRC Violations in November. PSL reported three for the month.
Unplanned Automatic Trips		PTN3 PTN4 PSL1 PSL2	≤ 1 ≤ 1 ≤ 1 ≤ 1		0 0 1 1	No auto trips occurred in November. Y-T-D, St. Lucie has experienced two auto trips.
Budget Performance (\$ Millions)		Capital O&M Div. Total	64.7 279.2 343.9	56.4 244.4 300.8	37.5 242.9 280.4	O&M and Capital actuals were below Y-T-D targets in November; underruns were mainly due to schedule revisions and project underruns.
Equivalent Availability Factor (%)		PTN3 PTN4 PSL1 PSL2	82.0 95.0 95.0 82.0	80.7 95.0 95.0 84.7	87.6 97.2 73.1 79.7	EAF(%) for PTN exceeded Y-T-D targets in November. The Division EAF Y-T-D was 84.4% and below the 88.9% target.
Inventory Levels (\$ Millions)		PTN Cap. PTN Reg. PSL Cap. PSL Reg.	n/a ≤ 39 n/a ≤ 40		7.3 37.4 23.5 43.1	PSL Y-T-D Regular Inventory Level was higher than the Year-End target in November.
Cost (¢) Per KWh	Production Cost (O&M and Fuel)	PTN PSL	1.94 1.62	1.96 1.60	1.80 1.85	Production Costs (¢/KWh) for PSL were higher than the Y-T-D target in November due primarily to lower than planned generation.
	Total Cost (O&M, Fuel, and Capital Carrying Costs)	Div. Total	4.08		5.22	In November, Division Total Cost (¢/KWh) was higher than the Y-E target due primarily to changes in capital strategy.
Nuclear Division Staffing Levels		FPL LT Contr Total	2237.0 492.2 2729.2		2,044.0 482.0 2,526.0	Total Nuclear Division Staffing Level was below target in November.
Refueling Outage Duration (Days)		PTN3 PTN4 PSL1 PSL2	53 NA NA 53		34 0 0 53	PSL2 is currently refueling. Future Refueling Outage schedules are as follows: PTN3 03/08/97 PSL1 03/26/96 PTN4 03/01/96 PSL2 04/15/97
Lost Time Injuries/Restricted Duty Cases per 200,000 Hours Worked (12 Month Running)		PTN PSL	0.40 0.40		0.22 0.40	No Lost Time Injuries or Restricted Duty Cases were reported in November.

NRC INDICATOR PERFORMANCE OVERVIEW

(Data through Quarter Ending March 31, 1995)



* Not Calculated for Operational Cycle

NOTES:

Plant Self-Trend Short Term: Based on the slope of a linear regression line plotted over each plant's data. Time intervals used in the trends are 4 Quarters for "Operations" and Forced Outages" indicators and 6 quarters for Shutdown" indicators.

Deviations from Peer Group Median Long Term: Comparisons are made of each plant to the performance of its peers over a 12 Quarter time interval.

Peer Groups: PSL 1&2 - Combustion Engineering with core protection calculator plants.

PTN 3&4 - Older Westinghouse 3-Loop.

INPO OVERALL INDICATOR PERFORMANCE OVERVIEW

(November 30, 1995)

Performance Indicators		Unit or Station Values				Industry Median Values*	
		PTN 3	PTN 4	PSL 1	PSL 2	3-Year Distribution (7/92 - 6/95)	1995 Goals
Unit Capability Factor (Unit %, 3-Yr. Distribution Ending November '95)		88.4%	87.2%	77.9%	75.8%	79.7%	80.0%
Unplanned Capability Loss Factor (Unit %, 3-Year Distr. Ending November '95)		2.8%	4.5%	10.9%	11.9%	6.0%	4.5%
Unplanned Automatic Scrams Per 7000 Hours Critical (Per Unit, 3-Year Distribution Ending November '95)		0.3	0.9	1.6	0.7	1.0	1.0
Safety System Performance:	High Pressure Safety Injection System (Per Unit, 3-Year Distribution Ending September '95)	0.008	0.009	0.025	0.010	0.003 - 0.008	0.020
	Auxiliary Feedwater System (Per Unit, 3-Year Distribution Ending September '95)	0.015	0.012	0.010	0.013	0.004 - 0.009	0.025
	Emergency AC Power System (Per Unit, 3-Year Distribution Ending September '95)	0.003	0.003	0.016	0.013	0.009 - 0.014	0.025
Thermal Performance (Ratio of Design to Actual Gross Heat Rate, 1-Yr. Distribution Ending September '95)		99.8%	100.2%	99.1%	98.5%	99.4% **	99.5% **
Collective Radiation Exposure (Man-Rem per unit per year, 3-year running avg. ending November '95)		162	162	220	220	157	185
Volume of Low-level Solid Radioactive Waste (Cubic meters per unit per year, 3-year avg. ending November '95)		39.2	39.2	54.6	54.6	62.0 #	110 #
Chemistry Index (January through November '95) Average		1.19	1.04	1.25	1.17	1.19 **	n/a
Industrial Safety Lost-time Accident Rate (Station rate per 200,000 man-hours worked, year ending November '95 includes Restricted Duty Cases)		0.22		0.40		0.51 **	0.50
Fuel Reliability (Unit microcuries/g, month ending November '95)		1.00E-6	5.20E-6	2.45E-5	n/a	Fuel Defect Reference 5.0E-04***	

NOTE: Shaded area denotes FPL performance is unfavorable to actual industry median.

Source of Industry Data:

- * 1995 Mid-Year Report for Performance Indicators for the U.S. Nuclear Utility Industry (7/92-6/95 Distribution).
- ** 1995 Mid-Year Report for Performance Indicators for the U.S. Nuclear Utility Industry (7/94-6/95 Distribution).
- *** 1995 Mid-Year Report for Performance Indicators for the U.S. Nuclear Utility Industry (4/95-6/95 Distribution).
- # 1995 Mid-Year Report for Performance Indicators for the U.S. Nuclear Utility Industry (1/92-12/94 Distribution).

INPO OVERALL INDICATOR PERFORMANCE OVERVIEW

Discussion of FPL Performance Unfavorable to Industry Median

(November 30, 1995)

UNIT CAPABILITY FACTOR

(3-Years Ending 11/30/95)

St. Lucie Unit 1. The 3-year running Capability Factor for the unit was 77.9%. Capability loss is attributed to the following: Refueling Outage and extension from 3/29/93 to 6/17/93 (7.4%); Hot Leg Valve MV-3480 leak repairs from 3/29/94 to 4/2/94 (0.4%); Main Transformer trip from 6/6/94 to 6/11/94 (0.4%); Quench Tank leak repairs from 2/27/95 to 3/8/95 (0.9%); 1A2 Reactor Coolant Pump seal repairs from 8/2/95 to 8/9/95 (0.7%); inoperable Power Operated Relief Valves from 8/9/95 to 8/17/95 (0.7%); and, inadvertent Containment Spray actuation and clean-up from 8/17/95 to 9/3/95 (1.3%) 1B2 Diesel Generator failure from 9/1/95 to 9/6/95 (0.5%); 1A Diesel Generator Radiator leakage from 9/6/95 to 9/10/95 (0.4%); and, Code Safety Valve repairs and modifications from 9/11/95 to 10/13/95 (2.4%). Other miscellaneous unplanned outages and derates accounted for the remaining 7.0% Unit Capability Factor loss.

St. Lucie Unit 2. Capability Factor for the three years ending 11/30/95 was 75.8%. Capability loss is attributed to the following: Turbine Valve Repairs from 11/24/92 to 12/2/92 (0.3%); Reactor Cooling System Vent Gas Valve Repairs from 12/2/92 to 12/8/92 (0.5%); 2A1 Reactor Coolant Pump repairs due to high vibration from 1/13/93 to 3/2/93 (4.4%); Pressurizer Nozzle leak repairs initiated on 3/2/93 through 4/1/93 (2.8%); dropped CEA's on 5/21/93 (0.5%); Condenser Tube leak repairs from 8/9/93 to 8/11/93 (0.2%); refueling outage from 2/13/94 to 4/22/94 (4.6%); shutdown for auto reactor trip investigation on 4/23/94 (0.3%); and, a refueling outage which began on 10/9/95 (4.8%). Other unplanned outages and power reductions accounted for the remaining 5.8% Unit Capability Factor loss.

UNPLANNED CAPABILITY LOSS FACTOR

(3-Years Ending 11/30/95)

St. Lucie Unit 1. The Unplanned Capability Loss Factor for the three years ending 11/30/95 was 10.9% compared to an industry median of 6.0%. Unplanned outages and power reductions contributing to this performance included: Waterbox cleaning due to jelly fish intrusion from 9/18/93 to 9/29/93 (0.7%); Refueling Outage extension from 6/1/93 to 6/17/93 (1.6%); Hot Leg Valve MV-3480 leak repairs from 3/29/94 to 4/2/94 (0.4%); Main Transformer trip from 6/6/94 to 6/11/94 (0.4%); Quench Tank leak repairs from 2/27/95 to 3/8/95 (0.9%); 1A2 Reactor Coolant Pump seal repairs from 8/2/95 to 8/9/95 (0.7%); inoperable Power Operated Relief Valves from 8/9/95 to 8/17/95 (0.7%); inadvertent Containment Spray actuation and clean-up from 8/17/95 to 9/1/95 (1.3%) 1B2 Diesel Generator failure from 9/1/95 to 9/6/95 (0.5%); 1A Diesel Generator Radiator leakage from 9/6/95 to 9/10/95 (0.4%); and, Code Safety Valve repairs and modifications from 9/11/95 to 10/31/95 (2.4%). Other miscellaneous unplanned outages and derates accounted for the remaining 0.9% Capability Loss.

St. Lucie Unit 2. The Unplanned Capability Loss Factor for the three years ending 11/30/95 was 11.9%. Major unplanned occurrences contributing to this performance included: Turbine Valve Repairs from 11/24/92 to 12/2/92 (0.3%); Reactor Cooling System Vent Gas Valve Repairs from 12/2/92 to 12/8/92 (0.5%); 2A1 Reactor Coolant Pump repairs due to high vibration from 1/13/93 to 3/2/93 (4.4%); Pressurizer Nozzle leak repairs initiated on 3/2/93 through 4/1/93 (2.8%); dropped CEA's on 5/21/93 (0.5%); and, shutdown for auto reactor trip investigation on 4/23/94 (0.3%). Other unplanned outages and power reductions accounted for the remaining 3.1% in Capability Loss.

UNPLANNED AUTOMATIC SCRAMS PER 7000 HOURS CRITICAL (3-Years Ending 11/30/95)

St. Lucie Unit 1. Increased rate for Unit 1 was the result of five auto trips occurring on 3/18/94, 4/3/94, 6/6/94, 10/26/94 and 7/8/95.

HIGH PRESSURE SAFETY INJECTION SYSTEM (3-Years Ending 9/30/95)

Turkey Point Unit 4. Average performance was affected due to last year's on-line replacement of the HPSI pump motors following discovery of cracked rotor bars.

St. Lucie Unit 1. Average performance for the last three years was affected by on line Motor Operated Valve testing in the 3rd Quarter of 1994 and a breaker failure on 2B in the 1st Quarter of 1995.

St. Lucie Unit 2. In the 1st quarter of 1995, average performance was affected as a result of Component Cooling Water (CCW) Heat Exchange cleaning which placed the respective HPSI pump OOS due to lack of dedicated seal cooling.

AUXILIARY FEEDWATER SYSTEM (3-Years Ending 9/30/95)

Turkey Point. Average performance for both units was affected by the B AFW Turbine failure in the 4th quarter of 1994 due to malfunction of the mechanical overspeed trip device, in the 3rd quarter of 1995, performance was affected by: Part 21 repairs on the Trip and Trottle Valves, and Unit 3 outage work.

St. Lucie. Average performance for three-years was affected by failure in the 4th Quarter of 1994 of the 1C AFW Pump Governor. In the 3rd quarter of 1995, performance was attributed to: failure of the AFW PP 2C Steam Admission Valve MV-08-13 to open, a mechanical trip linkage for AFW PP 2C when the Electrical Overspeed Solenoid was energized, and a discrepancy between field wiring and plant wiring drawing for the AFW PP 2B.

EMERGENCY DIESEL GENERATOR SYSTEM (3-Years Ending 9/30/95)

St. Lucie Unit 1. Unit 1's average performance for three-years was the result of a high water jacket temperature trip of 1A EDG and failure of the governor on 1B2 during monthly surveillance run which closed off fuel to the 12 cylinder engine in the 2nd quarter of 1995, and 1B diesel 12 cylinder engine valve failure in the 3rd quarter of 1995.

COLLECTIVE RADIATION EXPOSURE - MAN-REM (3-Years Ending 11/30/95)

Turkey Point. The three-year running average Collective Radiation Exposure level for Turkey Point was 162 Man-Rem per unit which was greater than the industry median of 157 Man-Rem. Site performance was influenced by scheduled refueling outages.

St. Lucie. Collective Radiation Exposure three-year running average level for St. Lucie was 220 Man-Rem per unit which was greater than the industry median of 157 Man-Rem. Site performance was influenced by unplanned and scheduled outages.

CHEMISTRY INDEX PERFORMANCE (12-Months Ending 11/30/95)

St. Lucie Unit 1. Unit 1's Chemistry Index Performance exceeded target as a result of elevated condensate dissolved oxygen.

OPERATIONS HIGHLIGHTS (November 1995)

OPERATING SUMMARY:

Turkey Point Unit 3. Unit 3 essentially operated at a 99.7% Equivalent Availability Factor (EAF) in November. Power losses totaled 2.0 equivalent hours for repair to the 3C Level Control Circuit. Year to date, Unit 3 operated at a 87.6% EAF which is higher than the 80.7% target.

Turkey Point Unit 4. Unit 4 EAF for the month was 96.8%. The unit experienced power losses which totaled 23.1 equivalent hours as a result of Turbine Valve testing (0.9 hours) and Waterbox cleaning (22.2 hours). For the year, the unit achieved an EAF of 97.2%, which is higher than the 95.0% target.

St. Lucie Unit 1. For the month, Unit 1 operated at 90.9% EAF. Power losses totaled 65.2 equivalent hours as a result of: a manual reactor trip and repair to the "B" Main Feedwater Regulating Valve (31.1 hours); repair to the Main Transformer Control Panel which caused loss of cooling to the 1A Main Transformer (2.4 hours); and Waterbox cleaning (31.7 hours). Through November, Unit 1's EAF was 73.1% which is below its target of 95.0%.

St. Lucie Unit 2. Unit 2 remained shutdown through November for a scheduled refueling outage. Year to date, Unit 2 operated at a 79.7% EAF which is below the 84.7% target.

AUTOMATIC REACTOR TRIPS:

No Unplanned Automatic trips occurred in November; year-to-date, two have been reported for St. Lucie.

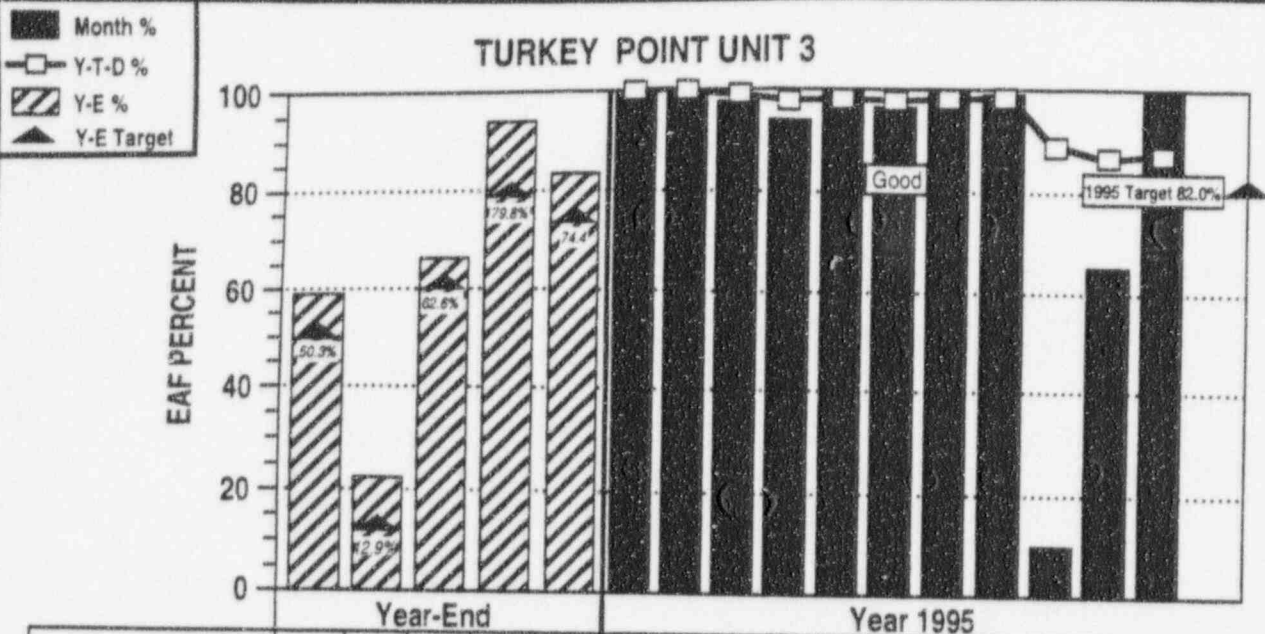
INDUSTRIAL SAFETY:

No Lost Time Accidents or Restricted Duty Cases were reported by Turkey Point or St. Lucie for the month.

Turkey Point had no OSHA Recordable Injuries in November; St. Lucie reported seven for the month.

EQUIVALENT AVAILABILITY

TURKEY POINT UNIT 3



Legend / Period	1990	1991	1992	1993	1994	Jan	Feb	Mar	Apr	May	Jun	Jul	Aug	Sep	Oct	Nov	Dec
EAF% Current Month						100	100	97.7	94.2	100	97.1	100	99.5	10.0	65.1	99.7	
EAF% Y-T-D	58.9	22.3	66.3	94.6	83.5	100	100	99.2	98.0	98.4	98.2	98.4	98.6	88.8	86.4	87.6	

Definitions

Equivalent Availability Factor (EAF) is the ratio of the actual energy production capability to the energy that would be produced operating at full power for the same period expressed as a percent. Equivalent Availability provides an indication of the effectiveness of plant programs and practices in maximizing electrical generation.

$$\text{EAF\%} = \frac{\text{Available Hrs} - (\text{Equivalent Unit Derated Hours} + \text{Equivalent Seasonal Derated Hours})}{\text{Period hours}} \times 100\%$$

Statistical Summary

	Nov	YTD	3 yr Running
EAF - 1995	99.7%	87.6%	88.4%
EAF - 1994	99.5%	83.2%	81.9%
Targets:			
Monthly Y-T-D 1995	80.7%		
Year End - 1995	82.0%		
Year End - 1994	74.4%		
Forecast:			
Year End 1995	88.5%		

Industry Performance

NERC / GADS

1994 (PWR's)	77.8%
1994 (All Types)	74.1%
1990 - 1994 (PWR's)	74.4%
1990 - 1994 (All Types)	71.2%

INPO

July 1994 - July 1995 Median	82.4%
July 1994 - July 1995 Average	77.4%

Performance Summary

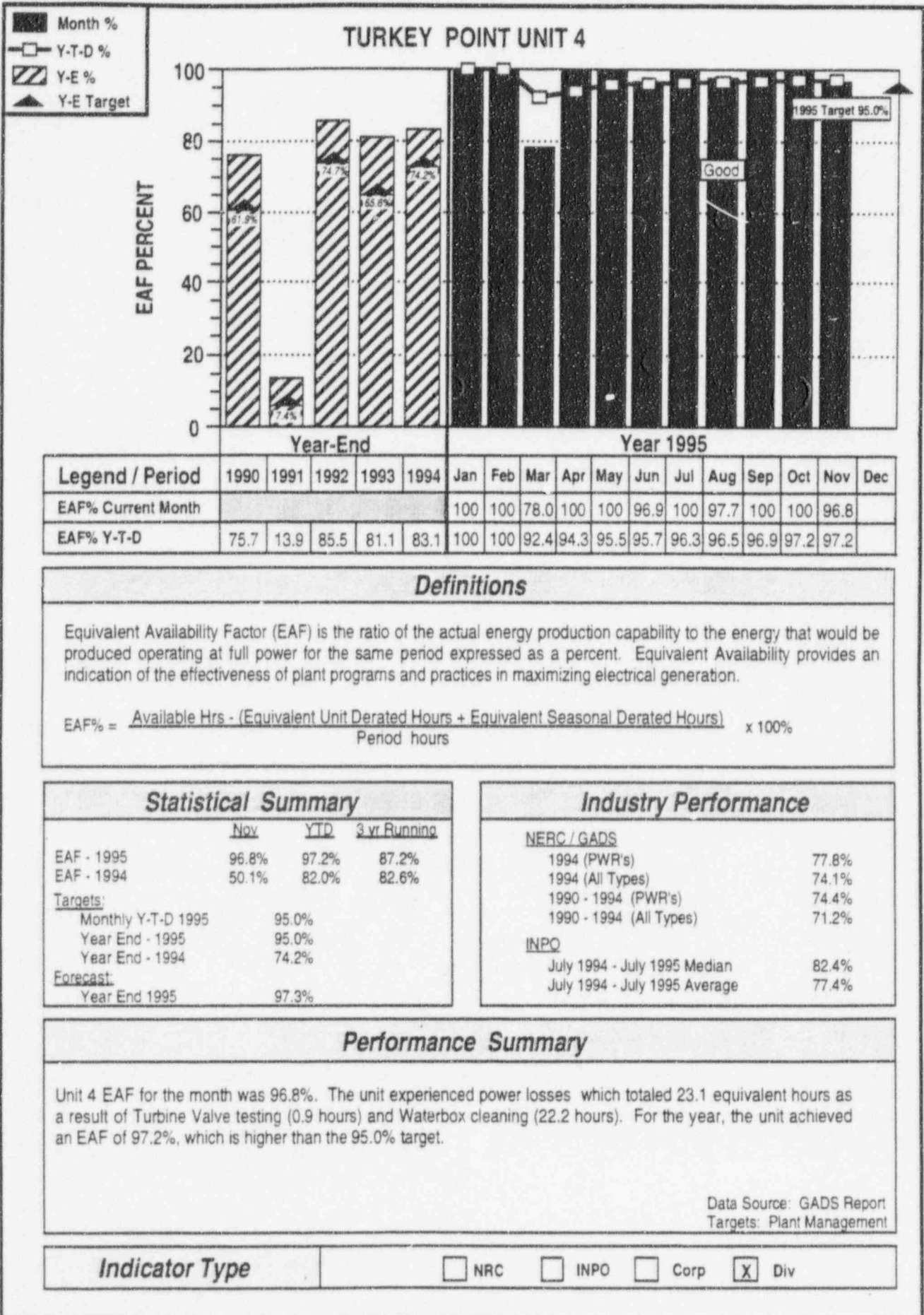
Unit 3 essentially operated at a 99.7% Equivalent Availability Factor (EAF) in November. Power losses totaled 2.0 equivalent hours for repair to the 3C Level Control Circuit. Year to date, Unit 3 operated at a 87.6% EAF which is higher than the 80.7% target.

Data Source: GADS Report
Targets: Plant Management

Indicator Type

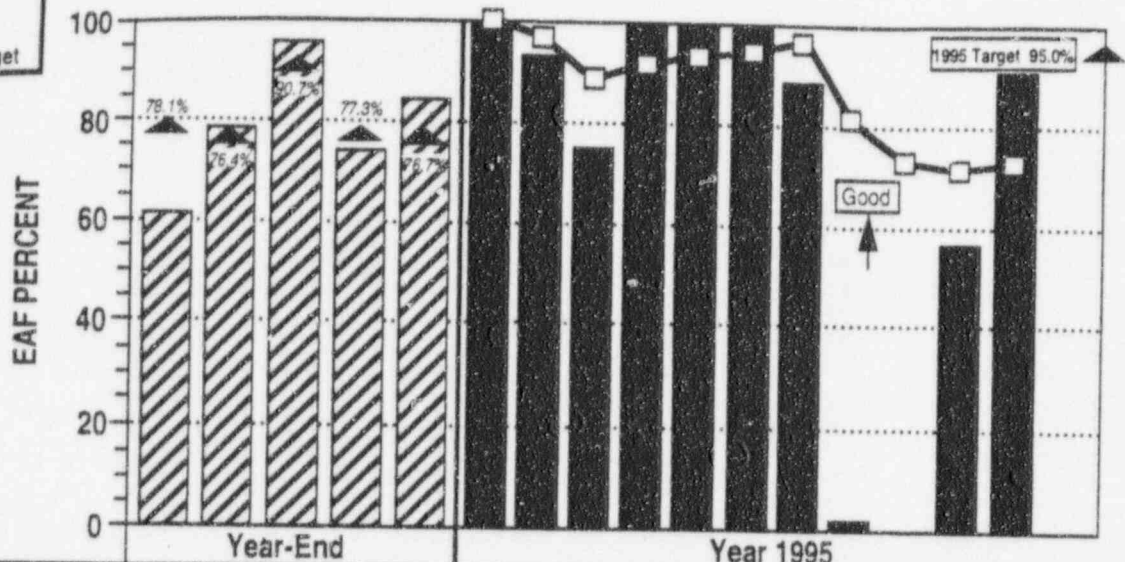
☐ NRC ☐ INPO ☐ Corp ☒ Div

EQUIVALENT AVAILABILITY



EQUIVALENT AVAILABILITY

ST. LUCIE UNIT 1



Legend / Period	1990	1991	1992	1993	1994	Jan	Feb	Mar	Apr	May	Jun	Jul	Aug	Sep	Oct	Nov	Dec
EAF% Current Month						100	92.9	75.0	100	100	99.5	88.6	2.0	0.0	56.6	90.9	
EAF% Y-T-D	61.2	78.5	95.4	74.0	84.5	100	96.6	89.2	91.9	93.5	94.5	96.3	82.0	73.0	71.3	73.1	

Definitions

Equivalent Availability Factor (EAF) is the ratio of the actual energy production capability to the energy that would be produced operating at full power for the same period expressed as a percent. Equivalent Availability provides an indication of the effectiveness of plant programs and practices in maximizing electrical generation.

$$\text{EAF\%} = \frac{\text{Available Hrs} - (\text{Equivalent Unit Derated Hours} + \text{Equivalent Seasonal Derated Hours})}{\text{Period hours}} \times 100\%$$

Statistical Summary

	Nov	YTD	3 yr Running
EAF - 1995	90.9%	73.1%	77.9%
EAF - 1994	94.1%	84.5%	84.6%
Targets:			
Monthly Y-T-D 1995	95.0%		
Year End - 1995	95.0%		
Year End - 1994	76.7%		
Forecast:			
Year End 1995	75.2%		

Industry Performance

NERC / GADS

1994 (PWR's)	77.8%
1994 (All Types)	74.1%
1990 - 1994 (PWR's)	74.4%
1990 - 1994 (All Types)	71.2%

INPO

July 1994 - July 1995 Median	82.4%
July 1994 - July 1995 Average	77.4%

Performance Summary

For the month, Unit 1 operated at 90.9% EAF. Power losses totaled 65.2 equivalent hours as a result of: a manual reactor trip and repair to the 'B' Main Feedwater Regulating Valve (31.1 hours); repair to the Main Transformer Control Panel which caused loss of cooling to the 1A Main Transformer (2.4 hours); and Waterbox cleaning (31.7 hours). Through November, Unit 1's EAF was 73.1% which is below its target of 95.0%.

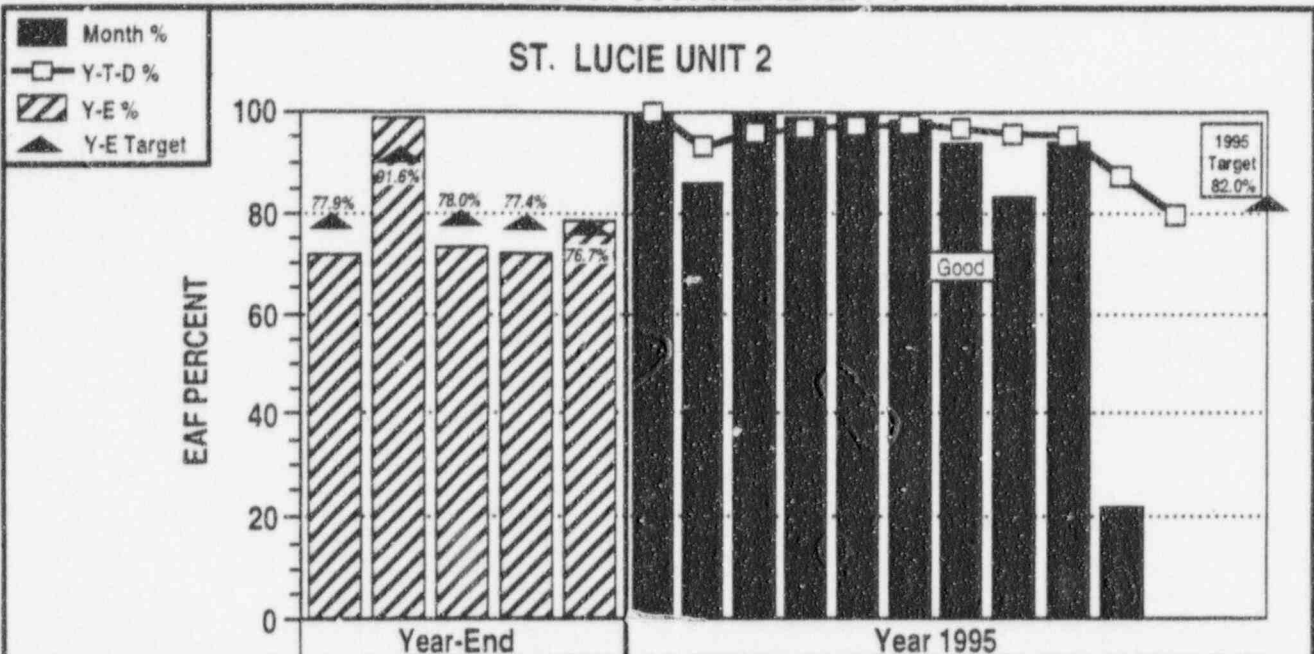
Data Source: GADS Report
Targets: Plant Management

Indicator Type

☐ NRC ☐ INPO ☐ Corp ☒ Div

EQUIVALENT AVAILABILITY

ST. LUCIE UNIT 2



Legend / Period	1990	1991	1992	1993	1994	Jan	Feb	Mar	Apr	May	Jun	Jul	Aug	Sep	Oct	Nov	Dec
EAFF% Current Month						100	85.9	99.8	99.0	99.7	98.6	93.8	83.3	94.0	21.9	0.0	
EAFF% Y-T-D	72.2	98.5	73.2	71.8	77.4	100	93.3	95.5	96.4	97.1	97.3	96.8	95.1	95.0	87.5	79.7	

Definitions

Equivalent Availability Factor (EAF) is the ratio of the actual energy production capability to the energy that would be produced operating at full power for the same period expressed as a percent. Equivalent Availability provides an indication of the effectiveness of plant programs and practices in maximizing electrical generation.

$$EAF\% = \frac{\text{Available Hrs.} - (\text{Equivalent Unit Derated Hours} + \text{Equivalent Seasonal Derated Hours})}{\text{Period hours}} \times 100\%$$

Statistical Summary

	Nov	YTD	3 yr Running
EAFF - 1995	0.0%	79.7%	75.8%
EAFF - 1994	96.5%	77.4%	74.1%
Targets:			
Monthly Y-T-D 1995	84.7%		
Year End - 1995	82.0%		
Year End - 1994	76.7%		
Forecast:			
Year End 1995	72.9%		

Industry Performance

NERC / GADS

1994 (PWR's)	77.8%
1994 (All Types)	74.1%
1990 - 1994 (PWR's)	74.4%
1990 - 1994 (All Types)	71.2%

INPO

July 1994 - July 1995 Median	82.4%
July 1994 - July 1995 Average	77.4%

Performance Summary

Unit 2 remained shutdown through November for a scheduled refueling outage. Year to date, Unit 2 operated at a 79.7% EAF which is below the 84.7% target.

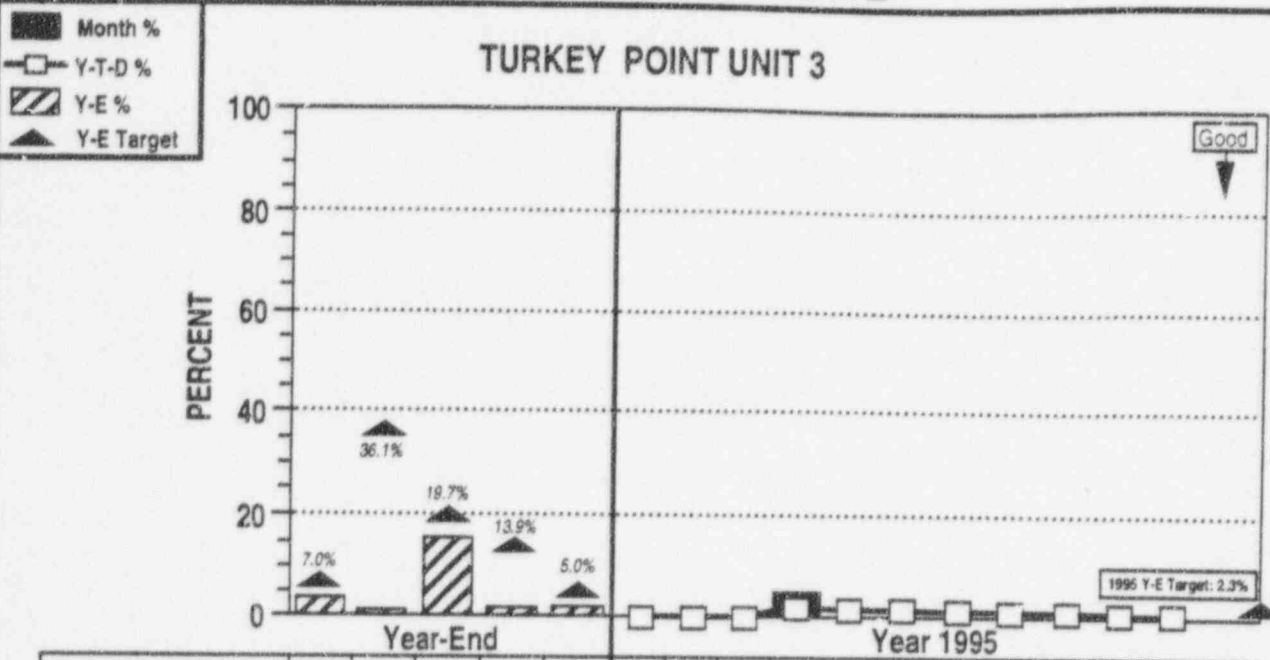
Data Source: GADS Report
Targets: Plant Management

Indicator Type

☐ NRC ☐ INPO ☐ Corp ☒ Div

FORCED OUTAGE RATE

TURKEY POINT UNIT 3



Legend / Period	1990	1991	1992	1993	1994	Jan	Feb	Mar	Apr	May	Jun	Jul	Aug	Sep	Oct	Nov	Dec
FOR% Current Month						0.0	0.0	0.0	5.6	0.0	0.0	0.0	0.0	0.0	0.0	0.0	
FOR% Y-T-D	3.8	1.2	15.6	1.8	1.9	0.0	0.0	0.0	1.4	1.1	0.9	0.8	0.7	0.7	0.6	0.6	

Definitions

Forced outage rate is the percentage of time that the unit was unavailable due to forced events compared to the time planned for electrical generation. Forced events are failures or other unplanned conditions that require removing the unit from service immediately or before the start of the next weekend. Forced events include startup failures and events initiated while the unit is in reserve shutdown (i.e., the unit is available, but not in service).

$$\text{Forced Outage Rate} = \frac{\text{Forced Outage Hours}}{\text{Forced Outage Hours} + \text{Service Hours}} \times 100\%$$

Statistical Summary

	Nov	YTD	12 Mo. Ending
FOR - 1995	0.0%	0.6%	1.5%
FOR - 1994	11.0%	1.9%	1.9%

Targets:

Year End - 1995	2.3%
Year End - 1994	5.0%

Industry Performance

NERC / GADS

1994 (PWR's)	8.1%
1994 (All Types)	10.6%
1990 - 1994 (PWR's)	7.6%
1990 - 1994 (All Types)	10.8%

Performance Summary

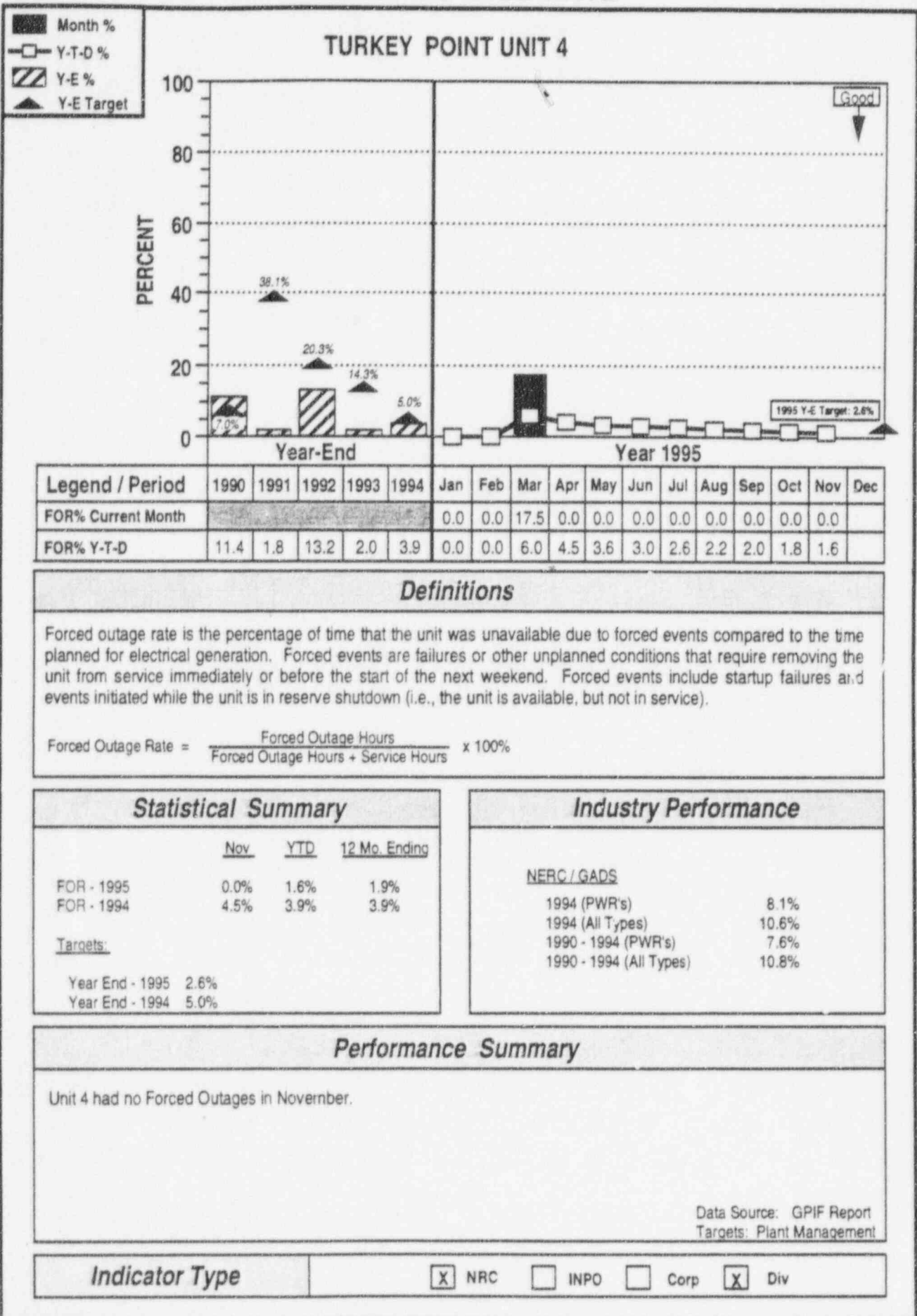
Unit 3 had no forced outages in November.

Data Source: GPIF Report
Targets: Fiant Management

Indicator Type

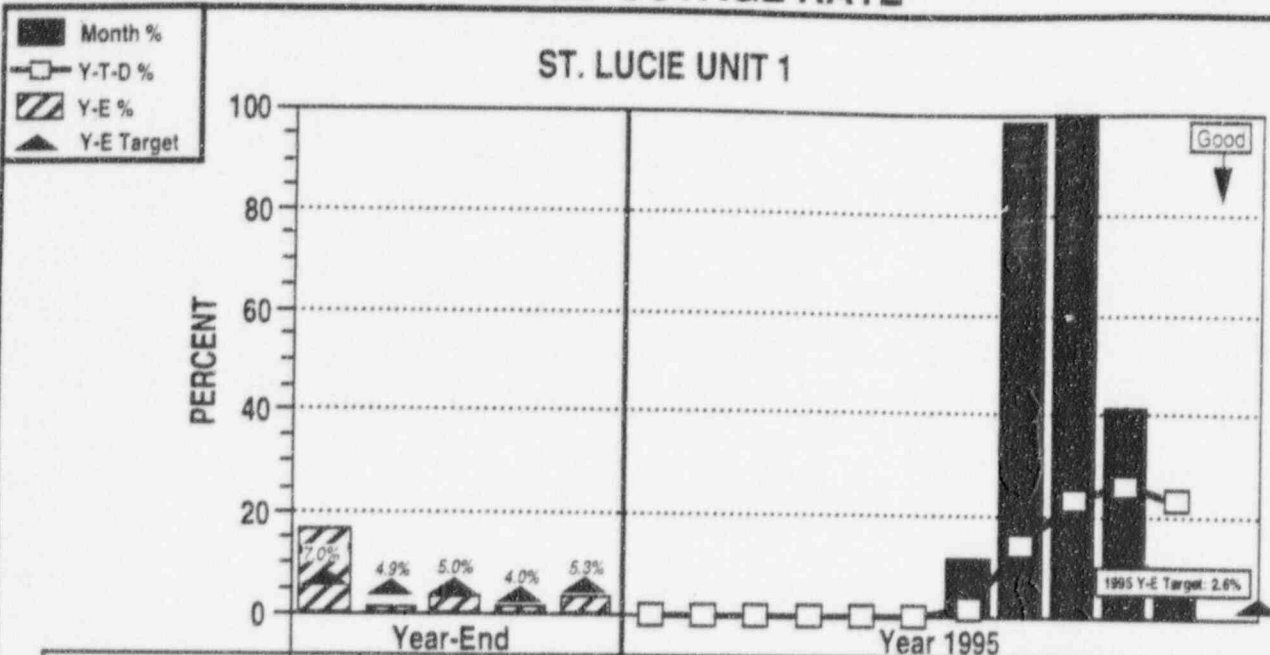
☒ NRC ☐ INPO ☐ Corp ☒ Div

FORCED OUTAGE RATE



FORCED OUTAGE RATE

ST. LUCIE UNIT 1



Definitions

Forced outage rate is the percentage of time that the unit was unavailable due to forced events compared to the time planned for electrical generation. Forced events are failures or other unplanned conditions that require removing the unit from service immediately or before the start of the next weekend. Forced events include startup failures and events initiated while the unit is in reserve shutdown (i.e., the unit is available, but not in service).

$$\text{Forced Outage Rate} = \frac{\text{Forced Outage Hours}}{\text{Forced Outage Hours} + \text{Service Hours}} \times 100\%$$

Statistical Summary

	Nov	YTD	12-Mo. Ending
FOR - 1995	4.3%	23.8%	21.8%
FOR - 1994	0.0%	4.3%	3.9%

Targets:

Year End - 1995 2.6%
Year End - 1994 5.3%

Industry Performance

NERC / GADS

1994 (PWR's)	8.1%
1994 (All Types)	10.6%
1990 - 1994 (PWR's)	7.6%
1990 - 1994 (All Types)	10.8%

Performance Summary

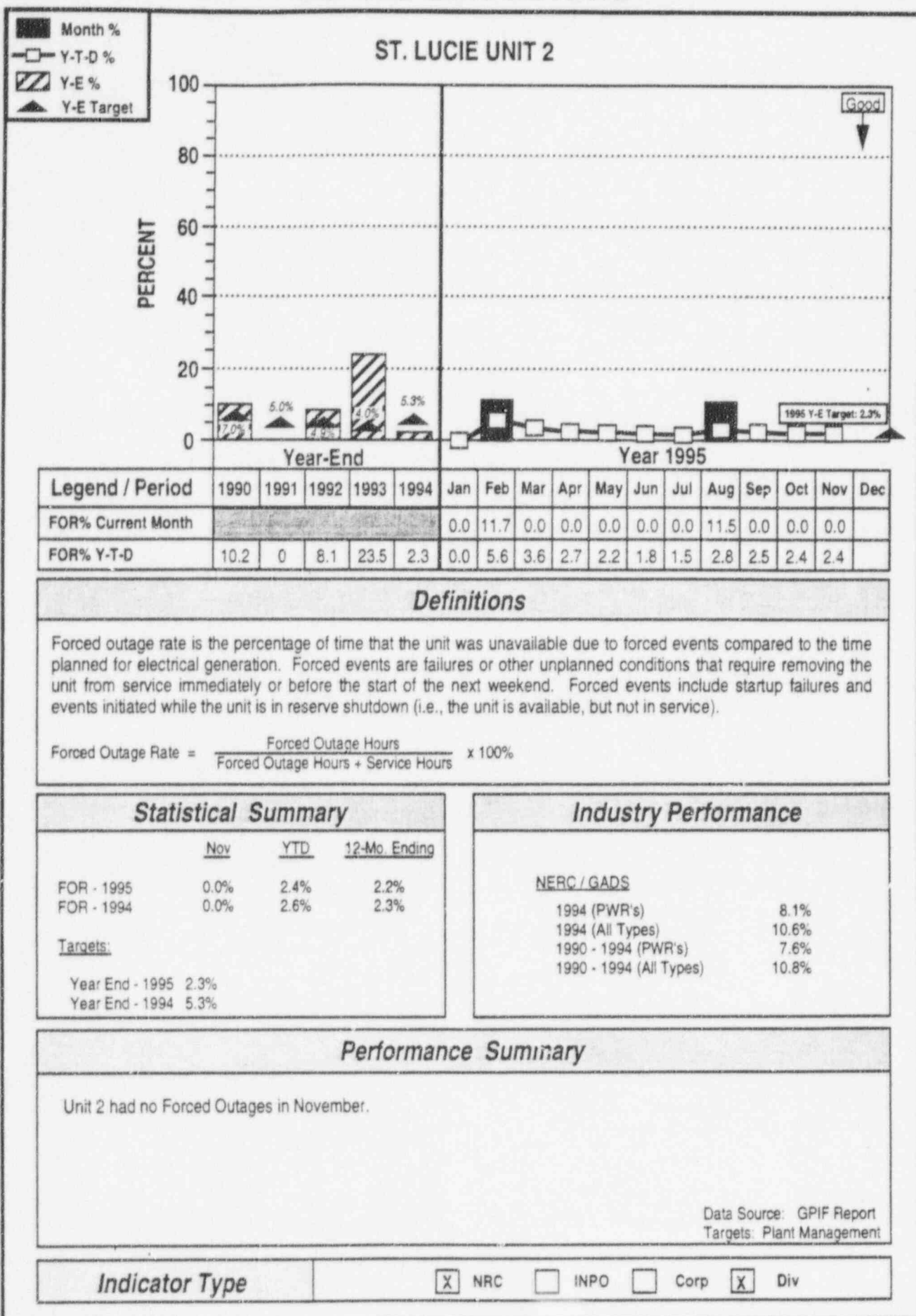
Unit 1 Forced Outage Rate in November was 4.3% as a result of repair to the "B" Main Feedwater Regulating Valve. For the year, Forced Outage Rate was 23.8%.

Data Source: GPIF Report
Targets: Plant Management

Indicator Type

☒ NRC ☐ INPO ☐ Corp ☒ Div

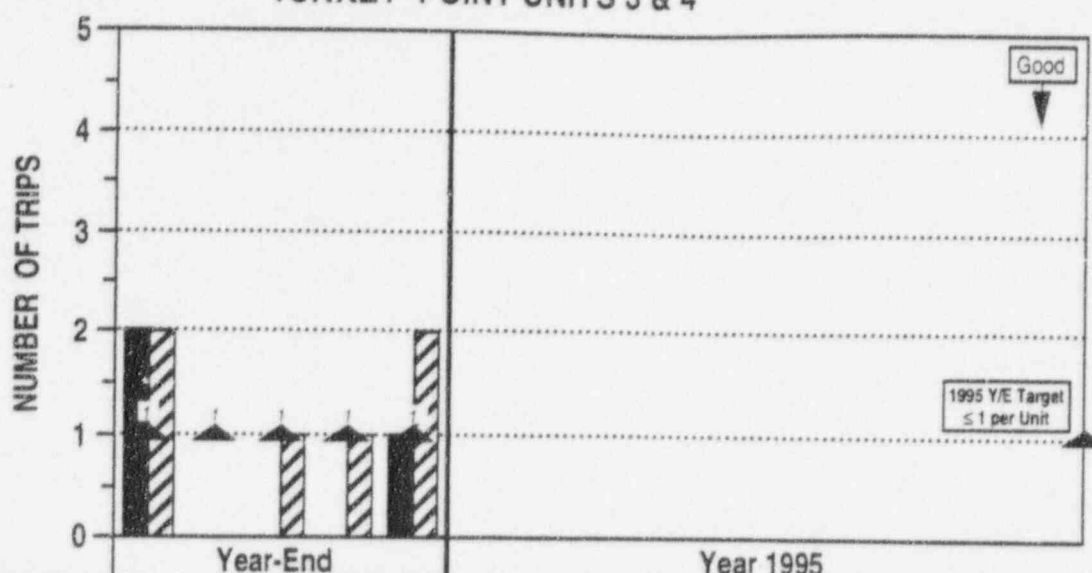
FORCED OUTAGE RATE



UNPLANNED AUTOMATIC TRIPS WHILE CRITICAL

Y-E Target
per Unit

TURKEY POINT UNITS 3 & 4



Legend / Period	1990	1991	1992	1993	1994	Jan	Feb	Mar	Apr	May	Jun	Jul	Aug	Sep	Oct	Nov	Dec
PTN 3	2	0	0	0	1	0	0	0	0	0	0	0	0	0	0	0	
PTN 4	2	0	1	1	2	0	0	0	0	0	0	0	0	0	0	0	

Definitions

An Unplanned Automatic Scram is a non-manual actuation of the reactor protection system that results in a scram signal any time the unit is critical. Scrams that are planned as part of special evaluations or tests are not included in this definition. The indicator provides an indication of success in improving plant safety by reducing the number of undesirable and unplanned thermal-hydraulic and reactivity transients requiring reactor scrams. It also provides an indication of how well a plant is operated and maintained.

Statistical Summary

	Nov	YTD	12-Mo. Ending
PTN 3	0	0	
Trips per 7000 Critical Hours			0.9
PTN 4	0	0	
Trips per 7000 Critical Hours			0.0
1995 Year-End Target per Unit ≤1			

Industry Performance

INPO	Trips/Unit
Trips per 7000 Critical Hours	
3-yr Distribution Median (7/92 - 6/95)	1.0
1994 Median	0.8
1995 Goal	1.0
NRC (1995 1st Qtr Performance Indicator Rpt)	
Qtrly Trips Annualized	1.0
Qtrly Trips Per 7000 Critical Hours	0.8

Performance Summary

Turkey Point had no Unplanned Automatic Trips in November.

Data Source: Monthly Value - Licensee Event Reports

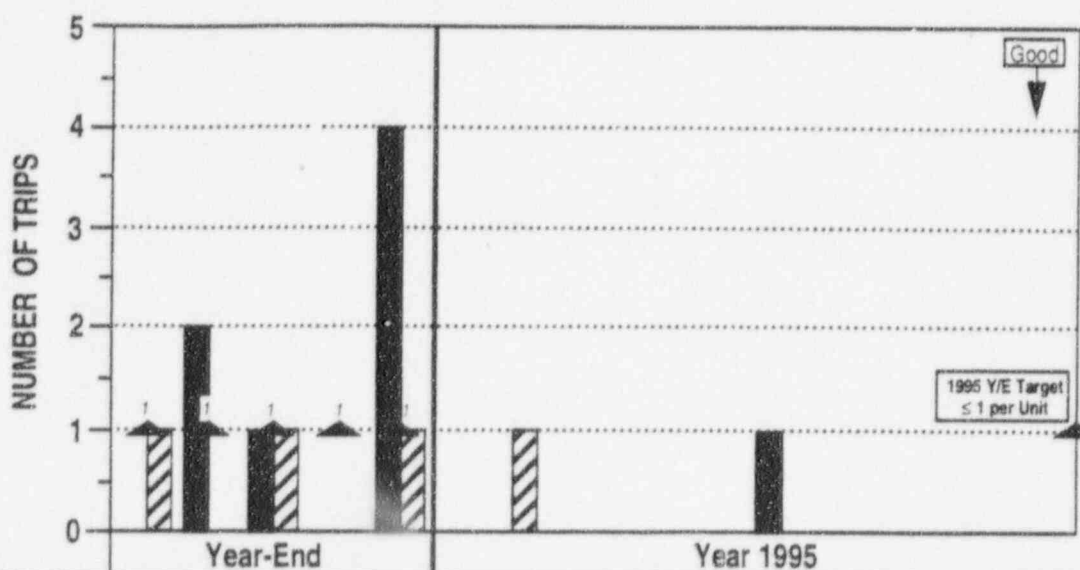
Indicator Type

☒ NRC ☒ INPO ☒ Corp ☒ Div

UNPLANNED AUTOMATIC TRIPS WHILE CRITICAL

Y-E Target
per Unit

ST. LUCIE UNITS 1 & 2



Legend / Period	1990	1991	1992	1993	1994	Jan	Feb	Mar	Apr	May	Jun	Jul	Aug	Sep	Oct	Nov	Dec
PSL 1	0	2	1	0	4	0	0	0	0	0	0	1	0	0	0	0	
PSL 2	1	0	1	0	1	0	1	0	0	0	0	0	0	0	0	0	

Definitions

An Unplanned Automatic Scram is a non-manual actuation of the reactor protection system that results in a scram signal any time the unit is critical. Scrams that are planned as part of special evaluations or tests are not included in this definition. The indicator provides an indication of success in improving plant safety by reducing the number of undesirable and unplanned thermal-hydraulic and reactivity transients requiring reactor scrams. It also provides an indication of how well a plant is operated and maintained.

Statistical Summary

	Nov	YTD	12-Mo. Ending
PSL 1	0	1	
Trips per 7000 Critical Hours			1.0
PSL 2	0	1	
Trips per 7000 Critical Hours			1.0

1995 Year-End Target per Unit ≤1

Industry Performance

INPO	Trips/Unit
Trips per 7000 Critical Hours	
3-yr Distribution Median (7/92 - 6/95)	1.0
1994 Median	0.8
1995 Goal	1.0
NRC (1995 1st Qtr Performance Indicator Rpt)	
Qtrly Trips Annualized	1.0
Qtrly Trips Per 7000 Critical Hours	0.8

Performance Summary

St. Lucie had no Unplanned Automatic Trips in November. Year-to-date, two auto trips have been reported:

7/08/95: St. Lucie Unit 1 tripped during Turbine Overspeed Surveillance Testing due to Personnel Error.

2/21/95: St. Lucie Unit 2 tripped on low A train Steam Generator water level due to failure of the Feedwater Regulating System Level Transmitter.

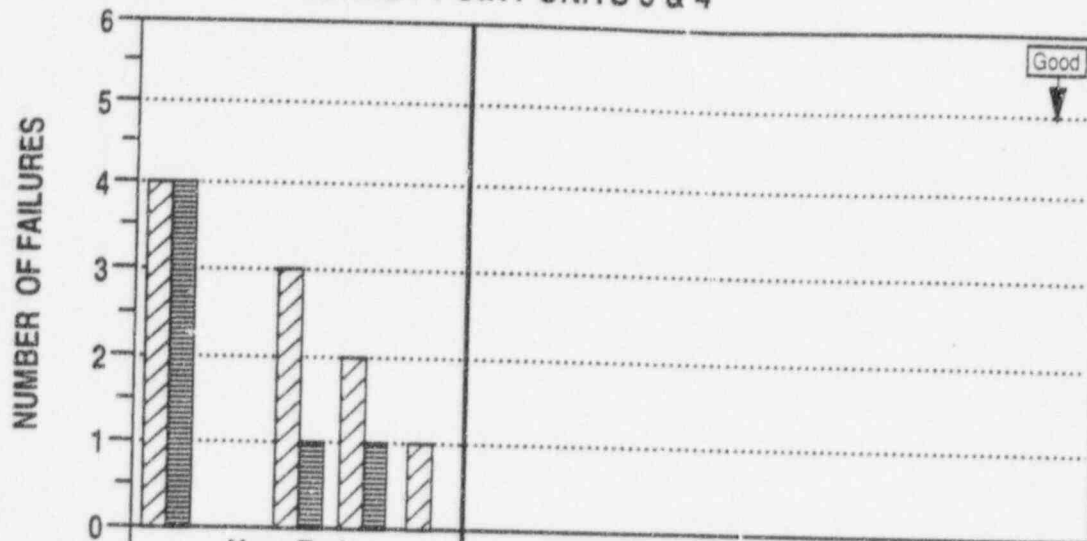
Data Source: Monthly Value - Licensee Event Reports

Indicator Type

☒ NRC ☒ INPO ☒ Corp ☒ Div

SAFETY SYSTEM FAILURES

TURKEY POINT UNITS 3 & 4



Legend / Period		Year-End					Year 1995			
		1990	1991	1992	1993	1994	1st Qtr	2nd Qtr	3rd Qtr	4th Qtr
Quarter	PTN 3						0	0	0	
	PTN 4						0	0	0	
Year-End	PTN 3	4	0	3	2	1				
	PTN 4	4	0	1	1	0				

Definitions

This indicator includes any event or condition that could prevent the fulfillment of the safety function of structures or systems. Twenty-six safety systems, subsystems, and components are monitored for plants that have received an operating license.

Statistical Summary

Safety System Failures	3rd Qtr	YTD	Running 4 Qtrs
1995 - PTN 3	0	0	1
1994 - PTN 3	0	0	0
1995 - PTN 4	0	0	0
1994 - PTN 4	0	0	0

Industry Performance

NRC	1st Qtr	YTD	Running 4 Qtrs
	Qtr		
1995 SSF/Unit	.28	.28	.34

Performance Summary

Turkey Point reported no Safety System Failures in the 3rd Quarter of 1995.

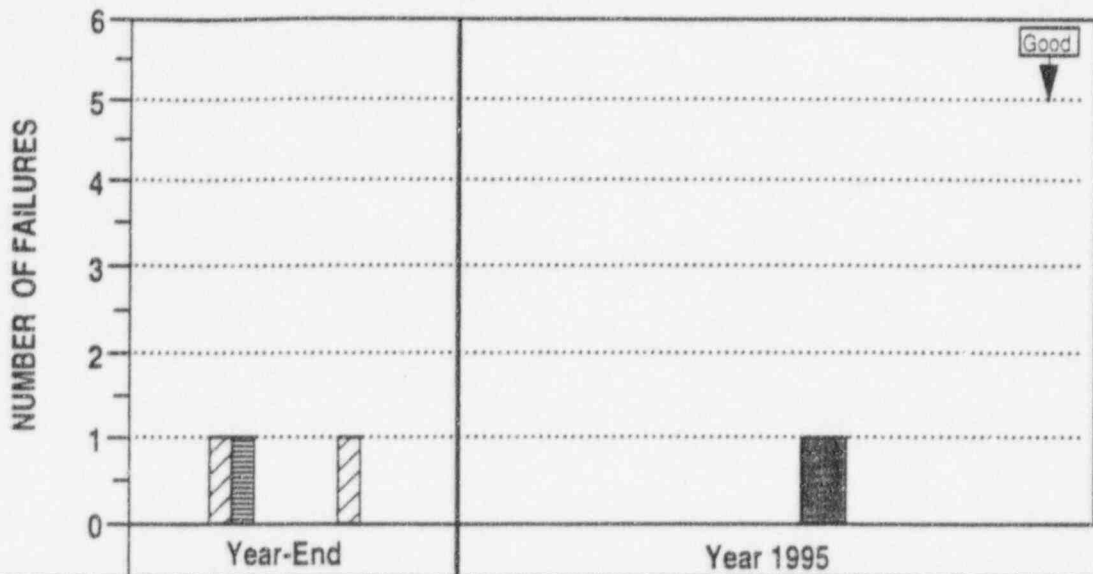
Contact: Tia Hellriegel 246-6791

Indicator Type

☒ NRC ☐ INPO ☐ Corp ☐ Div

SAFETY SYSTEM FAILURES

ST. LUCIE UNITS 1 & 2



Legend / Period	1990	1991	1992	1993	1994	1st Qtr	2nd Qtr	3rd Qtr	4th Qtr
Quarter PSL 1						0	0	1	
Quarter PSL 2						0	0	0	
Year-End PSL 1	0	1	0	1	0				
Year-End PSL 2	0	1	0	0	0				

Definitions

This indicator includes any event or condition that could prevent the fulfillment of the safety function of structures or systems. Twenty-six safety systems, subsystems, and components are monitored for plants that have received an operating license.

Statistical Summary

Safety System Failures	3rd Qtr	YTD	Running 4 Qtrs
1995 - PSL 1	1	1	1
1994 - PSL 1	0	0	0
1995 - PSL 2	0	0	0
1994 - PSL 2	0	0	0

Industry Performance

	1st Qtr	YTD	Running 4 Qtrs
<u>NRC</u>			
1995 SSF/Unit	.28	.28	.34

Performance Summary

St. Lucie reported one Safety System Failure in the 3rd Quarter of 1995:

LER #335-95-005 dated 8/22/95 - Reactor Coolant System Power Operated Relieve Valves (and Low Temperature Overpressure Protection System) Inoperable due to Personnel Error.

Contact: Mike Snyder 467-7036

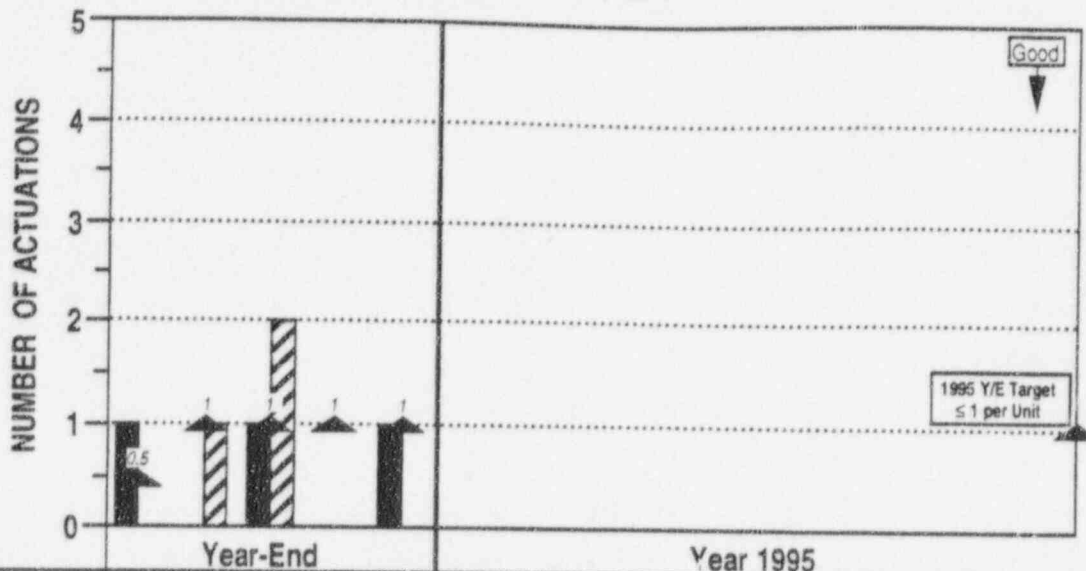
Indicator Type

☒ NRC ☐ INPO ☐ Corp ☐ Div

UNPLANNED SAFETY SYSTEM ACTUATIONS

Y-E Target
per Unit

TURKEY POINT UNIT 3 & 4



Legend / Period	1990	1991	1992	1993	1994	Jan	Feb	Mar	Apr	May	Jun	Jul	Aug	Sep	Oct	Nov	Dec
PTN 3	1	0	1	0	1	0	0	0	0	0	0	0	0	0	0	0	0
PTN 4	0	1	2	0	0	0	0	0	0	0	0	0	0	0	0	0	0

Definitions

Unplanned Safety System Actuations include actual and inadvertent actuations of Emergency Core Cooling Systems, as well as actuations of emergency AC power systems due to loss of power to a vital bus (INPO definition).

Statistical Summary

	Nov	YTD
PTN 3	0	0
PTN 4	0	0

Target - 1995 Year-End Target per Unit: ≤1

Industry Performance

NRC	1st Qtr
SSA 1st Qtr 1995 Average	0.10
SSA Running 4 Qtr Avg	0.41

Note: Running 4 Qtr Average Value annualized

Performance Summary

Turkey Point reported no Safety System Actuations in November.

Data Source: Monthly Value - Licensee Event Reports
Contact: Jim Knorr, 246-6757

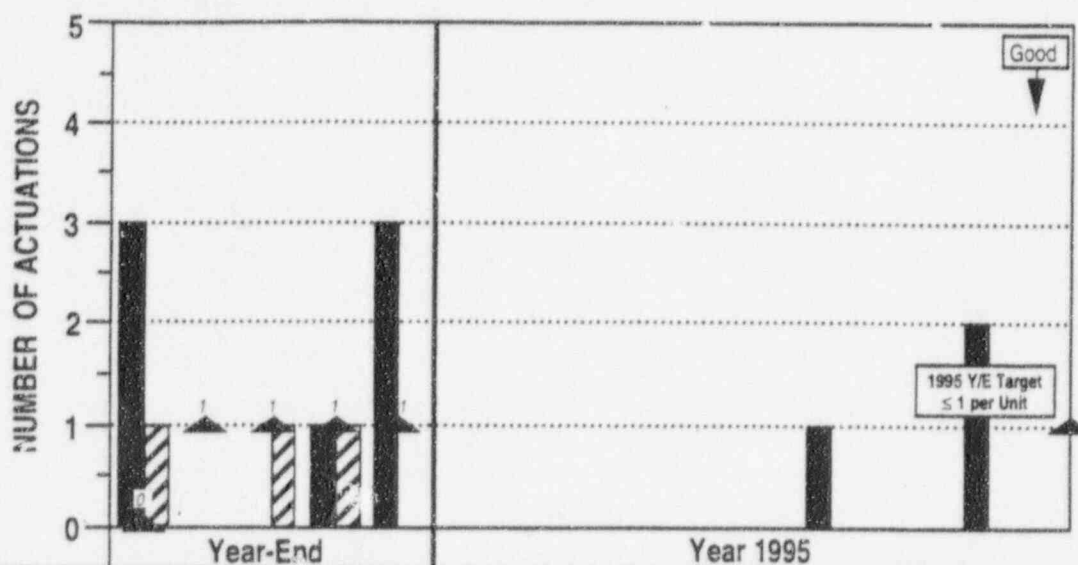
Indicator Type

☒ NRC ☒ INPO ☐ Core ☐ Div

UNPLANNED SAFETY SYSTEM ACTUATIONS

Y-E Target
per Unit

ST. LUCIE UNITS 1 & 2



Legend / Period	1990	1991	1992	1993	1994	Jan	Feb	Mar	Apr	May	Jun	Jul	Aug	Sep	Oct	Nov	Dec
PSL 1	3	0	0	1	3	0	0	0	0	0	0	0	1	0	0	2	
PSL 2	1	0	1	1	0	0	0	0	0	0	0	0	0	0	0	0	

Definitions

Unplanned Safety System Actuations include actual and inadvertent actuations of Emergency Core Cooling Systems, as well as actuations of emergency AC power systems due to loss of power to a vital bus (INPO definition).

Statistical Summary

	Nov	YTD
PSL 1	2	3
PSL 2	0	0

Target - 1995 Year-End Target per Unit: ≤1

Industry Performance

NRC	1st Qtr
SSA 1st Qtr 1995 Average	0.10
SSA Running 4 Qtr Avg	0.41

Note: Running 4 Qtr Average Value annualized

Performance Summary

Unit 2 reported two Safety System Actuations on November 16th:

- (1) A manual trip was initiated due to a failure of the 1B Main Feed Regulating Valve Controller. Reactor Protection System (RPS) actuated as designed following the manual trip.
- (2) Auxiliary Feedwater Actuation System also actuated as designed on low Steam Generator level.

Data Source: Monthly Value - Licensee Event Reports
Contact: Mike Snyder, 465-3363

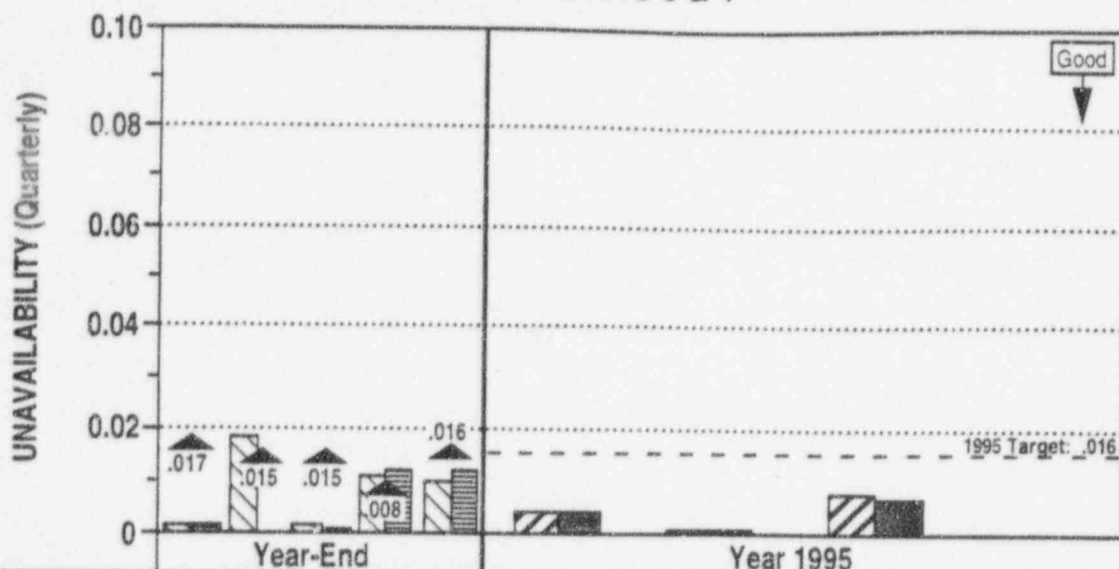
Indicator Type

☒ NRC ☒ INPO ☐ Corp ☐ Div

SAFETY SYSTEM PERFORMANCE

SAFETY INJECTION SYSTEM

TURKEY POINT UNITS 3 & 4



Legend / Period	1990	1991	1992	1993	1994	1st Qtr	2nd Qtr	3rd Qtr	4th Qtr
Quarter						.004	.001	.008	
						.004	.001	.007	
Year-End	.002	.019	.002	.011	.010				
	.002	.000	.001	.012	.012				

Definitions

This Safety System Performance indicator monitors the readiness of the Safety Injection (SI) System to respond to off-normal events or accidents. The indicator is determined from the unavailabilities, due to all causes, of the components in the system during a time period, divided by the number of trains in the system. The definition is further explained: component unavailability is the ratio of the hours the component was unavailable (unavailable hours) to the hours the system was required to be available for service. Data is reported on a quarterly basis.

$$\text{Unavailability} = \frac{(\text{Known Unavailable Hours}) + (\text{Estimated Unavailable Hours})}{(\text{Hours System Required}) \times (\text{Number of Trains})}$$

Statistical Summary

	<u>3rd Qtr</u>
PTN 3 1995	.008
PTN 3 1994	.016
PTN 4 1995	.007
PTN 4 1994	.017
Target:	
1995 Year-End Target	.016
1994 Year-End Target	.016

Industry Performance

<u>INPO</u>	<u>Unavailability</u>
July 1992 - June 1995 Median (PWR)	0.003 - 0.008
1995 Goal	0.020

Performance Summary

Turkey Point Units 3 & 4 Safety Injection System performance for the 3rd Quarter of 1995 was below the Year-End target and Industry Median.

Data Source: Plant Data Books
Contact: Carlos Melchor 246-6964

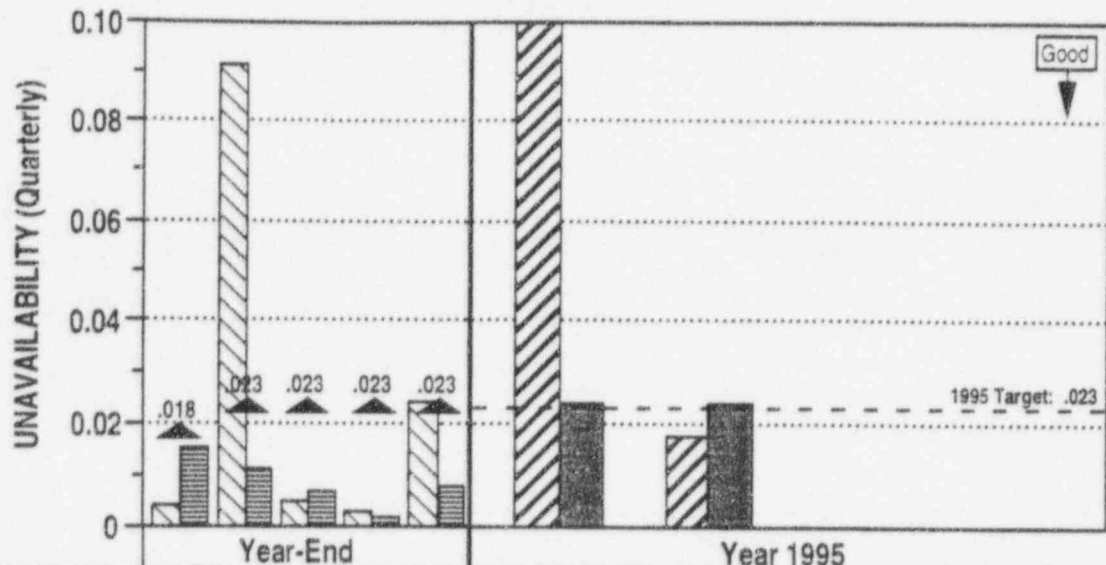
Indicator Type

☐ NRC ☒ INPO ☐ Corp ☐ Div

SAFETY SYSTEM PERFORMANCE

HIGH PRESSURE SAFETY INJECTION SYSTEM

ST. LUCIE UNITS 1 & 2



Legend / Period	1990	1991	1992	1993	1994	1st Qtr	2nd Qtr	3rd Qtr	4th Qtr
Quarter						.141	.018	.000	
						.024	.024	.000	
Year-End									
	.004	.091	.005	.002	.024				
	.016	.011	.007	.003	.008				

Definitions

This Safety System Performance indicator monitors the readiness of the High Pressure Safety Injection (HPSI) System to respond to off-normal events or accidents. The indicator is determined from the unavailabilities, due to all causes, of the components in the system during a time period, divided by the number of trains in the system. The definition is further explained: component unavailability is the ratio of the hours the component was unavailable (unavailable hours) to the hours the system was required to be available for service. Data is reported on a quarterly basis.

$$\text{Unavailability} = \frac{(\text{Known Unavailable Hours}) + (\text{Estimated Unavailable Hours})}{(\text{Hours System Required}) \times (\text{Number of Trains})}$$

Statistical Summary

	<u>3rd Qtr</u>
PSL1 1995	.000
PSL1 1994	.058
PSL2 1995	.000
PSL2 1994	.010
<u>Target</u>	
1995 Year-End Target	.023
1994 Year-End Target	.023

Industry Performance

<u>INPO</u>	<u>Unavailability</u>
July 1992 - June 1995 Median (PWR)	0.003 - 0.008
1995 Goal	0.020

Performance Summary

St. Lucie Units 1 and 2 High Pressure Safety Injection System performance for the 3rd Quarter of 1995 was below the Year-End target and Industry Median.

Contact: Catherine Swiatek 467-7081

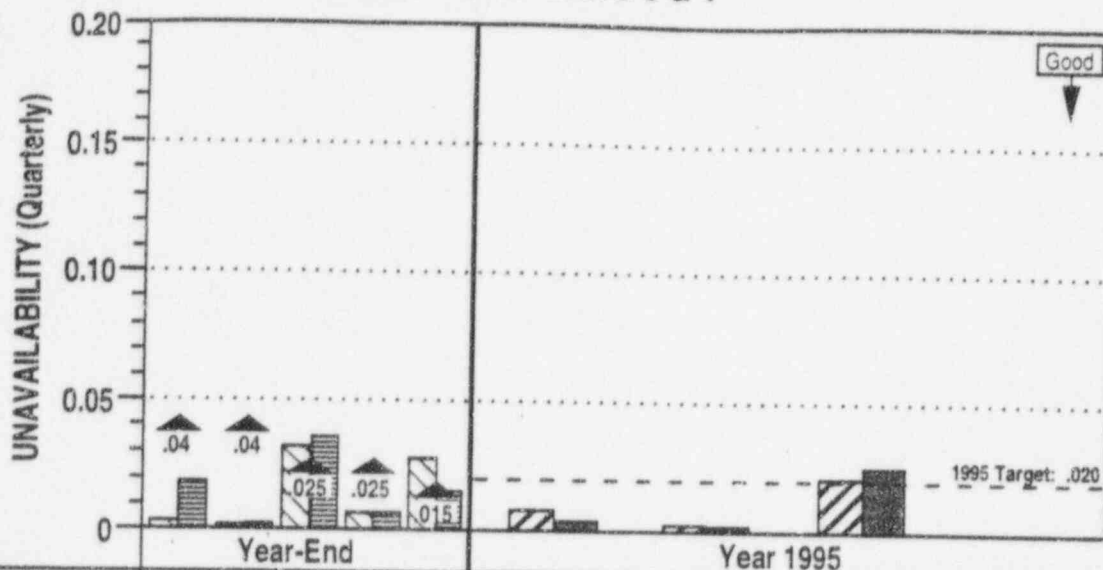
Indicator Type

☐ NRC ☒ INPO ☐ Corp ☐ Div

SAFETY SYSTEM PERFORMANCE

AUXILIARY FEEDWATER SYSTEM

TURKEY POINT UNITS 3 & 4



Legend / Period	1990	1991	1992	1993	1994	1st Qtr	2nd Qtr	3rd Qtr	4th Qtr
Quarter									
PTN 3						.009	.003	.021	
PTN 4						.004	.002	.025	
Year-End									
PTN 3	.004	.002	.031	.007	.028				
PTN 4	.018	.003	.035	.007	.015				

Definitions

This Safety System Performance indicator monitors the readiness of the Auxiliary Feedwater (AFW) System to respond to off-normal events or accidents. The indicator is determined from the unavailabilities, due to all causes, of the components in the system during a time period, divided by the number of trains in the system. The definition is further explained: component unavailability is the ratio of hours the component was unavailable (unavailable hours) to the hours the system was required to be available for service. Data is reported on a quarterly basis.

$$\text{AFW Unavailability} = \frac{(\text{Known Unavailable Hours}) + (\text{Estimated Unavailable Hours})}{(\text{Hours System Required}) \times (\text{Number of Trains})}$$

Statistical Summary

	3rdQtr
PTN 3 1995	.021
PTN 3 1994	.012
PTN 4 1995	.025
PTN 4 1994	.013
Target:	
1995 Year-End Target	.020
1994 Year-End Target	.015

Industry Performance

INPO	Unavailability
July 1992 - June 1995 Median (PWR)	0.004 - 0.009
1995 Goal	0.025

Performance Summary

Turkey Point Units 3 & 4 Auxiliary Feedwater System performance for the 3rd Quarter of 1995 was higher than the Year-End target and Industry Median as a result of: Part 21 repairs on the Trip and Trottle Valves and Unit 3 outage work.

Data Source: Plant Data Books
Contact: W. Raasch, 246-6527

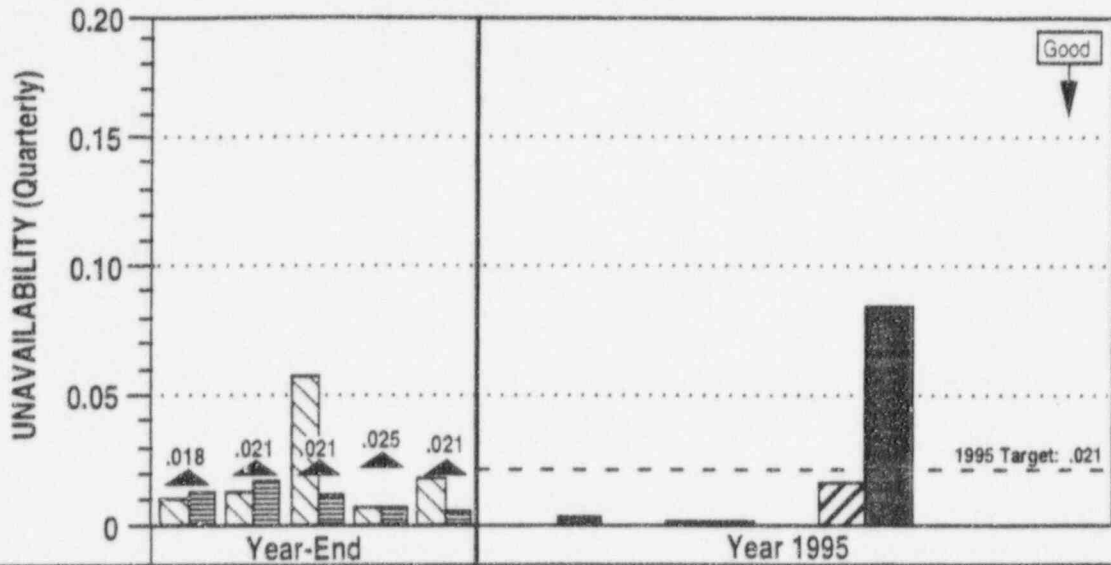
Indicator Type

☐ NRC
 ☒ INPO
 ☐ Corp
 ☐ Div

SAFETY SYSTEM PERFORMANCE

AUXILIARY FEEDWATER SYSTEM

ST. LUCIE UNITS 1 & 2



Legend / Period	1990	1991	1992	1993	1994	1st Qtr	2nd Qtr	3rd Qtr	4th Qtr
Quarter									
PSL 1						.000	.001	.016	
PSL 2						.003	.001	.084	
Year-End									
PSL 1	.010	.013	.057	.007	.018				
PSL 2	.013	.017	.011	.007	.006				

Definitions

This Safety System Performance indicator monitors the readiness of the Auxiliary Feedwater (AFW) System to respond to off-normal events or accidents. The indicator is determined from the unavailabilities, due to all causes, of the components in the system during a time period, divided by the number of trains in the system. The definition is further explained: component unavailability is the ratio of hours the component was unavailable (unavailable hours) to the hours the system was required to be available for service. Data is reported on a quarterly basis.

$$\text{AFW Unavailability} = \frac{(\text{Known Unavailable Hours}) + (\text{Estimated Unavailable Hours})}{(\text{Hours System Required}) \times (\text{Number of Trains})}$$

Statistical Summary

	3rd Qtr
PSL1 1995	.016
PSL1 1994	.023
PSL2 1995	.084
PSL2 1994	.004
Target:	
1995 Year-End Target	.021
1994 Year-End Target	.021

Industry Performance

<u>INPO</u>	<u>Unavailability</u>
July 1992 - June 1995 Median (PWR)	0.004 - 0.009
1995 Goal	0.025

Performance Summary

Performance of the Unit 1 Auxiliary Feedwater System for the 3rd Quarter of 1995 was below the Year-End target but higher than the Industry Median.

St. Lucie Unit 2 Auxiliary Feedwater System Unavailability performance for the 3rd Quarter of 1995 exceeded the Year-End target and Industry Median as a result of three events: (1) a discovered discrepancy between field wiring and the plant wiring drawing for the AFW PP 2B; (2) when starting the AFW PP 2C with steam admission valve MV-08-13, it did not open; and (3) mechanical trip linkage for the AFW PP 2C tripped when the Electrical Overspeed Solenoid was energized.

Contact: W. C. Green 467-7038

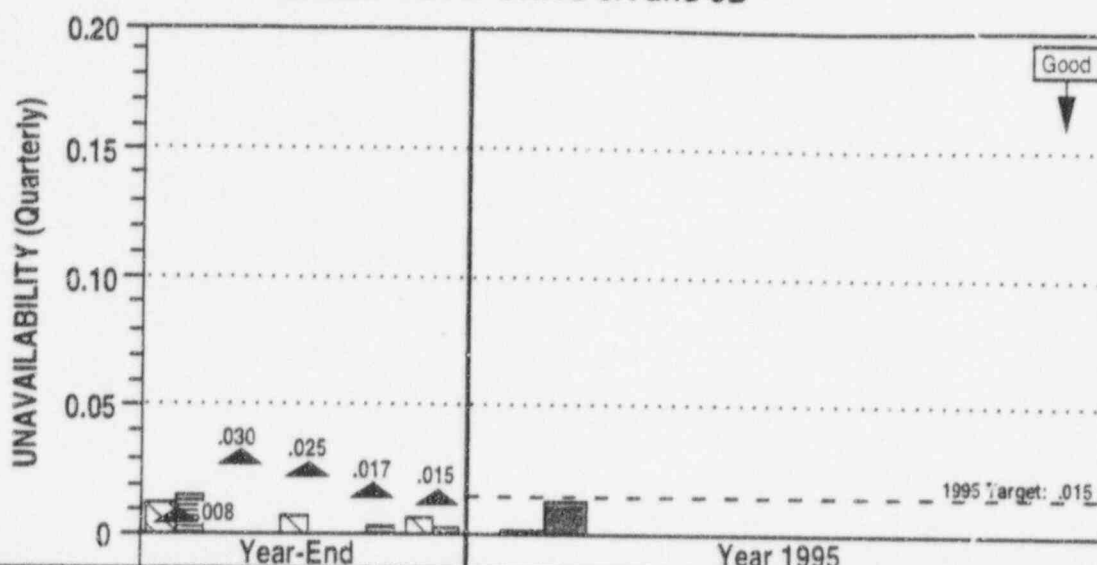
Indicator Type

☐ NRC ☒ INPO ☐ Corp ☐ Div

SAFETY SYSTEM PERFORMANCE

EMERGENCY DIESEL GENERATOR UNAVAILABILITY

TURKEY POINT UNITS 3A and 3B



Legend / Period		Year-End					Year 1995			
Quarter		1990	1991	1992	1993	1994	1st Qtr	2nd Qtr	3rd Qtr	4th Qtr
Year-End	3A	.013	.000	.007	.000	.007				
	3B	.016	.000	.000	.002	.003				

Definitions

Emergency Diesel Generator (EDG) performance is the sum of the unavailabilities, due to all causes, of the emergency generator in the system during a time period, divided by the number of hours the system was required for service. The emergency AC power system is monitored at the train level. That is, unavailable hours are recorded only when the emergency generator train is unavailable to deliver emergency AC power. The indicator value is calculated by averaging the individual EDG Unavailability values for the two EDG's at the site.

Value for each EDG = $\frac{(\text{Known Unavailable Hours}) + (\text{Estimated Unavailable Hours})}{(\text{Hours System Required})}$

Statistical Summary

SITE TOTALS	3rd Qtr	4 Qtr Moving Avg
EDG Unavailability 1995	.000	.004
EDG Unavailability 1994	.006	.004
Target:		
1995 Year-End Target	.015	
1994 Year-End Target	.015	

Industry Performance

INPO	Unavailability
July 1992 - June 1995 Median (PWR)	0.009 - 0.014
1995 Goal	0.025

Performance Summary

Turkey Point 3A and 3B Emergency Diesel Generator Unavailability performance for the 3rd Quarter of 1995 was below the Year-End target and Industry Median.

Data Source: Plant Data Books
Contact: D. Tomaszewski, 246-6158

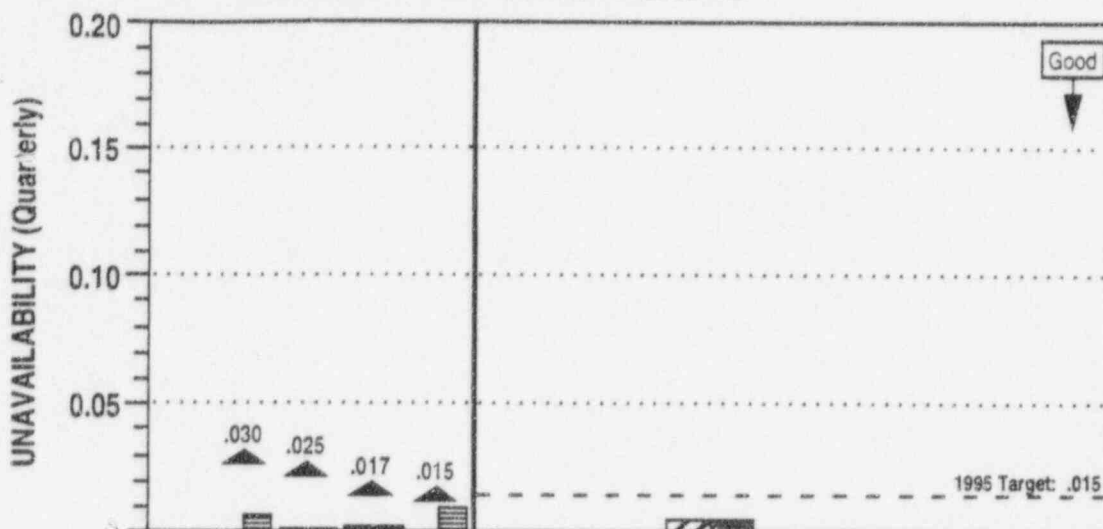
Indicator Type

☐ NRC
 ☒ INPO
 ☐ Corp
 ☐ Div

SAFETY SYSTEM PERFORMANCE

EMERGENCY DIESEL GENERATOR UNAVAILABILITY

TURKEY POINT UNITS 4A and 4B



Legend / Period		Year-End					Year 1995			
Quarter	4A	1990	1991	1992	1993	1994	1st Qtr	2nd Qtr	3rd Qtr	4th Qtr
	4B						.000	.005	.000	
Year-End	4A	n/a	.000	.001	.002	.000				
	4B	n/a	.008	.001	.002	.010				

Definitions

Emergency Diesel Generator (EDG) performance is the sum of the unavailabilities, due to all causes, of the emergency generator in the system during a time period, divided by the number of hours the system was required for service. The emergency AC power system is monitored at the train level. That is, unavailable hours are recorded only when the emergency generator train is unavailable to deliver emergency AC power. The indicator value is calculated by averaging the individual EDG Unavailability values for the two EDG's on each site.

$$\text{Value for each EDG} = \frac{(\text{Known Unavailable Hours}) + (\text{Estimated Unavailable Hours})}{(\text{Hours System Required})}$$

Statistical Summary

SITE TOTALS	3rd Qtr	4 Qtr Moving Avg
EDG Unavailability 1995	.000	.002
EDG Unavailability 1994	.005	.006
Target:		
1995 Year-End Target	.015	
1994 Year-End Target	.015	

Industry Performance

INPO	Unavailability
July 1992 - June 1995 Median (PWR)	0.009 - 0.014
1995 Goal	0.025

Performance Summary

Turkey Point 4A and 4B Emergency Diesel Generator Unavailability performance for the 3rd Quarter of 1995 was below the Year-End target and Industry Median.

Data Source: Plant Data Books
Contact: D. Tomaszewski, 246-6158

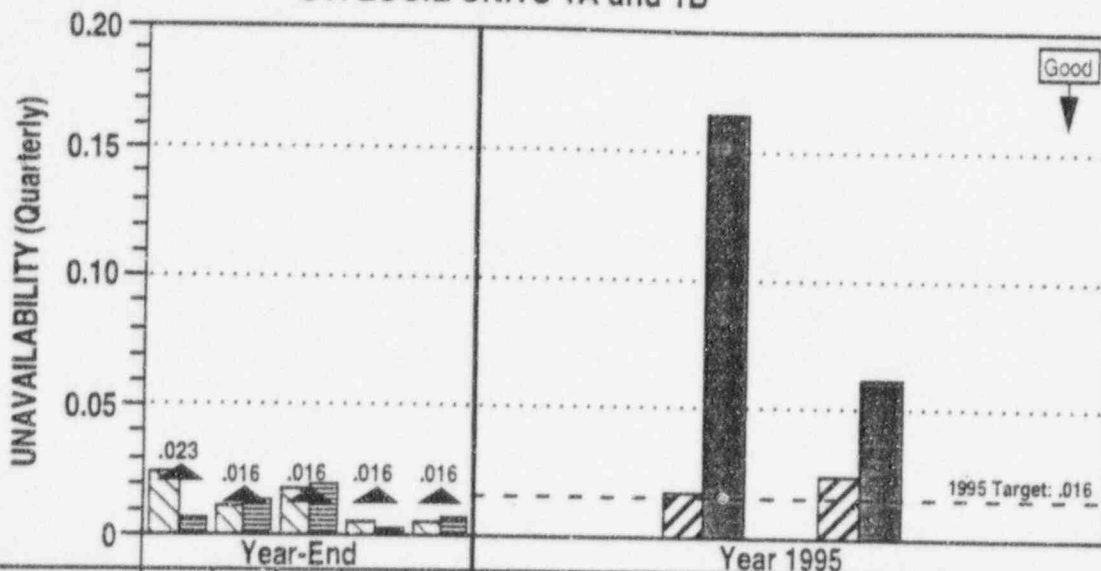
Indicator Type

☐ NRC
 ☒ INPO
 ☒ Corp
 ☒ Div

SAFETY SYSTEM PERFORMANCE

EMERGENCY DIESEL GENERATOR UNAVAILABILITY

ST. LUCIE UNITS 1A and 1B



Legend / Period		1990	1991	1992	1993	1994	1st Qtr	2nd Qtr	3rd Qtr	4th Qtr
Quarter	1A						.000	.018	.025	
	1B						.000	.166	.062	
Year-End	1A	.025	.011	.018	.006	.006				
	1B	.007	.014	.020	.008	.008				

Definitions

Emergency Diesel Generator (EDG) performance is the sum of the unavailabilities, due to all causes, of the emergency generator in the system during a time period, divided by the number of hours the system was required for service. The emergency AC power system is monitored at the train level. That is, unavailable hours are recorded only when the emergency generator train is unavailable to deliver emergency AC power. The indicator value is calculated by averaging the individual EDG Unavailability values for the two EDG's on each unit.

$$\text{Value for each EDG} = \frac{(\text{Known Unavailable Hours}) + (\text{Estimated Unavailable Hours})}{(\text{Hours System Required})}$$

Statistical Summary

Site Totals	3rd Qtr	4 Qtr Moving Avg
EDG Unavailability 1995	.044	.035
EDG Unavailability 1994	.012	.009
Target:		
1995 Year-End Target	.016	
1994 Year-End Target	.016	

Industry Performance

INPO	Unavailability
July 1992 - June 1995 Median (PWR)	0.009 - 0.014
1995 Goal	0.021

Performance Summary

St. Lucie Emergency Diesel Generator Unavailability performance for the 3rd Quarter of 1995 was above the Year-End target and Industry Median as a result of a failure of 1B Diesel 12 cylinder Engine Valve.

Contact: R. L. Kulavich 467-7080

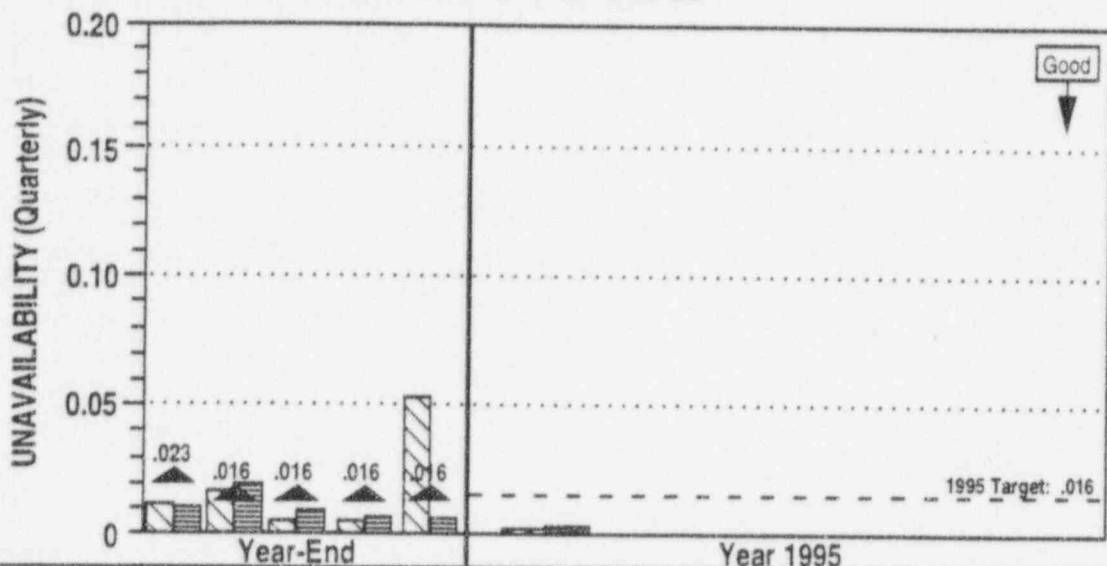
Indicator Type

☐ NRC
 ☒ INPO
 ☐ Corp
 ☐ Div

SAFETY SYSTEM PERFORMANCE

EMERGENCY DIESEL GENERATOR UNAVAILABILITY

ST. LUCIE UNITS 2A and 2B



Legend / Period		1990	1991	1992	1993	1994	1st Qtr	2nd Qtr	3rd Qtr	4th Qtr
Quarter	2A						.002	.000	.000	
	2B						.003	.000	.000	
Year-End	2A	.012	.017	.005	.005	.053				
	2B	.011	.020	.010	.007	.006				

Definitions

Emergency Diesel Generator (EDG) performance is the sum of the unavailabilities, due to all causes, of the emergency generator in the system during a time period, divided by the number of hours the system was required for service. The emergency AC power system is monitored at the train level. That is, unavailable hours are recorded only when the emergency generator train is unavailable to deliver emergency AC power. The indicator value is calculated by averaging the individual EDG Unavailability values for the two EDG's on each unit.

$$\text{Value for each EDG} = \frac{(\text{Known Unavailable Hours}) + (\text{Estimated Unavailable Hours})}{(\text{Hours System Required})}$$

Statistical Summary

SITE TOTALS	3rd Qtr	4 Qtr Moving Avg
EDG Unavailability 1995	.000	.026
EDG Unavailability 1994	.008	.009
Target:		
1995 Year-End Target	.016	
1994 Year-End Target	.016	

Industry Performance

INPO	Unavailability
July 1992 - June 1995 Median (PWR)	0.009 - 0.014
1995 Goal	0.025

Performance Summary

Units 2A and 2B Emergency Diesel Generator Unavailability performance for the 3rd Quarter of 1995 was below the Year-End target and Industry Median.

Contact: R. L. Kulavich 467-7080

Indicator Type

☐ NRC

☒ INPO

☐ Corp

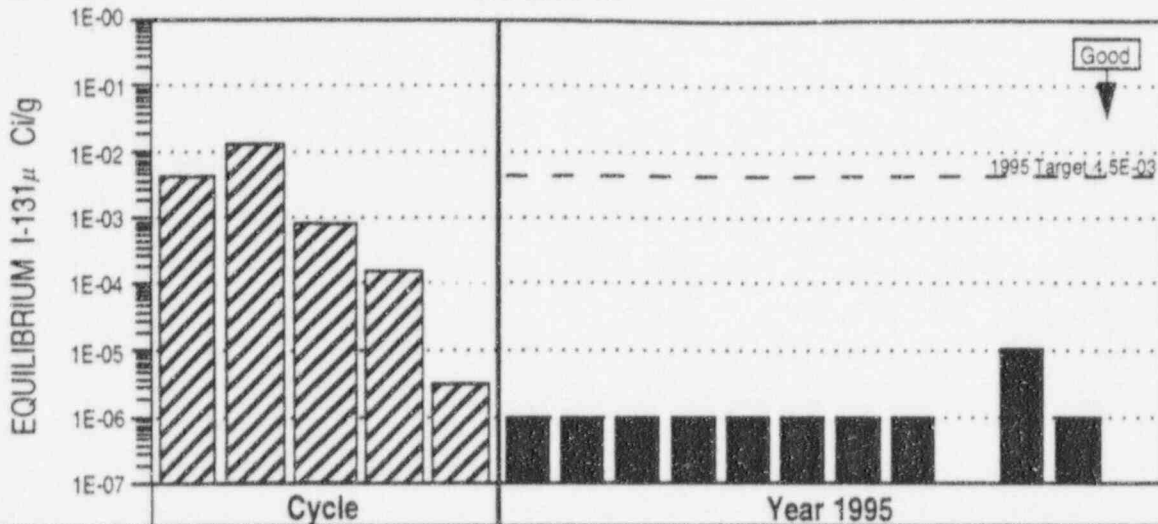
☐ Div

FUEL RELIABILITY

TURKEY POINT UNIT 3

CYCLE 15

☒ Month
☒ Past Cycle
 X Outage
 XX No Steady State



Legend / Period	10	11	12	13	14	Jan	Feb	Mar	Apr	May	Jun	Jul	Aug	Sep	Oct	Nov	Dec
Month						1.00E-6	1.00E-6	1.00E-6	1.00E-6	1.00E-6	1.00E-6	1.00E-6	1.00E-6	X	1.01E-5	1.00E-6	
	4.48E-03	1.44E-02	8.70E-04	1.75E-04	3.16E-06												

Definitions

Fuel reliability is indicated by the monthly average of Iodine-131 activity calculated from samples taken following at least three days of steady-state operation above 85% power with power variations less than 5%. The Iodine-131 data is adjusted by correcting for the activity of the naturally occurring fissionable material in the zircalloy cladding, and by normalizing to the plant's letdown purification rate. This indicator for Fuel Reliability is a measure of cladding integrity (or fuel failure). All measurements are in I-131 micro-curies per gram.

Note: If a calculated monthly value for a unit is less than 1.0E-6 microcuries per gram, the value is replaced by 1.0E6 microcuries per gram.

Statistical Summary

Nov
 Fuel Reliability - 1995 1.00E-06
 1995 Year End Target $\leq 4.5E-03$

Industry Performance

INPO

1994 Fuel Defect Reference Threshold (PWR) 5.00 E-04
 1994 Median 7.36 E-05
 1994 Best Quartile 3.00 E-06

Performance Summary

Unit 3 Fuel Reliability was 1.00E-06.

Data Source: Nuclear Fuel / JPN
 Contact: Modesto Jimenez 694-3323

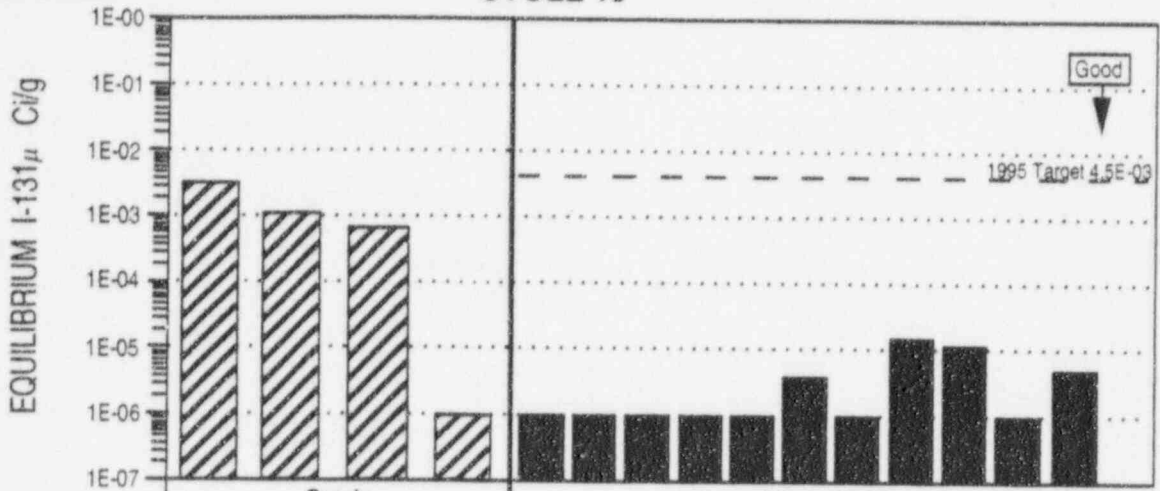
Indicator Type

☐ NRC ☒ INPO ☐ Corp ☐ Div

FUEL RELIABILITY

TURKEY POINT UNIT 4 CYCLE 15

☒ Month
☒ Past Cycle
 X Outage
 XX No Steady State



Legend / Period	11	12	13	14	Jan	Feb	Mar	Apr	May	Jun	Jul	Aug	Sep	Oct	Nov	Dec
Month					1.00E-6	1.00E-6	1.00E-6	1.00E-6	1.00E-6	4.27E-6	1.00E-6	1.72E-5	1.22E-5	1.00E-6	5.20E-6	
	3.24E-03	1.08E-03	7.43E-04	1.00E-06												

Definitions

Fuel reliability is indicated by the monthly average of Iodine-131 activity calculated from samples taken following at least three days of steady-state operation above 85% power with power variations less than 5%. The Iodine-131 data is adjusted by correcting for the activity of the naturally occurring fissionable material in the zircalloy cladding, and by normalizing to the plant's letdown purification rate. This indicator for Fuel Reliability is a measure of cladding integrity (or fuel failure). All measurements are in I-131 micro-curies per gram.

Note: If a calculated monthly value for a unit is less than 1.0E-6 microcuries per gram, the value is replaced by 1.0E6 microcuries per gram.

Statistical Summary

Nov
 Fuel Reliability - 1995 5.20E-06

1995 Year End Target $\leq 4.5E-03$

Industry Performance

INPO

1994 Fuel Defect Reference Threshold (PWR) 5.00 E-04
 1994 Median 7.36 E-05
 1994 Best Quartile 3.00 E-06

Performance Summary

Unit 4's Fuel Reliability continues to indicate zero fuel defects in Cycle 15.

Data Source: Nuclear Fuel / JPN
 Contact: Modesto Jimenez 694-3323

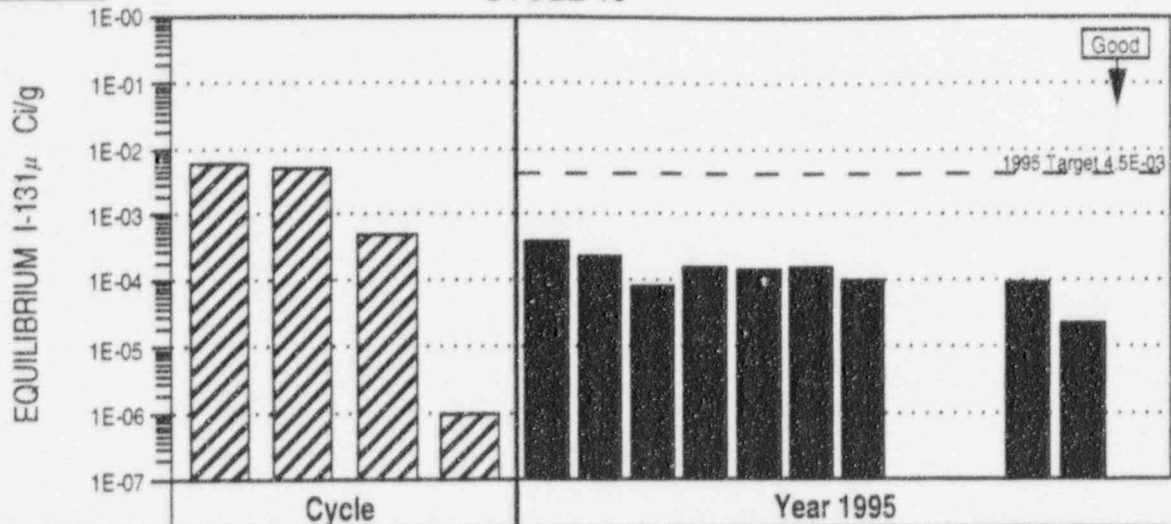
Indicator Type

☐ NRC ☒ INPO ☐ Corp ☐ Div

FUEL RELIABILITY

ST. LUCIE UNIT 1 CYCLE 13

☒ Month
☒ Past Cycle
 X Outage
 XX No Steady State



Legend / Period	Cycle				Year 1995											
	9	10	11	12	Jan	Feb	Mar	Apr	May	Jun	Jul	Aug	Sep	Oct	Nov	Dec
Month					3.41E-4	2.85E-4	8.38E-5	1.83E-4	1.62E-4	1.78E-4	9.80E-5	n/a	n/a	9.45E-5	2.45E-5	
	6.54E-03	5.41E-03	5.17E-04	1.00E-06												

Definitions

Fuel reliability is indicated by the monthly average of Iodine-131 activity calculated from samples taken following at least three days of steady-state operation above 85% power with power variations less than 5%. The Iodine-131 data is adjusted by correcting for the activity of the naturally occurring fissionable material in the zircalloy cladding, and by normalizing to the plant's letdown purification rate. This indicator for Fuel Reliability is a measure of cladding integrity (or fuel failure). All measurements are in I-131 micro-curies per gram.

Note: If a calculated monthly value for a unit is less than 1.0E-6 microcuries per gram, the value is replaced by 1.0E6 microcuries per gram.

Statistical Summary

Fuel Reliability - 1995 Nov 2.45E-05

1995 Year End Target $\leq 4.5E-03$

Industry Performance

INPO

1994 Fuel Defect Reference Threshold (PWR) 5.00 E-04
 1994 Median 7.36 E-05
 1994 Best Quartile 3.00 E-06

Performance Summary

Reactor Coolant System radioisotopic data and spiking iodine following a reactor shutdown at the end of February and in July indicate the presence of one third-cycle failed fuel rod in the current Cycle 13.

Data Source: Nuclear Fuel / JPN
 Contact: Modesto Jimenez 694-3323

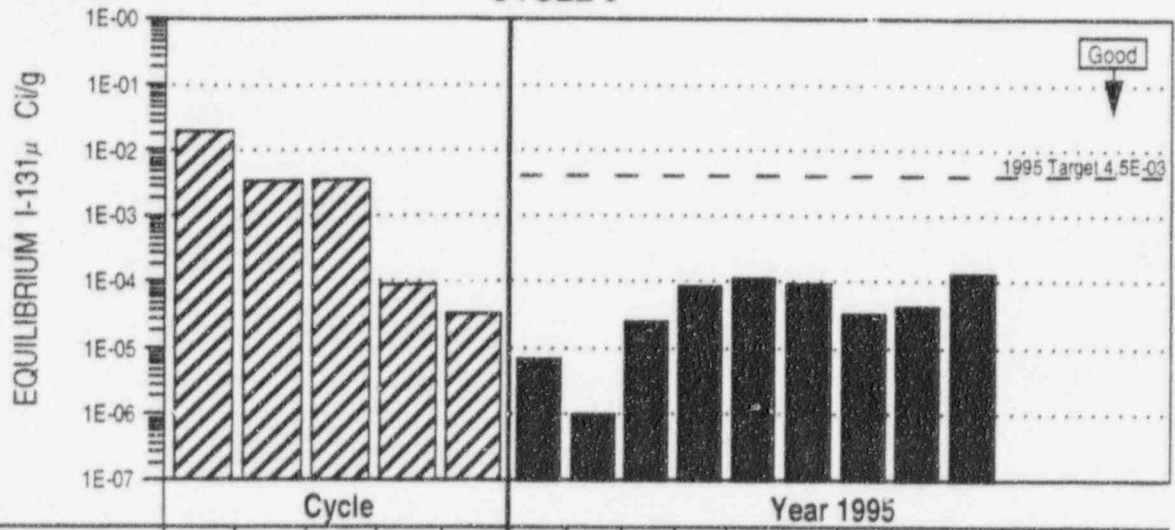
Indicator Type

☐ NRC ☒ INPO ☐ Corp ☐ Div

FUEL RELIABILITY

☐ Month
☒ Past Cycle
 X Outage
 XX No Steady State

ST. LUCIE UNIT 2 CYCLE 9



Legend / Period	Cycle					Year 1995											
	4	5	6	7	8	Jan	Feb	Mar	Apr	May	Jun	Jul	Aug	Sep	Oct	Nov	Dec
Month						7.65E-6	1.00E-6	2.87E-5	8.89E-5	1.11E-4	9.25E-5	3.50E-5	4.18E-5	1.38E-4	X	X	
	2.01E-02	3.71E-03	3.92E-03	9.5E-05	3.62E-05												

Definitions

Fuel reliability is indicated by the monthly average of Iodine-131 activity calculated from samples taken following at least three days of steady-state operation above 85% power with power variations less than 5%. The Iodine-131 data is adjusted by correcting for the activity of the naturally occurring fissionable material in the zircalloy cladding, and by normalizing to the plant's letdown purification rate. This indicator for Fuel Reliability is a measure of cladding integrity (or fuel failure). All measurements are in I-131 micro-curies per gram.

Note: If a calculated monthly value for a unit is less than 1.0E-6 microcuries per gram, the value is replaced by 1.0E6 microcuries per gram.

Statistical Summary

Fuel Reliability - 1995
 1995 Year End Target ≤4.5E-03
 Outage

Industry Performance

INPO

1994 Fuel Defect Reference Threshold (PWR) 5.00 E-04
 1994 Median 7.36 E-05
 1994 Best Quartile 3.00 E-06

Performance Summary



Unit 2 remained shutdown for refueling through November.

Data Source: Nuclear Fuel / JPN
 Contact: Modesto Jimenez 694-3323

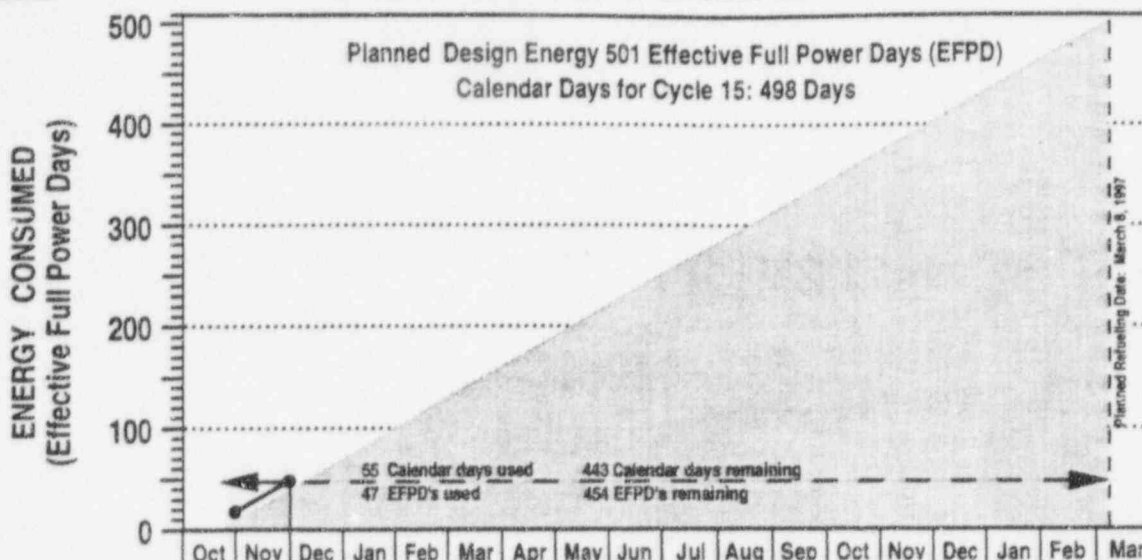
Indicator Type

☐ NRC ☒ INPO ☐ Corp ☐ Div

FUEL UTILIZATION

Planned 
Actual 

TURKEY POINT UNIT 3 CYCLE 15



	Oct	Nov	Dec	Jan	Feb	Mar	Apr	May	Jun	Jul	Aug	Sep	Oct	Nov	Dec	Jan	Feb	Mar
Planned Burnup (EFPD)	5	35	66	98	127	158	188	219	249	281	312	342	373	403	435	466	494	501
Actual Burnup (EFPD)	18	47																
Variance (EFPD) (+/-)	+13	+12																

Definitions

Fuel Utilization plots the amount of nuclear energy used during the current fuel cycle. The amount of nuclear energy is expressed in effective full power days (EFPD). One EFPD is the equivalent to operating the reactor at maximum thermal rating (2200 megawatts thermal) for a 24 hour period. Planned Energy is compared to Actual Energy Used during the cycle. Fuel utilization is directly related to plant performance. The significance of variance EFPD (+/-) is the difference between planned and actual, this can also be used to project longer or shorter fuel cycle runs.

Fuel Cycle Operating Assumptions

In accordance with the April 26, 1995 Approved Operating Schedule (AOS), Unit 3, Cycle 15 is scheduled to run 498 calendar days (October 27, 1995 to March 8, 1997) with design energy to run 501 (EFPD) days.

Note: Due to the 34-day outage, startup began on October 7, 1995, 20 days earlier than planned.

$$\text{Actual Energy} + \text{Calendar Days} = \text{Fuel Utilization Factor}$$

Performance Summary

Unit 3 achieved a Fuel Utilization Factor of 97.0% and operated for 698 effective full power hours in the month of November. For the period October 7, 1995 to November 30, 1995, the Fuel Utilization Factor was 85.2%.

Data Source: 1192 Report
Targets: Nuclear Fuel
Contact: Ed Knuckles 694-3320

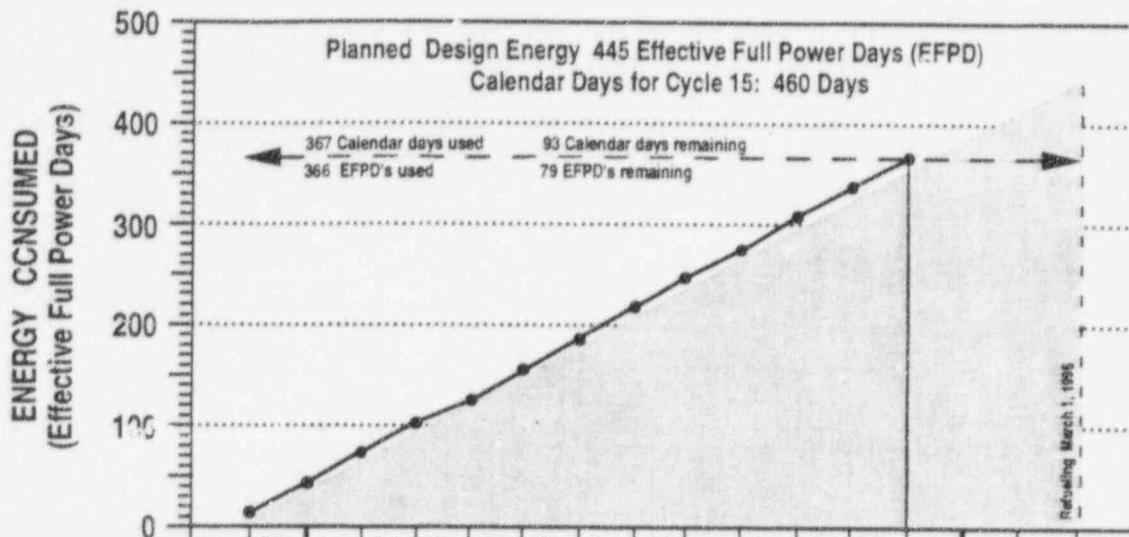
Indicator Type

☐ NRC ☐ INPO ☐ Corp ☒ Div

FUEL UTILIZATION

Planned
Actual

TURKEY POINT UNIT 4 CYCLE 15



	Nov	Dec	Jan	Feb	Mar	Apr	May	Jun	Jul	Aug	Sep	Oct	Nov	Dec	Jan	Feb	Mar
Planned Burnup (EFPD)	4	34	64	91	121	150	180	209	239	269	298	328	357	387	417	445	
Actual Burnup (EFPD)	13	42	73	101	125	155	186	215	246	276	306	337	366				
Variance (EFPD) (+/-)	+9	+8	+9	+10	+4	+5	+6	+6	+7	+7	+8	+9	+9				

Definitions

Fuel Utilization plots the amount of nuclear energy used during the current fuel cycle. The amount of nuclear energy is expressed in effective full power days (EFPD). One EFPD is the equivalent to operating the reactor at maximum thermal rating (2200 megawatts thermal) for a 24 hour period. Planned Energy is compared to Actual Energy Used during the cycle. Fuel utilization is directly related to plant performance. The significance of variance EFPD (+/-) is the difference between planned and actual, this can also be used to project longer or shorter fuel cycle runs.

Fuel Cycle Operating Assumptions

The actual Unit 4 Cycle 15 start-up was on November 14, 1994, 13 days earlier than the November 27, 1994 assumption in the October 28, 1994 Approved Operating Schedule(AOS). Cycle 15 is scheduled to run to March 1, 1996, with a design energy of 445 EFPD.

$$\text{Actual Energy} \div \text{Calendar Days} = \text{Fuel Utilization Factor}$$

Performance Summary

Unit 4 achieved a Fuel Utilization Factor of 96.7% and operated for 696 effective full power hours in the month of November 1995. For the period of November 14, 1994 to November 30, 1995, the Fuel Utilization Factor was 95.2%.



Data Source: 1192 Report
Targets: Nuclear Fuel
Contact: Ed Knuckles 694-3320

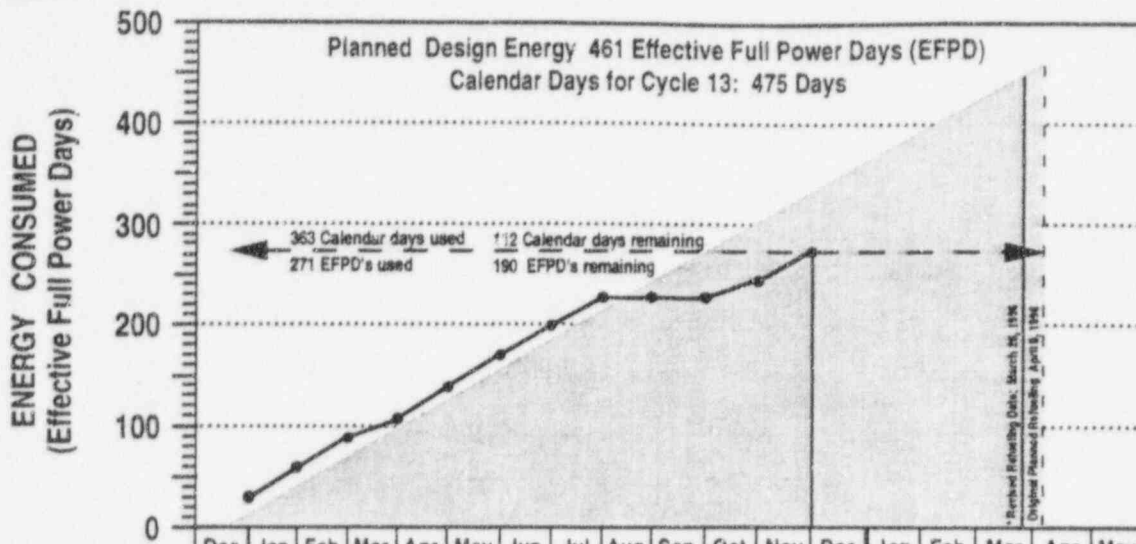
Indicator Type

☐ NRC ☐ INPO ☐ Corp ☒ Div

FUEL UTILIZATION

ST. LUCIE UNIT 1 CYCLE 13

Planned 
Actual 



	Dec	Jan	Feb	Mar	Apr	May	Jun	Jul	Aug	Sep	Oct	Nov	Dec	Jan	Feb	Mar	Apr	May
Planned Burnup (EFPD)	11	41	68	98	127	158	187	217	247	276	306	335	366	396	423	453	461	
Actual Burnup (EFPD)	29	60	86	109	139	170	199	227	227	227	244	271						
Variance (EFPD) (+/-)	+18	+19	+18	+11	+12	+12	+12	+10	-20	-49	-62	-64						

Definitions

Fuel Utilization plots the amount of nuclear energy used during the current fuel cycle. The amount of nuclear energy is expressed in effective full power days (EFPD). One EFPD is the equivalent to operating the reactor at maximum thermal rating (2700 megawatts thermal) for a 24 hour period. Planned Energy is compared to Actual Energy Used during the cycle. Fuel utilization is directly related to plant performance. The significance of variance EFPD (+/-) is the difference between planned and actual, this can also be used to project longer or shorter fuel cycle runs.

Fuel Cycle Operating Assumptions

In accordance with the October 28, 1994 Approved Operating Schedule (AOS), Unit 1, Cycle 13 was scheduled to begin operation December 20, 1994. This provided for a cycle of 475 calendar days with design energy to run 461 effective full power days (EFPD). Unit 1, Cycle 13 actually began operation November 29, 1994 and is currently scheduled to refuel April 8, 1996.

* Per the revised Approved Operating Schedule dated April 26, 1995, the unit will shutdown 13 days earlier than previously scheduled.

$$\text{Actual Energy} \div \text{Calendar Days} = \text{Fuel Utilization Factor}$$

Performance Summary



Unit 1 achieved a Fuel Utilization Factor of 90.9% and operated at 655 effective full power hours in the month of November. For the period of April 22, 1994 through November 30, 1995, the Fuel Utilization Factor was 74.0%.

Data Source: 1192 Report
Targets: Nuclear Fuel
Contact: Erwin Wunderlich 694-3439

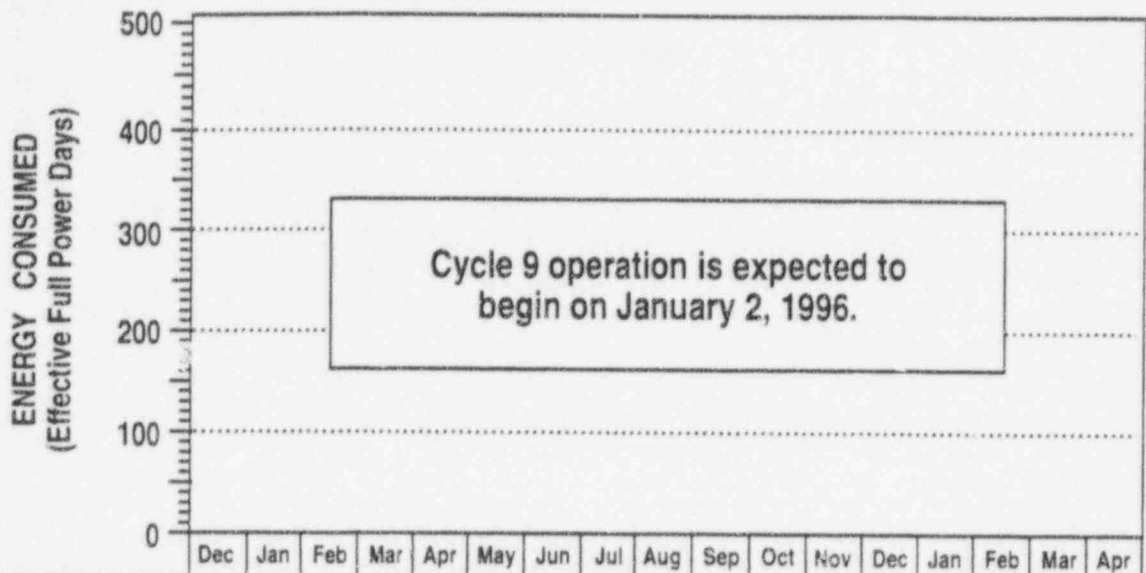
Indicator Type

☐ NRC ☐ INPO ☐ Corp ☒ Div

FUEL UTILIZATION

Planned 
Actual 

ST. LUCIE UNIT 2 CYCLE 9



	Dec	Jan	Feb	Mar	Apr	May	Jun	Jul	Aug	Sep	Oct	Nov	Dec	Jan	Feb	Mar	Apr
Planned Burnup (EFPD)																	
Actual Burnup (EFPD)																	
Variance (EFPD) (+/-)																	

Definitions

Fuel Utilization plots the amount of nuclear energy used during the current fuel cycle. The amount of nuclear energy is expressed in effective full power days (EFPD). One EFPD is the equivalent to operating the reactor at maximum thermal rating (2700 megawatts thermal) for a 24 hour period. Planned Energy is compared to Actual Energy Used during the cycle. Fuel utilization is directly related to plant performance. The significance of variance EFPD (+/-) is the difference between planned and actual, this can also be used to project longer or shorter fuel cycle runs.

Fuel Cycle Operating Assumptions

$$\text{Actual Energy} \div \text{Calendar Days} = \text{Fuel Utilization Factor}$$

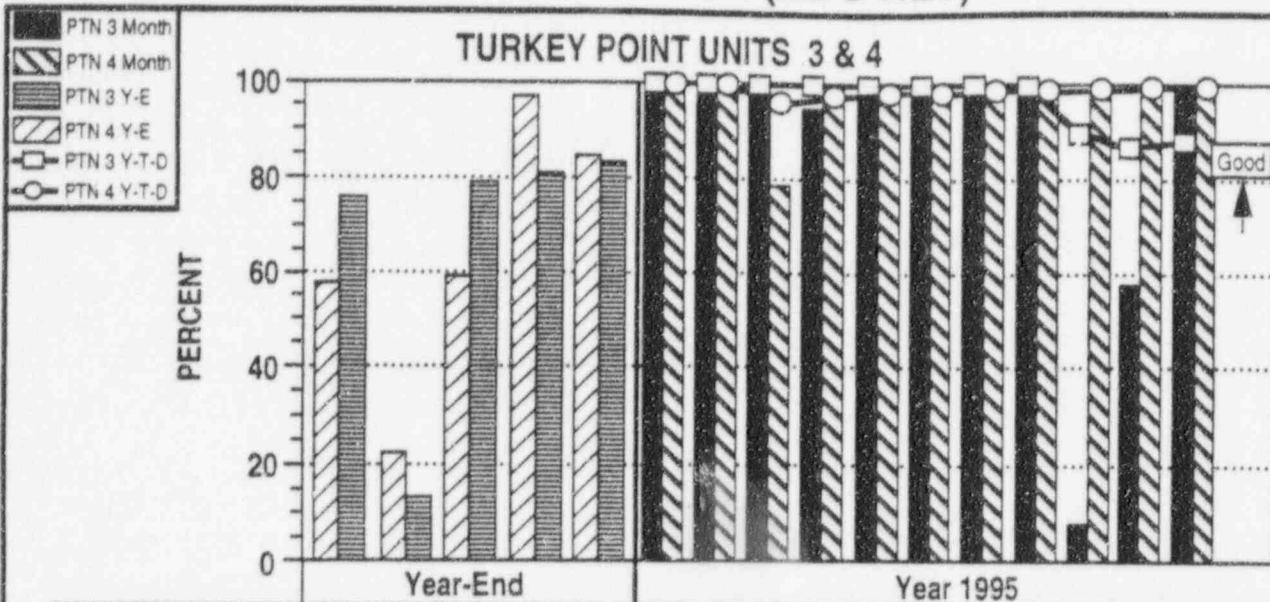
Performance Summary

Data Source: 1192 Report
Targets: Nuclear Fuel
Contact: Erwin Wunderlich 694-3439

Indicator Type

☐ NRC ☐ INPO ☐ Corp ☒ Div

CAPACITY FACTOR (MDC NET)



Legend / Period		1990	1991	1992	1993	1994	Jan	Feb	Mar	Apr	May	Jun	Jul	Aug	Sep	Oct	Nov	Dec
Unit 3	Month %						104	103	101	94.9	99.9	97.3	100	99.4	7.9	57.4	101	
	Y-T-D %	57.6	22.5	58.4	97.0	84.4	104	104	103	101	101	100	100	99.9	89.8	86.5	87.8	
Unit 4	Month %						105	104	79.0	103	101	97.9	101	97.9	101	102	99.5	
	Y-T-D %	75.1	13.7	79.3	81.4	83.0	105	105	95.7	97.4	98.1	98.1	98.4	98.4	98.7	99.0	99.1	

Definitions

Unit Capacity Factor (using Maximum Dependable Capacity) is the ratio of the net electrical energy generated to the product of maximum dependable capacity multiplied by the hours in the reporting period. The maximum dependable capacity is the gross electrical output measured at the output terminals of the turbine generator during the most restrictive seasonal condition less the normal station service loads. The higher the capacity factor, the closer the plant is to operating at its dependable energy production capacity.

$$\text{Capacity Factor} = \frac{\text{Net Electrical Energy}}{\text{Maximum Dependable Capacity} \times \text{Period Hours}} \times 100\%$$

Statistical Summary

	Nov	YTD
Unit 3 - 1995	101.1%	87.8%
Unit 3 - 1994	102.6%	84.3%
Unit 4 - 1995	99.5%	99.1%
Unit 4 - 1994	41.4%	81.6%
Targets:	Not established	

Industry Performance

NERC GADS	Capacity Factor
1994 Avg - PWR's	76.7%
1994 Avg - All Units	72.8%
1990 - 1994 Avg - PWR's	72.8%
1990 - 1994 Avg - All Units	69.5%
INPO	
July 1994 - June 1995 Median	82.4%
July 1994 - June 1995 Average	77.4%

Performance Summary

Capacity Factor (%) for Unit 3 was 101.1% in November; year-to-date, Capacity Factor was 87.8%.

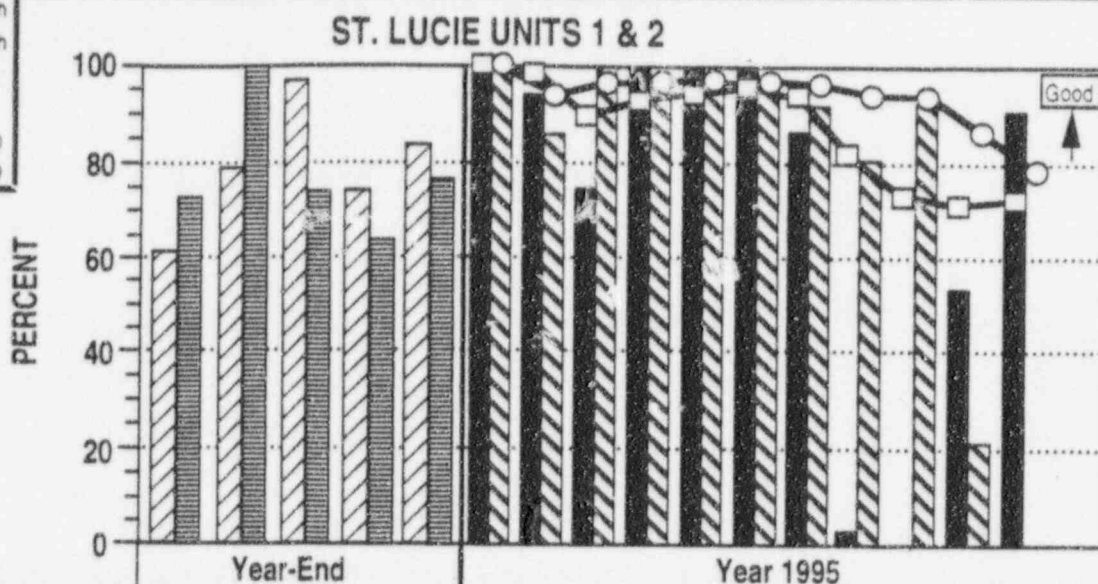
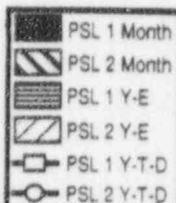
Unit 4 Capacity Factor (%) for Unit 4 was 99.5% for the month and 99.1% year-to-date.

Data Source: 1192N Report

Indicator Type

☐ NRC ☐ INPO ☐ Corp ☐ Div

CAPACITY FACTOR (MDC NET)



Legend / Period		1990	1991	1992	1993	1994	Jan	Feb	Mar	Apr	May	Jun	Jul	Aug	Sep	Oct	Nov	Dec
Unit 1	Month %						102	94.1	74.2	101	101	99.2	86.8	0.3	0.0	53.3	90.2	
	Y-T-D %	61.1	78.8	96.9	73.9	84.1	102	98.2	89.9	92.8	94.4	95.2	94.0	82.0	72.8	70.8	72.6	
Unit 2	Month %						101	85.7	101	97.1	98.2	96.6	92.1	80.4	92.3	21.4	0.0	
	Y-T-D %	72.4	101.1	73.7	64.1	76.3	101	94.0	96.3	96.5	96.9	96.8	96.1	94.1	93.9	86.5	78.7	

Definitions

Unit Capacity Factor (using Maximum Dependable Capacity) is the ratio of the net electrical energy generated to the product of maximum dependable capacity multiplied by the hours in the reporting period. The maximum dependable capacity is the gross electrical output measured at the output terminals of the turbine generator during the most restrictive seasonal condition less the normal station service loads. The higher the capacity factor, the closer the plant is to operating at its dependable energy production capacity.

$$\text{Capacity Factor} = \frac{\text{Net Electrical Energy}}{\text{Maximum Dependable Capacity} \times \text{Period Hours}} \times 100\%$$

Statistical Summary

	Nov	YTD
Unit 1 - 1995	90.2%	72.6%
Unit 1 - 1994	0.0%	83.2%
Unit 2 - 1995	0.0%	78.7%
Unit 2 - 1994	100.2%	74.3%
Targets: Not established		

Industry Performance

NERC GADS	Capacity Factor
1994 Avg - PWR's	76.7%
1994 Avg - All Units	72.8%
1990 - 1994 Avg - PWR's	72.8%
1990 - 1994 Avg - All Units	69.5%
INPO	
July 1994 - June 1995 Median	82.4%
July 1994 - July 1995 Average	77.4%

Performance Summary

Unit 1 Capacity Factor (%) for the month was 90.2% and 72.6% year-to-date.

Unit 2 was shutdown in November for a scheduled Refueling Outage. Capacity Factor (%) was 78.7% year-to-date.

Data Source: 1192N Report

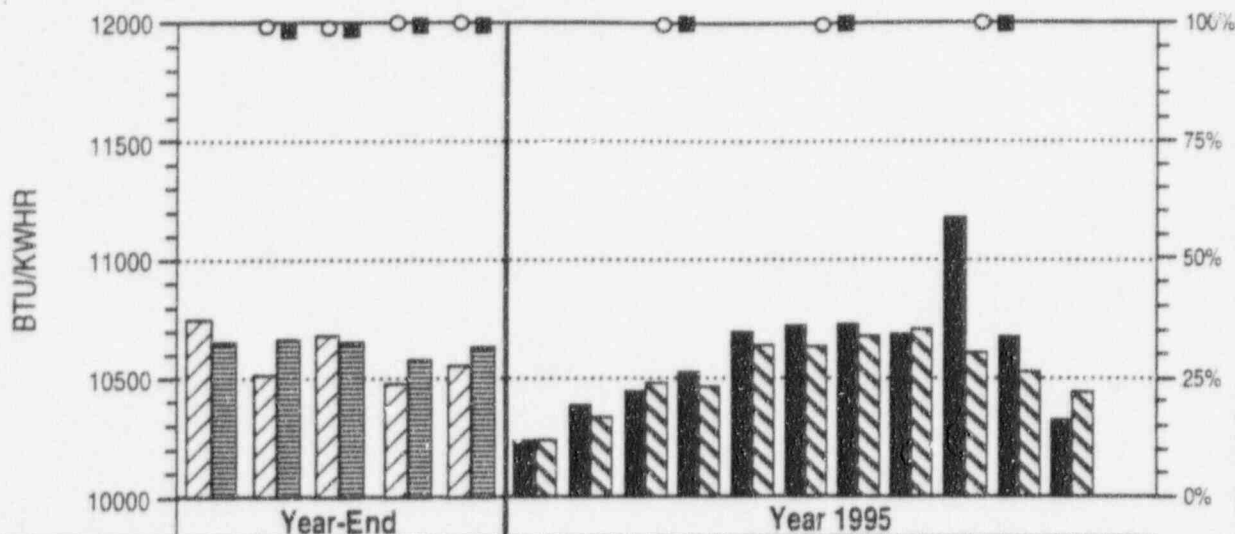
Indicator Type

☐ NRC ☐ INPO ☐ Corp ☐ Div

THERMAL PERFORMANCE

(GROSS HEAT RATE)

TURKEY POINT UNITS 3 & 4



Legend / Period	1990	1991	1992	1993	1994	Jan	Feb	Mar	Apr	May	Jun	Jul	Aug	Sep	Oct	Nov	Dec
PTN 3 Month						10236	10387	10438	10517	10697	10710	10713	10694	11181	10681	10315	
PTN 3 Year-End	10746	10505	10674	10474	10557												
PTN 3 Y-E and Qtr (%)		99.3%	99.2%	100.4%	100.3%			99.5%		99.5%				99.6%			
PTN 4 Month						10236	10343	10482	10464	10645	10635	10676	10701	10603	10530	10438	
PTN 4 Year-End	10641	10657	10655	10578	10625												
PTN 4 Y-E and Qtr (%)		98.7%	99.0%	99.7%	99.7%			100.1%		100.5%				99.9%			

Definitions

Gross Heat Rate is the ratio of thermal energy in British thermal units produced by the reactor during a given period to the total gross electrical energy in kilowatt-hours produced by the generator during the same period.

Statistical Summary

	Nov	12-Month Running Avg
PTN 3	10315	10530
PTN 4	10438	10511
1995 Target:		
PTN 3	10397	
PTN 4	10391	

Industry Performance

INPO

BTU/KHR	
1994 Industry Average	10188
Ratio of Design to Actual Gross Heatrate (%)	
July 1994 - June 1995 Median	99.4
1995 Goal	99.5

Performance Summary

Units 3 & 4 12-Month Running Average Gross Heat Rate was higher than the Industry Average of 10188 in November.

Data Source: 1192 Report
Contact: Luis Gutierrez, 246-7261

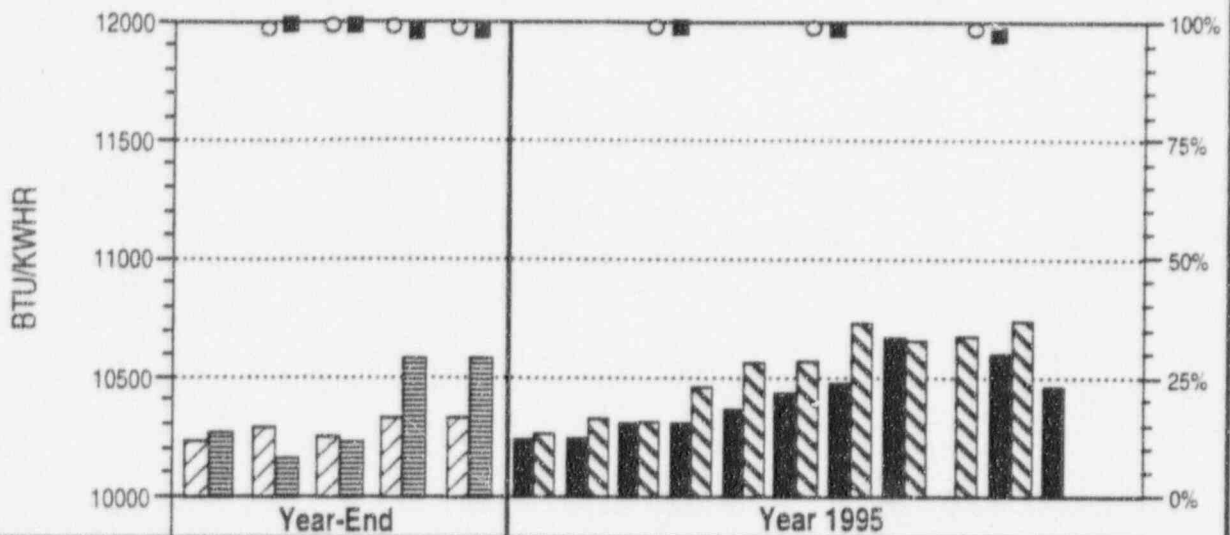
Indicator Type

☐ NRC ☒ INPO ☐ Corp ☐ Div

THERMAL PERFORMANCE

(GROSS HEAT RATE)

ST. LUCIE UNITS 1 & 2



Legend / Period	1990	1991	1992	1993	1994	Jan	Feb	Mar	Apr	May	Jun	Jul	Aug	Sep	Oct	Nov	Dec
PSL 1 Month						10241	10246	10308	10308	10362	10431	10474	10665	0	10600	10462	
PSL 1 Year-End	10227	10299	10248	10336	10423												
PSL 1 Y-E and Qtr(%)		99.0%	99.3%	99.2%	99.0%			99.1%			99.0%			98.7%			
PSL 2 Month						10269	10318	10314	10466	10559	10561	10703	10663	10679	10738	0	
PSL 2 Year-End	10270	10165	10231	10579	10466												
PSL 2 Y-E and Qtr(%)		99.6%	99.3%	97.8%	98.5%			99.2%			98.4%			97.5%			

Definitions

Gross Heat Rate is the ratio of thermal energy in British thermal units produced by the reactor during a given period to the total gross electrical energy in kilowatt-hours produced by the generator during the same period.

Statistical Summary

	Nov	12-Month Running Avg
PSL 1	10462	10365
PSL 2	0	10493
1995 Target:		
PSL 1	10141	
PSL 2	10141	

Industry Performance

INPO	
BTU/KHR	
1994 Industry Average	10188
Ratio of Design to Actual Gross Heatrate (%)	
July 1994 - June 1995 Median	99.4
1995 Goal	99.5

Performance Summary

St. Lucie Units 1 & 2 12-Month Running Average Gross Heat Rate was higher than the Industry Average of 10188 for the month of November.

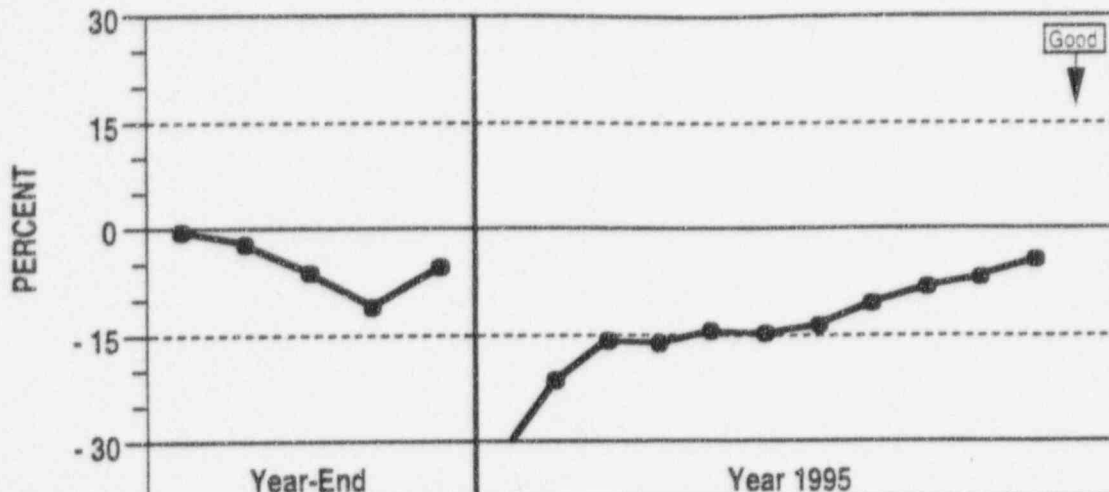
Data Source: 1192 Report
Contact: Ray Riha 465-3092

Indicator Type

☐ NRC ☒ INPO ☐ Corp ☐ Div

O & M BUDGET VARIANCE

NUCLEAR DIVISION



Legend/ Period	1990	1991	1992	1993	1994	Jan	Feb	Mar	Apr	May	Jun	Jul	Aug	Sep	Oct	Nov	Dec
Y-T-D Actual (\$M)	349.4	342.4	295.2	290.6	285.2	12.7	27.8	45.8	65.4	80.3	97.0	113.8	134.6	164.7	211.3	242.9	
Y-T-D Budgeted (\$M)	351.4	333.3	314.6	326.4	302.0	18.9	35.5	54.2	76.2	93.9	113.4	131.7	150.5	179.5	227.4	254.8	
Y-T-D Variance (%)	-0.5	2.7	-6.2	-11.0	-5.6	-33.1	-21.8	-15.6	-14.2	-14.4	-14.5	-13.6	-10.6	-8.2	-7.1	-4.7	

Definitions

Operating and Maintenance Expenditures include Nuclear Division operation and maintenance expenses associated with direct employees, contractors and consultants, equipment, tools, design, engineering and other items/activities required to sustain the electrical generation of the plants and to provide required support. Fuel costs, corporate administrative and general expenses, and charges from other departments outside the Nuclear Division are excluded.

$$\frac{\text{Y-T-D Actual Expenses} - \text{Y-T-D Budgeted Expenses}}{\text{Y-T-D Budgeted Expenses}} \times 100\% = \text{O \& M Variance \%}$$

Statistical Summary

	Nov YTD Actual (\$ M)	YTD Variance (\$ M)	%
O&M - 1995	242.9 (\$ M)	-11.9 (\$ M)	-4.7%
O&M - 1994	255.1 (\$ M)	-21.5 (\$ M)	-7.8%

1995 Year-End Budget: 279.2 (\$ M)

Industry Performance

O&M Budget

IBG Actual 1994 Avg.	\$ 297.0M
IBG Top Quartile Entry	\$ 270.0M
IBG Top Quartile Avg.	\$ 213.2M
IBG Projected 1995 Avg.	\$ 284.0M

Projection (per dual unit site) derived by trending 1986-94 Actual data and 1995 Budgeted data for IBG Group.

Performance Summary

O&M expenditures through November 1995 were \$242.9 million which represented a budget underrun of \$11.9 million (or 4.7%). The variance was primarily due to: underruns in materials and contracted services at Turkey Point due to less trouble and breakdown than planned; Nuclear Division O&M Contingency not yet needed; and over-accrual of estimated 1994 Year-End liabilities.

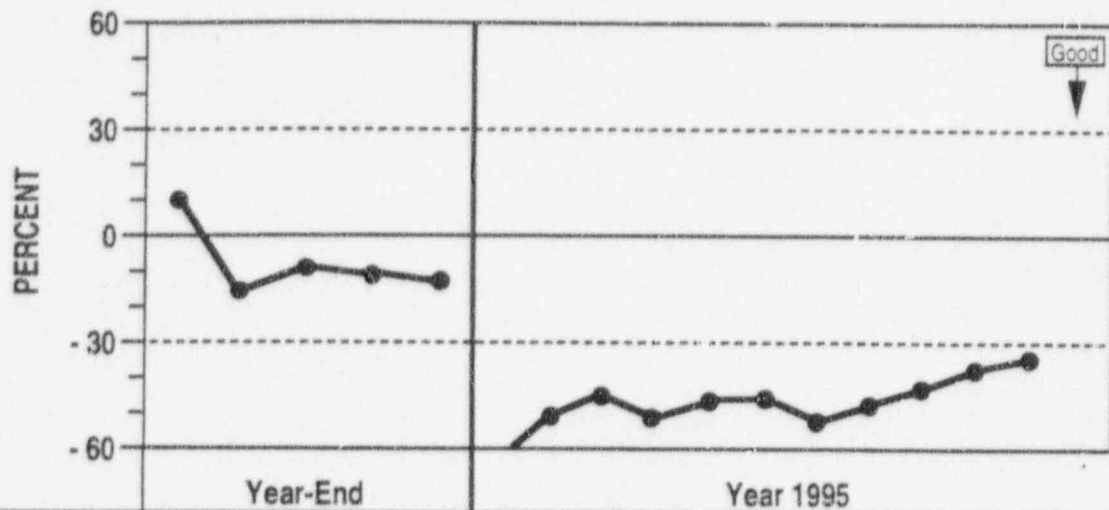
Data Source: Resource Allocation (PRA)

Indicator Type

☐ NRC ☐ INPO ☒ Corp ☒ Div

CAPITAL BUDGET VARIANCE

NUCLEAR DIVISION



Legend / Period	1990	1991	1992	1993	1994	Jan	Feb	Mar	Apr	May	Jun	Jul	Aug	Sep	Oct	Nov	Dec
Y-T-D Actual (\$M)	195.5	150.7	68.4	90.3	76.5	1.5	3.6	6.0	7.3	9.5	11.7	13.6	18.0	24.0	31.4	37.5	
Y-T-D Budgeted (\$M)	186.1	179.6	75.1	101.4	87.7	5.0	7.4	10.8	14.7	17.7	21.7	28.8	35.2	42.0	51.1	56.9	
Y-T-D Variance (%)	5.0	-16.1	-9.0	-11.0	-12.7	-65.0	-50.4	-44.2	-50.7	-46.4	-46.0	-52.8	-48.7	-42.8	-39.6	-34.0	

Definitions

Capital Expenditures are those directly incurred/budgeted by the Nuclear Division for the construction of new utility plant additions and improvements made to increase efficiency, reliability or safety. Capital fuel costs are excluded.

$$\frac{\text{Y-T-D Actual Expenses} - \text{Y-T-D Budgeted Expenses}}{\text{Y-T-D Budgeted Expenses}} \times 100\% = \text{Capital Variance \%}$$

Statistical Summary

	Nov Y-T-D Actual (\$ M)	YTD Variance (\$ M)	%
Capital - 1995	37.5 (\$ M)	-19.4 (\$ M)	-34.0%
Capital - 1994	57.8 (\$ M)	-23.5 (\$ M)	-28.9%

1995 Year-End Budget: 64.7 (\$ M)

Industry Performance

Capital Budget

IBG Actual 1994 Avg.	\$ 63.1M
IBG Top Quartile Entry	\$ 28.3M
IBG Top Quartile Avg.	\$ 21.1M
IBG Projected 1995 Avg.	\$ 58.5M

Projection (per dual unit site) derived by trending 1986-94 Actual data and 1995 Budgeted data for IBG Group.

Performance Summary

Capital expenditures through November 1995 were \$37.5 million. This represented a budget underrun of \$19.4 million (or 34.0%). The variance was primarily due to: project cancellations, scope/estimate reductions, and schedule changes for various projects; underrun in Contractor Wrap-Up Insurance; and, delay in support services and staffing under budget for the Steam Generator Replacement Project at St. Lucie.

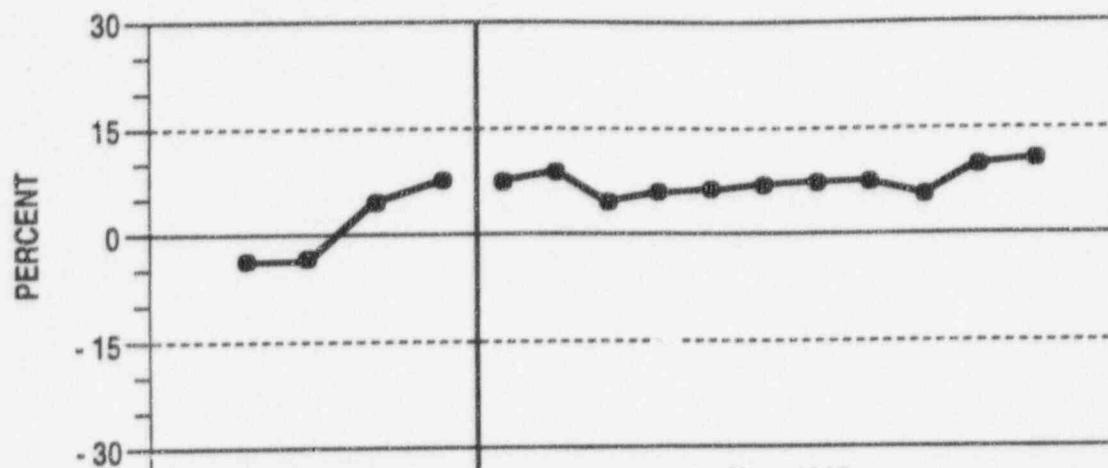
Data Source: Resource Allocation (PRA)

Indicator Type

☐ NRC ☐ INPO ☒ Corp ☒ Div

FUEL OPERATING EXPENSE VARIANCE

TURKEY POINT UNITS 3 & 4



Legend/ Period	Year-End					Year 1995											
	1990	1991	1992	1993	1994	Jan	Feb	Mar	Apr	May	Jun	Jul	Aug	Sep	Oct	Nov	Dec
Y-T-D Actual (\$M)	n/a	15.9	55.9	73.1	67.6	5.8	11.2	16.5	22.3	26.0	33.8	39.6	45.4	48.8	53.2	60.9	
Y-T-D Budgeted (\$M)	n/a	16.5	58.1	70.0	62.9	5.4	10.4	15.8	21.0	26.4	31.6	36.9	42.3	46.0	48.5	55.2	
Y-T-D Variance (%)	n/a	-4.1	-3.7	4.4	7.5	7.4	8.6	4.6	5.9	6.2	7.1	7.3	7.4	6.1	9.8	10.4	

Definitions

Fuel Operating Expenses include fuel investment amortization, interest, and administrative charges for the fuel leases, payments and credits from the Department of Energy (DOE) for spent fuel disposal, and payment to the Department of the Treasury for Decontamination and Decommissioning of the DOE enrichment facilities.

$$\frac{\text{Y-T-D Actual Expenses} - \text{Y-T-D Budgeted Expenses}}{\text{Y-T-D Budgeted Expenses}} \times 100\% = \text{Fuel Operating Expense Variance \%}$$

Statistical Summary

	Nov YTD Actual (\$ M)	YTD Variance (\$ M)	%
Fuel - 1995	60.9 (\$ M)	5.7 (\$ M)	10.4%
Fuel - 1994	62.4 (\$ M)	3.4 (\$ M)	5.8%

1995 Year-End Budget: 60.1 (\$ M)

Industry Performance

Not applicable. Nuclear Fuel Expense reported on an Industry basis in mills/kwh.

Performance Summary

Turkey Point Y-T-D variance was due to higher generation (+8.7%) and a higher amortization rate (+1.6%). The increase in Y-T-D generation was due to the early start up of Unit 3, Cycle 15, and both units operating at a higher plant capacity than projected. The higher Y-T-D amortization rate is due to: (a) increased amortization due to the design change for Unit 3, which caused the early discharge of eight assemblies at the end of Cycle 14, and, an accounting adjustment due to the fact that a set of four assemblies were rescheduled for discharge during Unit 3's refueling outage; and, (b) a decrease in amortization due to better actual heat rate for both units. In addition, the Y-T-D higher generation lowered the impact of the Decontamination and Decommissioning fee, thereby reducing the amortization rate.

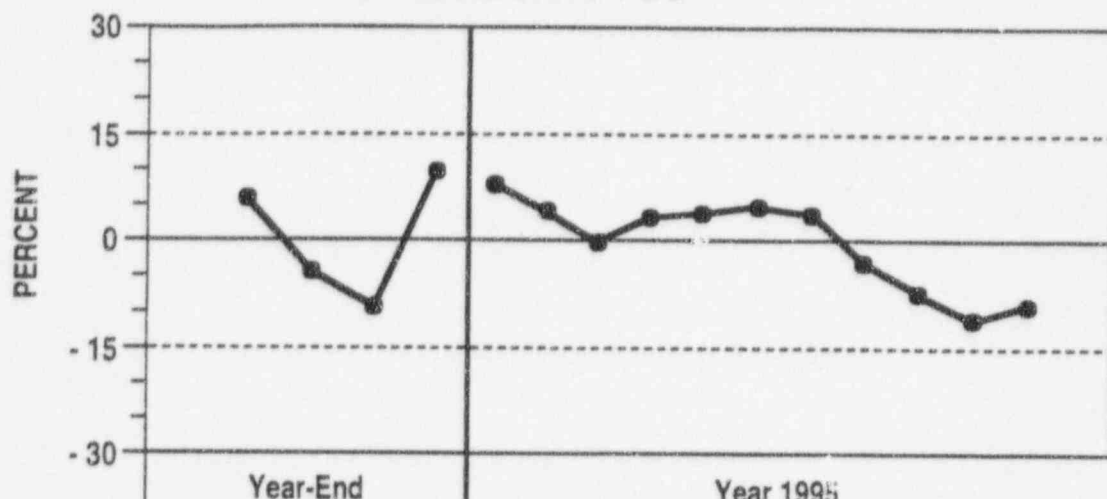
Contact: Lorraine Tymms, 694-3450

Indicator Type

☐ NRC ☐ INPO ☒ Corp ☒ Div

FUEL OPERATING EXPENSE VARIANCE

ST. LUCIE UNITS 1 & 2



Legend/ Period	1990	1991	1992	1993	1994	Jan	Feb	Mar	Apr	May	Jun	Jul	Aug	Sep	Oct	Nov	Dec
Y-T-D Actual (\$M)	n/a	86.7	78.5	64.1	74.1	6.8	12.5	18.2	25.0	31.6	38.3	44.2	47.4	50.6	53.3	59.4	
Y-T-D Budgeted (\$M)	n/a	81.9	82.1	70.6	67.6	6.3	12.0	18.3	24.3	30.5	36.5	42.6	48.8	54.8	59.9	65.4	
Y-T-D Variance (%)	n/a	5.8	- 4.3	- 9.2	9.7	7.8	4.1	- 0.3	2.9	3.5	4.8	3.7	- 2.8	- 7.5	- 11.1	- 9.2	

Definitions

Fuel Operating Expenses include fuel investment amortization, interest, and administrative charges for the fuel leases, payments and credits from the Department of Energy (DOE) for spent fuel disposal, and payment to the Department of the Treasury for Decontamination and Decommissioning of the DOE enrichment facilities.

Note: PSL 2 reflects FPL's share only.

$$\frac{\text{Y-T-D Actual Expenses} - \text{Y-T-D Budgeted Expenses}}{\text{Y-T-D Budgeted Expenses}} \times 100\% = \text{Fuel Operating Expense Variance \%}$$

Statistical Summary

	Nov YTD Actual (\$ M)	YTD Variance (\$ M)	%
Fuel - 1995	59.4 (\$ M)	- 6.0 (\$ M)	- 9.2%
Fuel - 1994	68.1 (\$ M)	6.4 (\$ M)	10.4%

1995 Year-End Budget: 75.1 (\$ M)

Industry Performance

Not applicable. Nuclear Fuel Expense reported on an Industry basis in mills/kwh.

Performance Summary

The St. Lucie variance as of the end of November was due to lower generation (-15.3%) and a higher amortization rate (+7.2%). The reduction in Y-T-D generation was due to: (a) a decrease in generation for Unit 1 due to the March outage for Pressurizer Valve repair, the reactor trip in July during Turbine Control testing, and various operational problems in August through October; and (b) an decrease in generation for Unit 2 due to the refueling outage starting earlier than assumed in the budget for last year. The increase in Y-T-D amortization rate is mostly due to the design change for Unit 2, which caused full amortization of additional assemblies scheduled to be discharged at the end of Cycle 8.

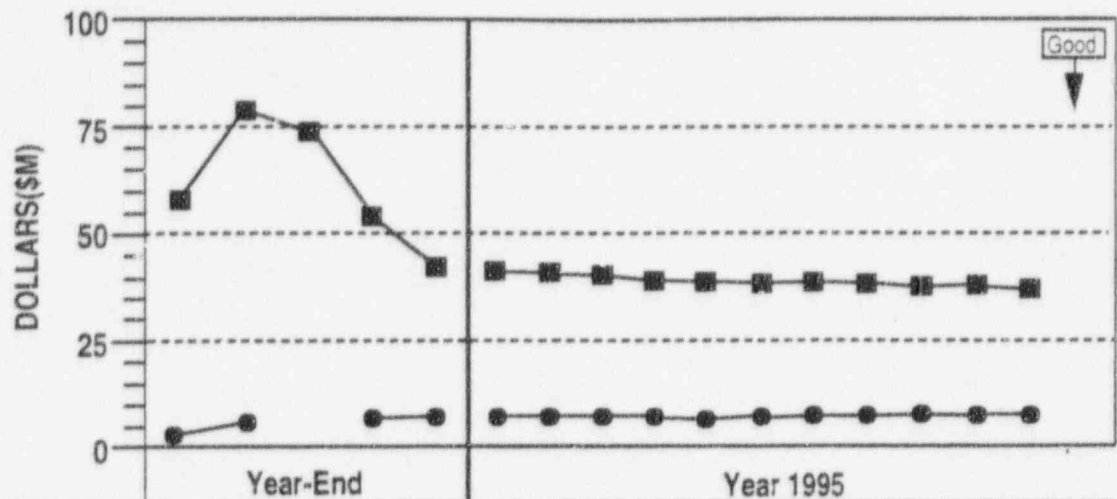
Contact: Lorraine Tymms, 694-3450

Indicator Type

☐ NRC ☐ INPO ☒ Corp ☒ Div

INVENTORY LEVELS

TURKEY POINT



Legend/ Period	1990	1991	1992	1993	1994	Jan	Feb	Mar	Apr	May	Jun	Jul	Aug	Sep	Oct	Nov	Dec
● Y-T-D CAPITAL	2.7	5.8	n/a	6.4	6.9	6.9	6.9	6.9	6.9	6.3	7.1	7.3	7.3	7.4	7.3	7.3	
■ Y-T-D REGULAR	58.0	79.1	74.7	54.6	42.4	41.5	40.9	40.4	39.6	39.4	39.0	39.1	39.0	37.9	38.1	37.4	

Definitions

Capital - Part or equipment which: meets the criteria established by the Property Retirement Unit Catalog for a Retirement Unit; is associated with a specific plant in service, the failure of which would seriously impair the utility's ability to provide continued operation; and, is not readily available from suppliers, or typically require a long lead time.

Regular - Materials needed to keep operational the physical equipment and facilities of the plant (e.g., spare parts, consumables, commodities, tools).

Statistical Summary

	Start	End	% Change
Capital			
Monthly	7.3	7.3	0.0%
Y-T-D	6.9	7.3	5.8%
Regular			
Monthly	38.1	37.4	- 1.8%
Y-T-D	42.4	37.4	- 11.8%

1995 Target: Regular \$39M

Industry Performance

	Total (Regular Only)
IBG (Year 1994)	
Average	\$ 43.6
Top Quartile Entry	\$ 34.7
Top Quartile Average	\$ 28.9

Performance Summary

Turkey Point's Capital inventory remained the same for the month.

Regular inventory at Turkey Point decreased in November due to continued dormant material evaluation and restock analysis.

Data Source: Dick Rose 246-6692

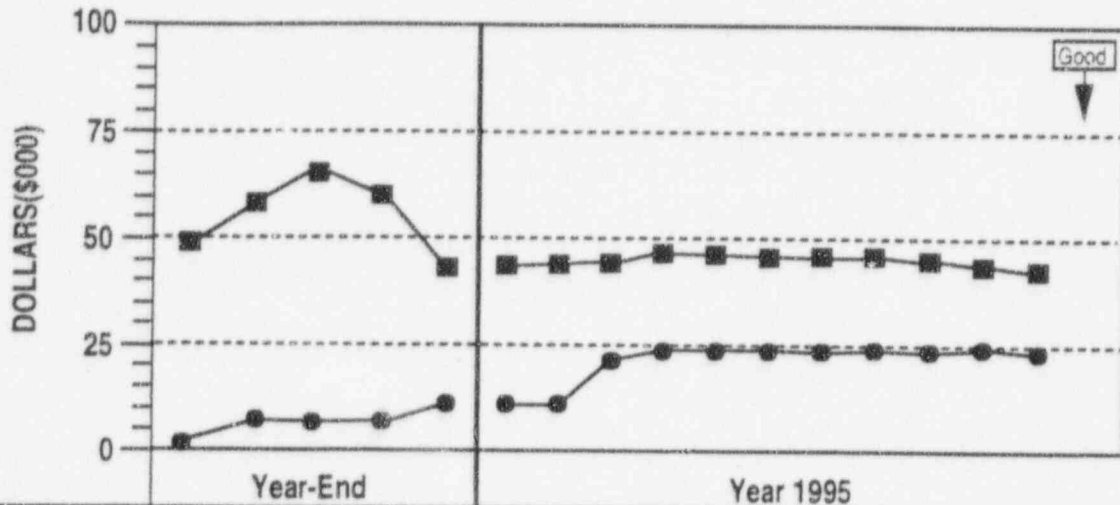
Nuclear Materials Management Manager

Indicator Type

☐ NRC ☐ INPO ☐ Corp ☒ Div

INVENTORY LEVELS

ST. LUCIE



Legend/ Period	1990	1991	1992	1993	1994	Jan	Feb	Mar	Apr	May	Jun	Jul	Aug	Sep	Oct	Nov	Dec
● Y-T-D CAPITAL	1.8	7.2	7.0	7.1	11.2	11.2	11.2	21.5	24.3	24.3	24.3	24.2	24.3	24.2	24.5	23.5	
■ Y-T-D REGULAR	49.4	58.7	65.2	60.2	43.7	44.2	44.6	44.9	46.6	46.3	46.1	46.1	46.2	45.2	44.2	43.1	

Definitions

Capital - Part or equipment which: meets the criteria established by the Property Retirement Unit Catalog for a Retirement Unit; is associated with a specific plant in service, the failure of which would seriously impair the utility's ability to provide continued operation; and, is not readily available from suppliers, or typically requires a long lead time.

Regular - Materials needed to keep operational the physical equipment and facilities of the plant (e.g., spare parts, consumables, commodities, tools).

Statistical Summary

	Start	End	% Change
Capital			
Monthly	24.5	23.5	- 4.1%
Y-T-D	11.2	23.5	109.8%
Regular			
Monthly	44.2	43.1	- 2.5%
Y-T-D	43.7	43.1	- 1.4%

1995 Target: Regular \$40M

Industry Performance

	Total (Regular Only)
IBG (Year 1994)	
Average	\$ 43.6
Top Quartile Entry	\$ 34.7
Top Quartile Average	\$ 28.9

Performance Summary

St. Lucie's Regular Inventory Level decreased by 2.5% in November and by 1.4% year-to-date.

Note: Regular Inventory values shown above were adjusted for July through October 1995.

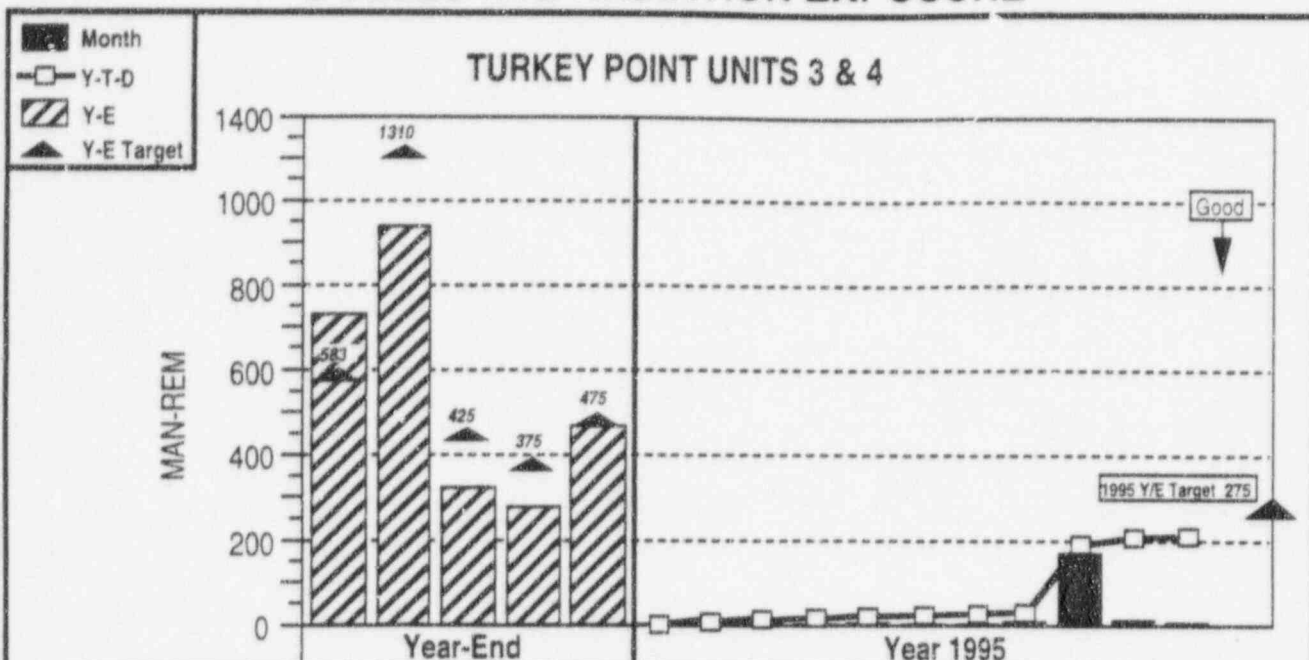
Data Source: Tom Kreinburg 465-4183
Nuclear Materials Management Manager

Indicator Type

☐ NRC ☐ INPO ☐ Corp ☒ Div

COLLECTIVE RADIATION EXPOSURE

TURKEY POINT UNITS 3 & 4



Legend / Period	1990	1991	1992	1993	1994	Jan	Feb	Mar	Apr	May	Jun	Jul	Aug	Sep	Oct	Nov	Dec
Month						3.0	2.2	9.8	2.2	3.1	2.9	1.3	4.6	168	10.8	4.2	
Year-To-Date	731	938	326	276	469	3.0	5.3	15.0	17.3	20.4	23.3	24.6	29.2	198	208	213	

Definitions

Collective Radiation Exposure is the total external whole-body radiation dose received by all on-site personnel (including contractors and visitors) during the time period as measured by thermoluminescent dosimeters (TLD's). It is reported in man-rem for the station. Current month readings are taken from direct reading dosimeters (DRD's). The 3 Year Running Average values are adjusted as TLD results become available.

Statistical Summary

	Nov	YTD	3 Yr Running
Man-Rem - 1995	4.2	212.6	323.8
Man-Rem - 1994	21.9	462.1	362.5
Targets:			
1995 Y-T-D (TLD)	270.0		
1995 Year-End (TLD)	275.0		
1994 Year-End (TLD)	475.0		

Industry Performance

INPO (PWR's)	Man-Rem (2 Unit Site)
Median 3-yr Distribution (7/92 - 6/95)	314
1995 Goal	370

Performance Summary

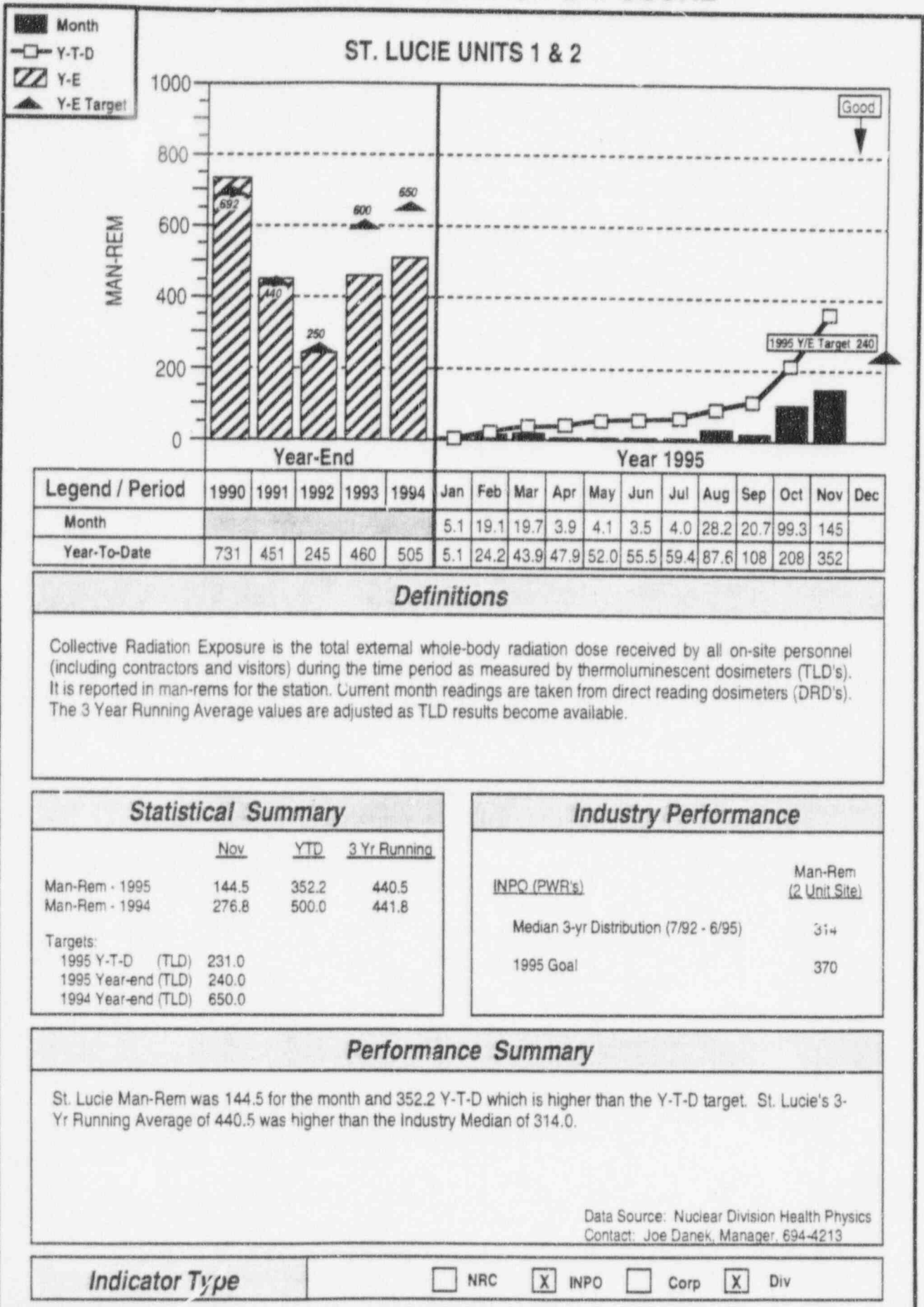
Turkey Point Man-Rem was 4.2 in November and 212.6 Y-T-D which was below the Y-T-D target of 270.0. Turkey Point's 3-Yr Running Average of 323.8 exceeds Industry Median of 314.0.

Data Source: Nuclear Division Health Physics
 Contact: Joe Daneke, Manager, 694-4213

Indicator Type

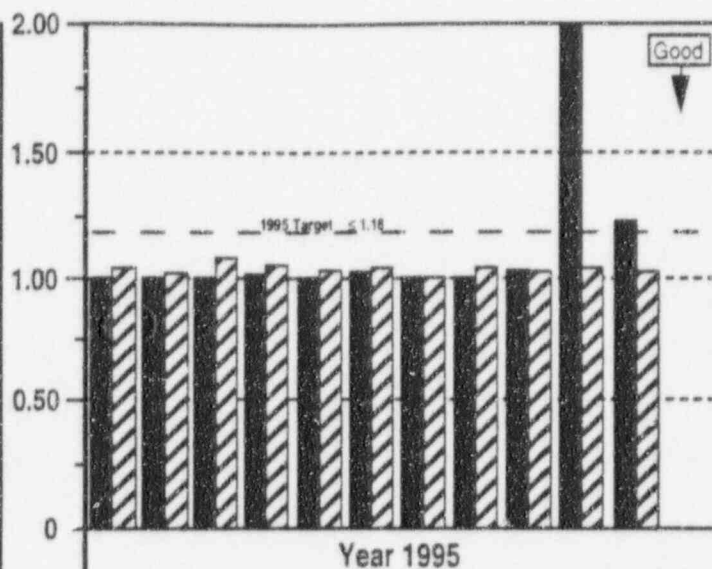
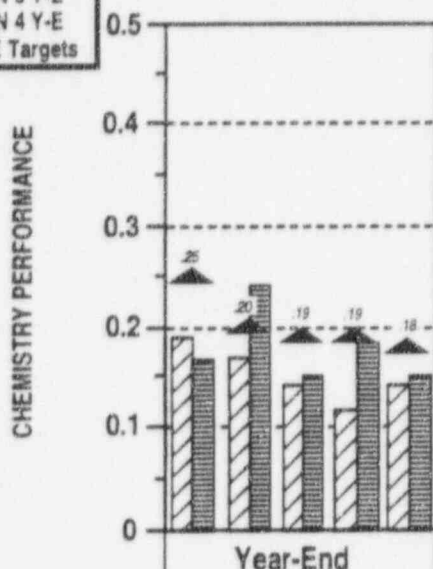
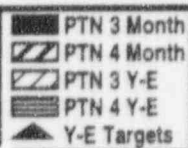
☐ NRC
 ☒ INPO
 ☐ Corp
 ☒ Div

COLLECTIVE RADIATION EXPOSURE



SECONDARY CHEMISTRY PERFORMANCE

TURKEY POINT UNITS 3 & 4



Legend / Period	1990	1991	1992	1993	1994
PTN 3	.19 [#]	.17	.14	.12	.14
PTN 4	.17 [#]	.24	.15	.18	.15

Jan	Feb	Mar	Apr	May	Jun	Jul	Aug	Sep	Oct	Nov	Dec
1.00	1.00	1.00	1.01	1.00	1.02	1.00	1.01	1.03	2.80	1.23	
1.07	1.02	1.08	1.05	1.03	1.04	1.00	1.04	1.02	1.07	1.02	

Definition through 1994

Secondary Chemistry Performance index (CPI) is calculated as follows:

$$\frac{\frac{KA + NA + O_2}{0.8 \quad 20 \quad 10^2}}{3}$$

KA = Monthly Average Steam Generator blowdown cation conductivity in $\mu\text{mhos/cm}$
 NA = Monthly Average Steam Generator blowdown sodium in ppb
 O₂ = Monthly Average condensate dissolved oxygen in ppb

Definition Beginning 1995

Secondary Chemistry Indicator (CI) is calculated as follows:

$$\frac{SGCl/LVx + SGSO_4/LVx + SGNa/LVx + FWFe/LVx + FWCu/LVx + O_2/LVx}{6}$$

SGCl = Steam Generator blowdown chloride
 SGSO₄ = Steam Generator blowdown sulfate
 SGNa = Steam Generator blowdown sodium
 FWFe = Final feedwater iron
 FWCu = Final feedwater copper
 O₂ = Condensate dissolved oxygen
 LVx = Limiting value for that parameter

SGCl LVx = 1.60 ppb
 SGSO₄ LVx = 1.70 ppb
 SGNa LVx = 0.80 ppb
 FWFe LVx = 5.00 ppb
 FWCu LVx = 0.20 ppb
 Cond. DO = 3.30 ppb

Statistical Summary

	Nov	Y-T-D Avg
Unit 3	1.23	1.19
Unit 4	1.02	1.04
Turkey Point Units 3 & 4 1995 Year-End Targets: ≤ 1.18		

Industry Performance

INPO (PWR's with Recirculating Steam Generators Not on Molar Ratio Control)	CI
July 1994 - June 1995 U. S. Median	1.19
1995 Lowest Chemistry Index Value Attainable	1.00

Performance Summary

Turkey Point Unit 3's Chemistry Performance exceeded target due to residual contaminants associated with the startup of Fuel Cycle 15. Year-to-date, Unit 3 slightly exceeded the Year-End target of ≤ 1.18 but was equal to the Industry Median of 1.19.

Unit 4's year-to-date Chemistry Performance of 1.04 was below the Year-End target of ≤ 1.18 and the Industry Median of 1.19.

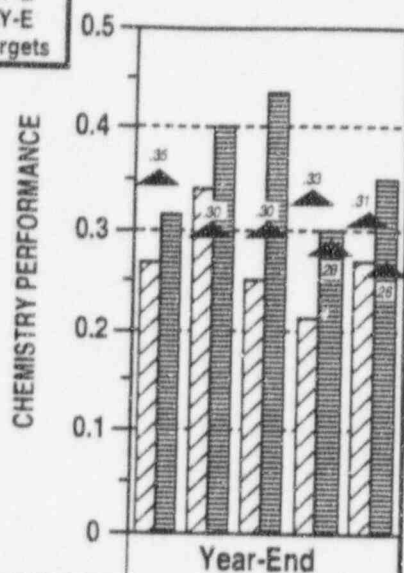
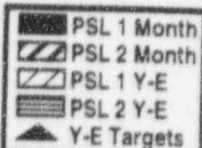
Data Source: Plant Chemistry
Contact: J. Seager 694-4176

Indicator Type

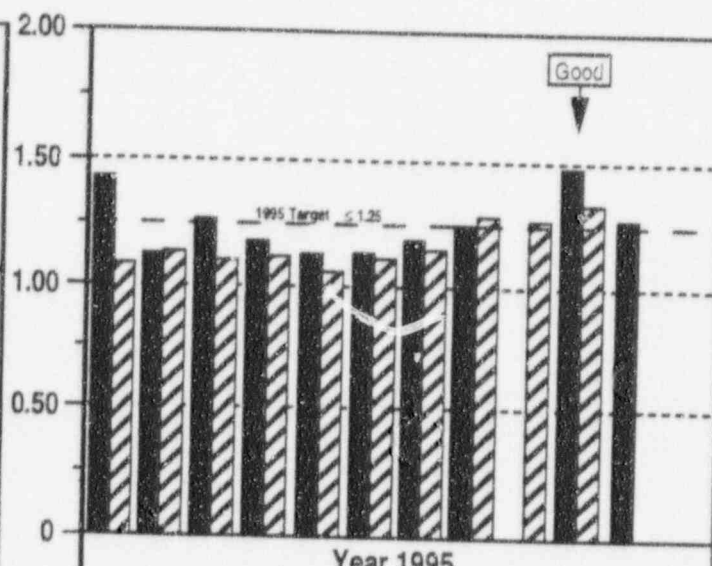
☐ NRC ☒ INPO ☐ Corp ☐ Div

SECONDARY CHEMISTRY PERFORMANCE

ST. LUCIE UNITS 1 & 2



Legend / Period	1990	1991	1992	1993	1994
PSL 1	.27	.34	.25	.22	.24
PSL 2	.32	.40	.43	.30	.37



Month	Jan	Feb	Mar	Apr	May	Jun	Jul	Aug	Sep	Oct	Nov	Dec
CI	1.43	1.13	1.26	1.18	1.13	1.14	1.19	1.25	*	1.49	1.28	
LVx	1.09	1.14	1.10	1.12	1.06	1.12	1.15	1.29	1.27	1.34	*	

Definition through 1994

Secondary Chemistry Performance index (CPI) is calculated as follows:

$$\frac{KA + NA + O_2}{0.8 \cdot 20 \cdot 10^2} \cdot 3$$

KA = Monthly Average Steam Generator blowdown cation conductivity in $\mu\text{mhos/cm}$
 NA = Monthly Average Steam Generator blowdown sodium in ppb
 O₂ = Monthly Average condensate dissolved oxygen in ppb

Definition Beginning 1995

Secondary Chemistry Indicator (CI) is calculated as follows:

$$\frac{SGCl/LVx + SGSO_4/LVx + SGNa/LVx + FWFe/LVx + FWCu/LVx + O_2/LVx}{6}$$

SGCl = Steam Generator blowdown chloride
 SGSO₄ = Steam Generator blowdown sulfate
 SGNa = Steam Generator blowdown sodium
 FWFe = Final feedwater iron
 FWCu = Final feedwater copper
 O₂ = Condensate dissolved oxygen
 LVx = Limiting value for that parameter

SGCl LVx = 1.60 ppb
 SGSO₄ LVx = 1.70 ppb
 SGNa LVx = 0.80 ppb
 FWFe LVx = 5.00 ppb
 FWCu LVx = 0.20 ppb
 Cond. DO = 3.30 ppb

Statistical Summary

	Nov	Y-T-D Avg
Unit 1	1.28	1.25
Unit 2	0.00	1.17

St. Lucie Units 1 & 2
 1995 Year-End Targets: ≤ 1.25

Industry Performance

INPO (PWR's with Recirculating Steam Generators Not on Molar Ratio Control)	CI
July 1994 - June 1995 U. S. Median	1.19
1995 Lowest Chemistry Index Value Attainable	1.00

Performance Summary

St. Lucie Unit 1 Chemistry Performance exceeded the target in November due to elevated condensate dissolved oxygen and higher levels of sodium contaminant which appear to still be present from the turbine work performed during the last refueling. Year-to-date, both units were at or below the Year-End targets of ≤ 1.25 .

Unit 2 was shutdown for refueling.

* Refueling Outage

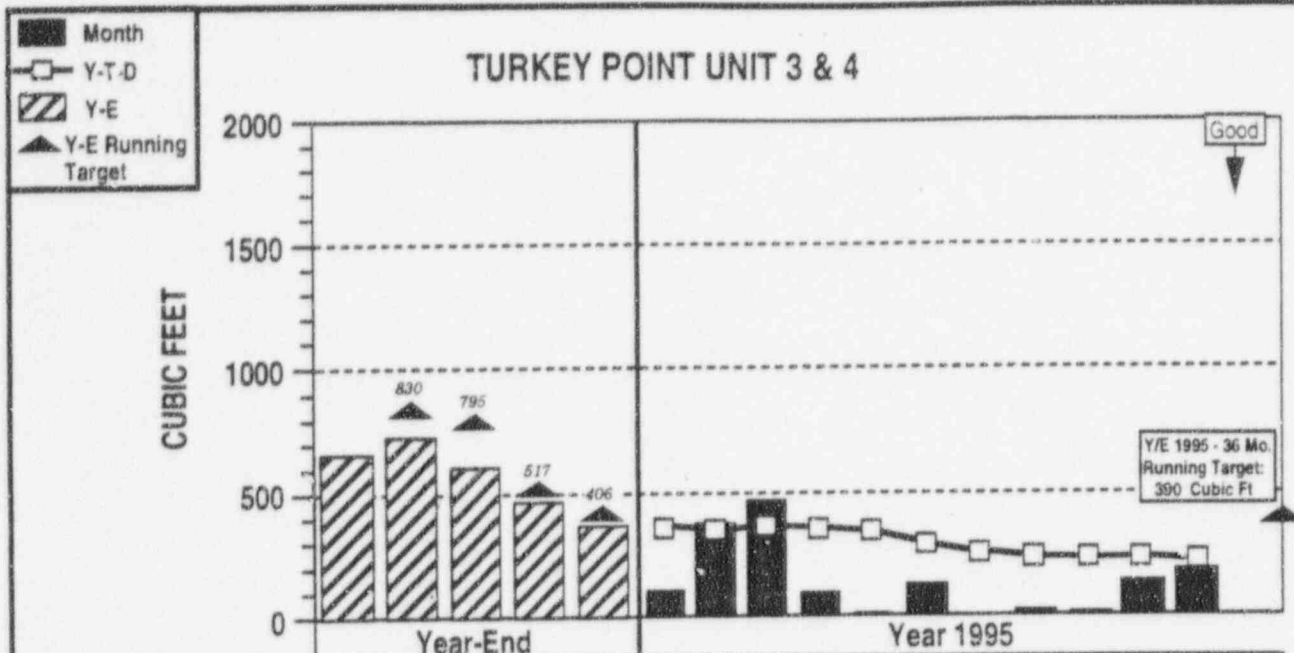
Data Source: Plant Chemistry
 Contact: J. Seager 694-4176

Indicator Type

☐ NRC ☒ INPO ☐ Corp ☐ Div

SOLID WASTE DISPOSED

TURKEY POINT UNIT 3 & 4



Legend / Period	1990	1991	1992	1993	1994	Jan	Feb	Mar	Apr	May	Jun	Jul	Aug	Sep	Oct	Nov	Dec
Month						105	380	488	99.1	3.5	131	0	18.7	13.8	135	194	
36 Mo. Running Avg	673	738	602	478	373	366	364	367	360	356	293	263	241	236	236	231	

Definitions

Solid Waste Disposed is the volume of low-level solid radioactive waste shipped, in final form ready for burial during a given period.

* The value is based, in part, on initial vendor and plant reports and may be updated when reports are documented.

Statistical Summary

	Nov	36 Mo. Running
Solid Waste Disposed - 1995	193.8	230.9
Solid Waste Disposed - 1994	79.3	363.9

Target:

Year-End 1995 - 36 Mo. Running Avg: 390 cu.ft.

Industry Performance

Industry (PWR's)	Monthly Cubic Feet (2 Unit Site)
Median 3-yr Distribution (7/92 - 6/95)	364.9
1994 Median	270.7
1995 Goal	647.4

Performance Summary

Turkey Point Solid Waste Disposed in November was 193.8. The 36-Month Running average of 230.9 cubic feet was below the 1995 Year-End target and 3-yr. Industry Median.

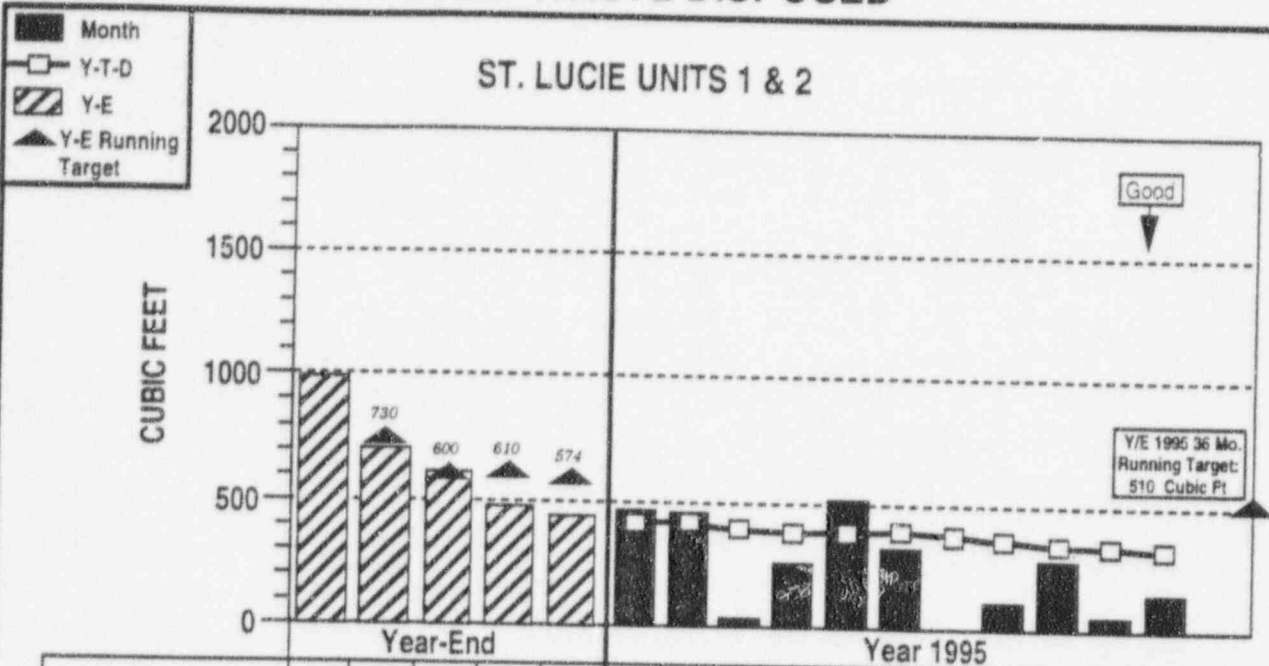
Data Source: Nuclear Division Health Physics
Contact: A. J. Gould, 694-4199

Indicator Type

☐ NRC ☒ INPO ☐ Corp ☐ Div

SOLID WASTE DISPOSED

ST. LUCIE UNITS 1 & 2



Legend / Period	1990	1991	1992	1993	1994	Jan	Feb	Mar	Apr	May	Jun	Jul	Aug	Sep	Oct	Nov	Dec
Month						480	468	37.5	268	528	323	0	106	287	53.6	150	
36 Mo. Running Avg	991	713	611	475	444	413	418	398	384	387	389	378	367	342	331	321	

Definitions

Solid Waste Disposed is the volume of low-level solid radioactive waste shipped, in final form ready for burial during a given period.

* The value is based, in part, on initial vendor and plant reports and may be updated when reports are documented.

Statistical Summary

	Nov	36 Mo. Running
Solid Waste Disposed - 1995	149.7	321.3
Solid Waste Disposed - 1994	49.1	451.0

Target:

Year-End 1995 - 36 Mo. Running Avg: 510 cu. ft.

Industry Performance

Industry (PWR's)	Monthly Cubic Feet (2 Unit Site)
Median 3-yr Distribution (7/92 - 6/95)	364.9
1994 Median	270.7
1995 Goal	647.4

Performance Summary

St. Lucie Solid Waste Disposed in November was 149.7. The 36-Month Running average of 321.3 cubic feet was below the 1995 Year-End target and the 3-yr. Industry Median.

Data Source: Nuclear Division Health Physics
 Contact: A. J. Gould, 694-4199

Indicator Type

☐ NRC

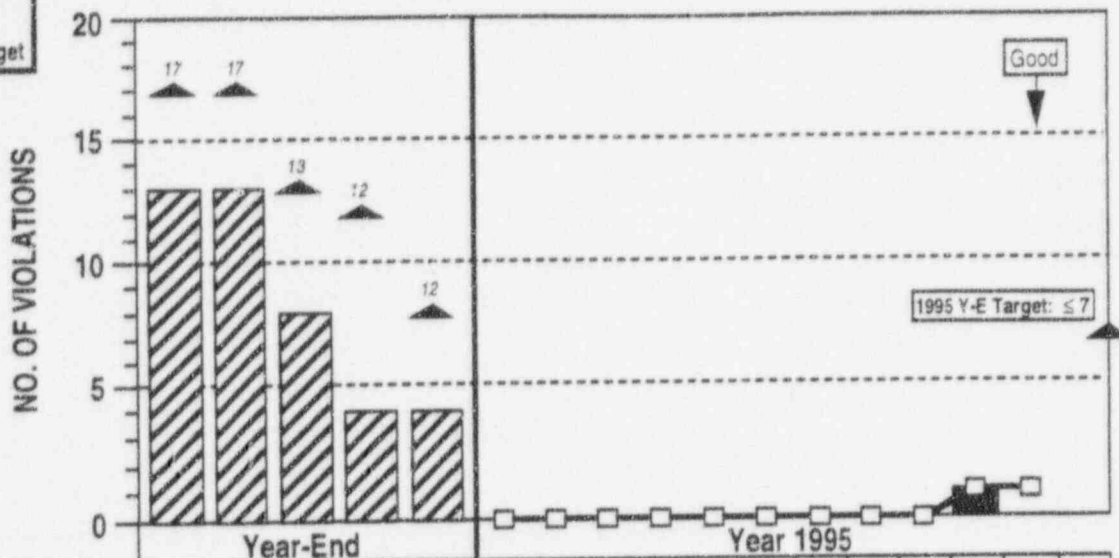
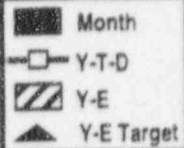
☒ INPO

☐ Corp

☐ Div

NRC VIOLATIONS

TURKEY POINT 3 & 4



Legend / Period	1990	1991	1992	1993	1994	Jan	Feb	Mar	Apr	May	Jun	Jul	Aug	Sep	Oct	Nov	Dec
Month						0	0	0	0	0	0	0	0	0	1	0	
Y-T-D	13	13	8	4	4	0	0	0	0	0	0	0	0	0	1	1	

Definitions

Violations are categorized in terms of five levels of severity to show their relative importance. Severity Levels I and II are violations that involve actual or high potential impact on the public. Severity Level III violations are cause for significant concern. Severity Level IV violations are less serious, but are of more than minor concern; i.e., if left uncorrected, could lead to a more serious concern. Severity Level V violations are of minor safety or environmental concern. Violations are counted on the date of the inspection exit meeting.

Violations are now counted with respect to the date of occurrence (using the date of the inspection exit meeting) instead of the date of the inspection report, as was done in the past.

Statistical Summary

	Nov	YTD
NOV's - 1995	0	1
NOV's - 1994	2	4
Targets:		
Year End - 1995	≤ 7	
Year End - 1994	≤ 7	

Industry Performance

NRC Violations

1994 IBG Group Mean	16.2
1994 Region II Mean	13.1
1994 IBG Top Quartile Mean	9.0
1994 Region II Top Quartile Entry	8.0

Source: Nuclear Data Services Database 10/95

Performance Summary

Turkey Point reported no NRC Violations in November:

Year-to-date, Turkey Point has received one NRC Violation.

#95-16-02 - Failure to Promptly Identify and Take Appropriate Corrective Action to Resolve Potential Unit 3 CCW Heat Exchanger Flow Related Vulnerability. Exit Meeting Date: 10/14/95.

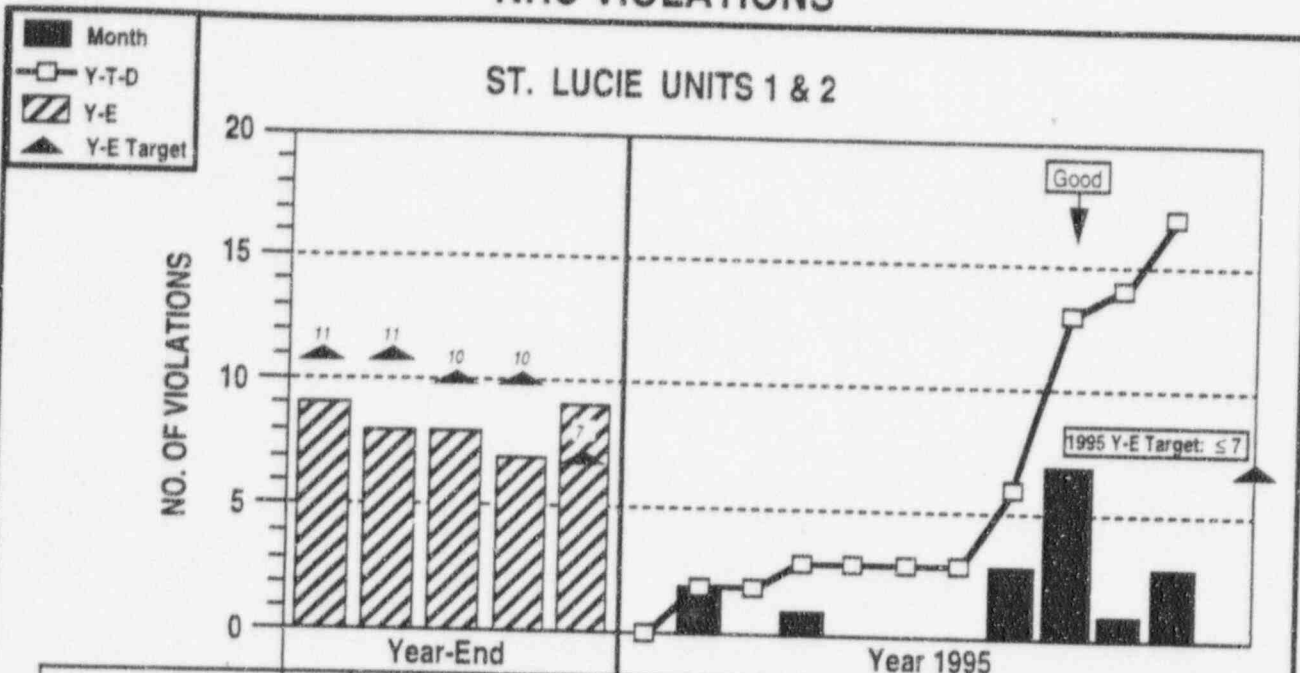
Data Source: PTN Nuclear Licensing
Contact: E. Weinkam, 246-7383

Indicator Type

☐ NRC
 ☐ INPO
 ☐ Corp
 ☐ Div

NRC VIOLATIONS

ST. LUCIE UNITS 1 & 2



Legend / Period	1990	1991	1992	1993	1994	Jan	Feb	Mar	Apr	May	Jun	Jul	Aug	Sep	Oct	Nov	Dec
Month						0	2	0	1	0	0	0	3	7	1	3	
Y-T-D	9	8	8	7	9	0	2	2	3	3	3	3	6	13	14	17	

Definitions

Violations are categorized in terms of five levels of severity to show their relative importance. Severity Levels I and II are violations that involve actual or high potential impact on the public. Severity Level III violations are cause for significant concern. Severity Level IV violations are less serious, but are of more than minor concern; i.e., if left uncorrected, could lead to a more serious concern. Severity Level V violations are of minor safety or environmental concern. Violations are counted on the date of the inspection exit meeting.

Violations are now counted with respect to the date of occurrence (using the date of the inspection exit meeting) instead of the date of the inspection report, as was done in the past.

Statistical Summary

	Nov	YTD
NOV's - 1995	3	17
NOV's - 1994	2	8
Targets:		
Year End - 1995	≤ 7	
Year End - 1994	≤ 7	

Industry Performance

NRC Violations	
1994 IBG Group Mean	16.2
1994 Region II Mean	13.1
1994 IBG Top Quartile Mean	9.0
1994 Region II Top Quartile Entry	8.0

Source: Nuclear Data Services Database 10/95

Performance Summary

St. Lucie reported three (*) NRC Violations for November. The following 17 NRC Violations have been received Y-T-D:

- #95-01-01 - Failure to Sample Safety Injection Tank in Accordance With Tech Specification Surveillance. Exit Mtg Date: 2/4/95.
- #95-01-02 - Failure to Follow Procedure in Implementing Independent Verification Associated With the Letdown Letter of Instruction. Exit Mtg Date: 2/4/95.
- #95-05-01 - Failure to Meet Design Control Measures on Overload Heaters Installed in a Battery Charger. Exit Mtg Date: 4/28/95.
- #95-15-01 - Failure to Follow Procedures and Block MSIS Actuation. Exit Mtg Date: 9/15/95.
- #95-15-02 - Two Examples of Failure to Follow Procedures During RCP Restaging. Exit Mtg Date: 9/15/95.
- #95-15-03 - Failure to Follow Procedure and Document Abnormal Valve Alignment in Valve Deviation Log. Exit Mtg Date: 9/15/95.
- #95-15-04 - Failure to Follow Procedures During Alignment of Shutdown Cooling System. Exit Mtg Date: 9/15/95.
- #95-15-05 - Failure to Follow Procedure and Document a Deficiency on a Containment Spray Valve Test Procedure. Exit Mtg Date: 9/15/95.
- #95-15-06 - Failure to Initial Maintenance Steps as Work was Completed. Exit Mtg Date: 9/15/95.
- #95-15-07 - Failure to Follow Procedures Resulting in Spray Down on Containment. Exit Mtg Date: 9/15/95.
- #95-16-01 - Post Maintenance Testing Inadequate on PORV's (Not IAW 10CFR50 App. B, Criteria). Exit Mtg Date: 8/30/95.
- #95-16-02 - In-Service Surveillance Testing Inadequate on PORV's (Not IAW 10CFR50 App. B, Criteria). Exit Mtg Date: 8/30/95.
- #95-16-03 - LTOP Operability Tech Spec was Violated by not Having the PORV's Operable When Required During Two Previous Outages. Exit Mtg Date: 8/30/95.
- #95-18-01 - Failure to Follow Procedure and Not Properly Log Locked Key-Switch Deviations. Exit Mtg Date: 11/1/95.
- #95-18-02 - Failure to Follow Procedure and Issue a Clearance, Which Resulted in Personnel Injury. Exit Mtg Date: 11/1/95.
- #95-18-03 - Inadequate Design of Unit 2 EDG Resulting in Reverse Powering of the EDG During ESFAS Testing. Exit Mtg Date: 11/1/95.
- #95-20-01 - Failure to Take Prompt Corrective Actions for Relief Valve Deficiencies, Paragraph 2. Exit Mtg Date: 10/10/95.

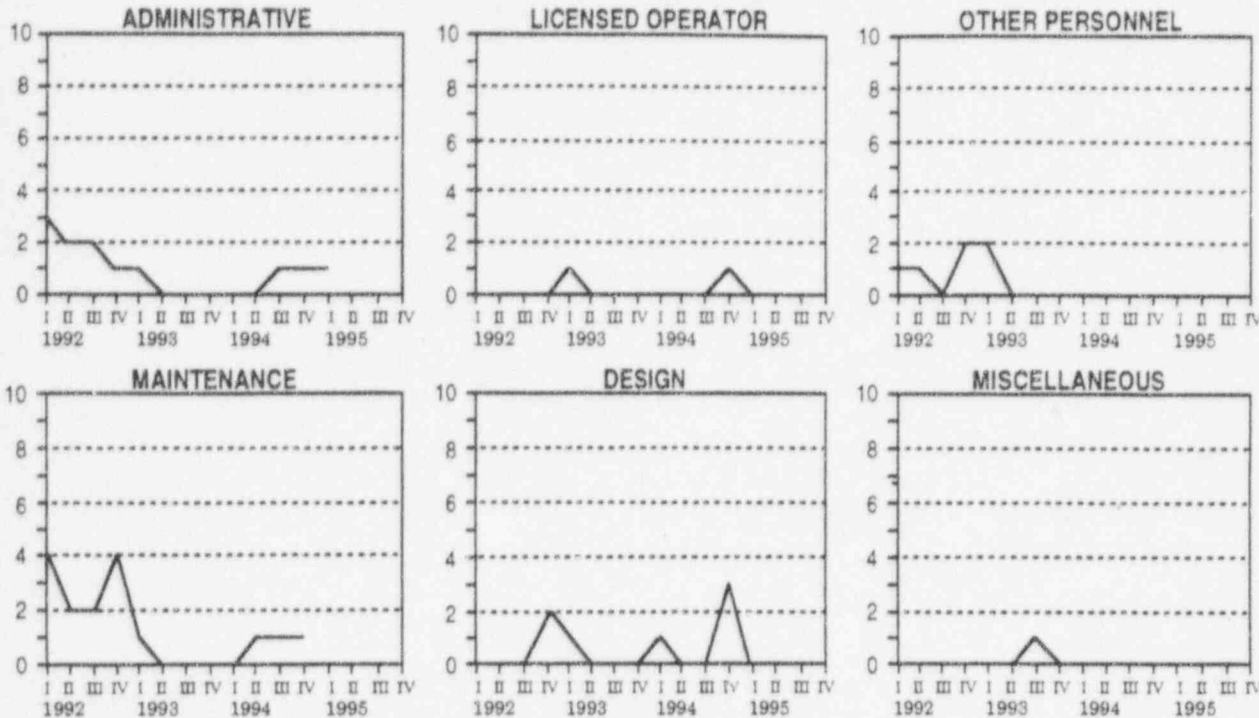
Data Source: PSL Nuclear Licensing
 Contact: P. E. Dawson, 465-4107

Indicator Type

☐ NRC ☐ INPO ☐ Corp ☐ Div

LICENSEE EVENT REPORTS BY CAUSE CODES

TURKEY POINT UNIT 3



Definitions

Licensee Event Reports (LER) are submitted to the NRC by licensees to report the unusual occurrences prescribed by 10CFR50.73. Cause Codes are intended to identify possible programmatic deficiencies. The cause code trend data are developed using the NRC's Sequence Coding Search System (SCSS) database. This database is developed from all LER's and lags other performance data by as much as two quarters.

Statistical Summary

Cause Codes	Qtr 95-1	Running 4 Qtrs
Administrative	1	3
Licensed Operator	0	1
Other Personnel	0	0
Maintenance	0	3
Design	0	3
Miscellaneous	0	0
Total	1	10

Industry Performance

NRC Cause Codes	Qtr 95-1	Industry Average Running 4 Qtrs
Administrative	0.7	5.5
Licensed Operator	0.2	1.0
Other Personnel	0.2	2.0
Maintenance	0.9	7.4
Design	0.5	3.1
Miscellaneous	0.1	0.4
Total	2.5	19.4

Performance Summary

Unit 3 performance for the 1995 1st Quarter was below the 1st Quarter industry average for all cause codes except Administrative. Unit 3 performance for the last 4 Quarters was below the 4 Quarter Industry Average for all cause codes.

Unit 3 reported one LER in November:

LER #95-007-00 entitled "Manual Reactor Trip Following Drop of Four Control Rods" dated November 9, 1995. The event occurred on October 17, 1995.

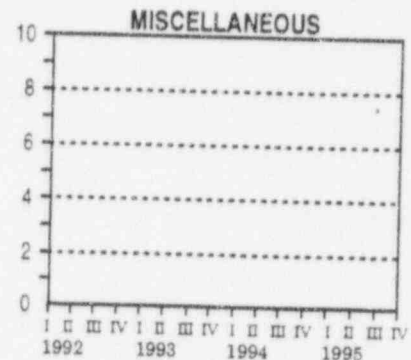
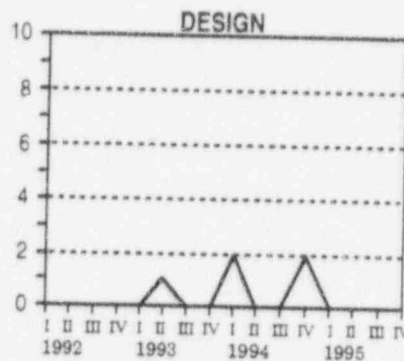
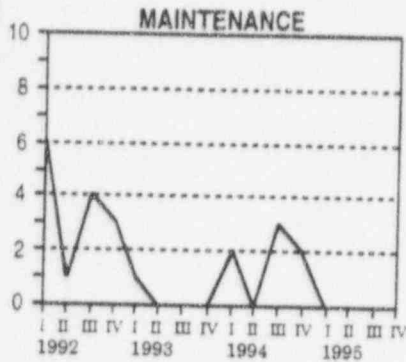
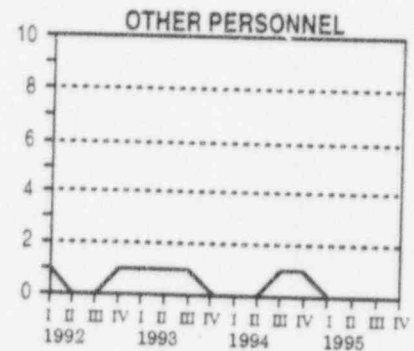
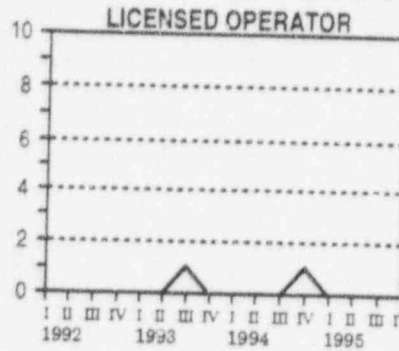
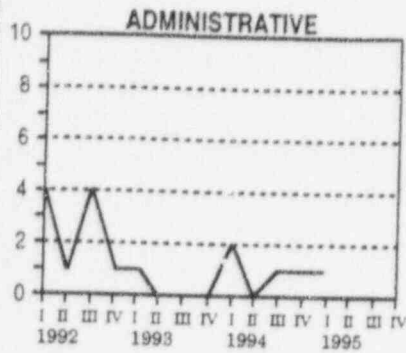
Data Source: NRC (AEOD) Quarterly Performance Indicators Report

Indicator Type

☒ NRC ☐ INPO ☐ Corp ☐ Div

LICENSEE EVENT REPORTS BY CAUSE CODES

TURKEY POINT UNIT 4



Definitions

Licensee Event Reports (LER) are submitted to the NRC by licensees to report the unusual occurrences prescribed by 10CFR50.73. Cause Codes are intended to identify possible programmatic deficiencies. The cause code trend data are developed using the NRC's Sequence Coding Search System (SCSS) database. This database is developed from all LER's and lags other performance data by as much as two quarters.

Statistical Summary

Cause Codes	Qtr 95-1	Running 4 Qtrs
Administrative	1	3
Licensed Operator	0	1
Other Personnel	0	2
Maintenance	0	5
Design	0	2
Miscellaneous	0	0
Total	1	13

Industry Performance

NRC		
Cause Codes	Qtr 95-1	Running 4 Qtrs
Administrative	0.7	5.5
Licensed Operator	0.2	1.0
Other Personnel	0.2	2.0
Maintenance	0.9	7.4
Design	0.5	3.1
Miscellaneous	0.1	0.4
Total	2.5	19.4

Performance Summary

Unit 4 performance for the 1st Quarter of 1995 was below the 1st Quarter industry average for all cause codes except Administrative. For the past 4 Quarters, Unit 4 performance was below the 4 Quarter Industry Average for all cause codes.

Unit 4 reported no LER's in November.

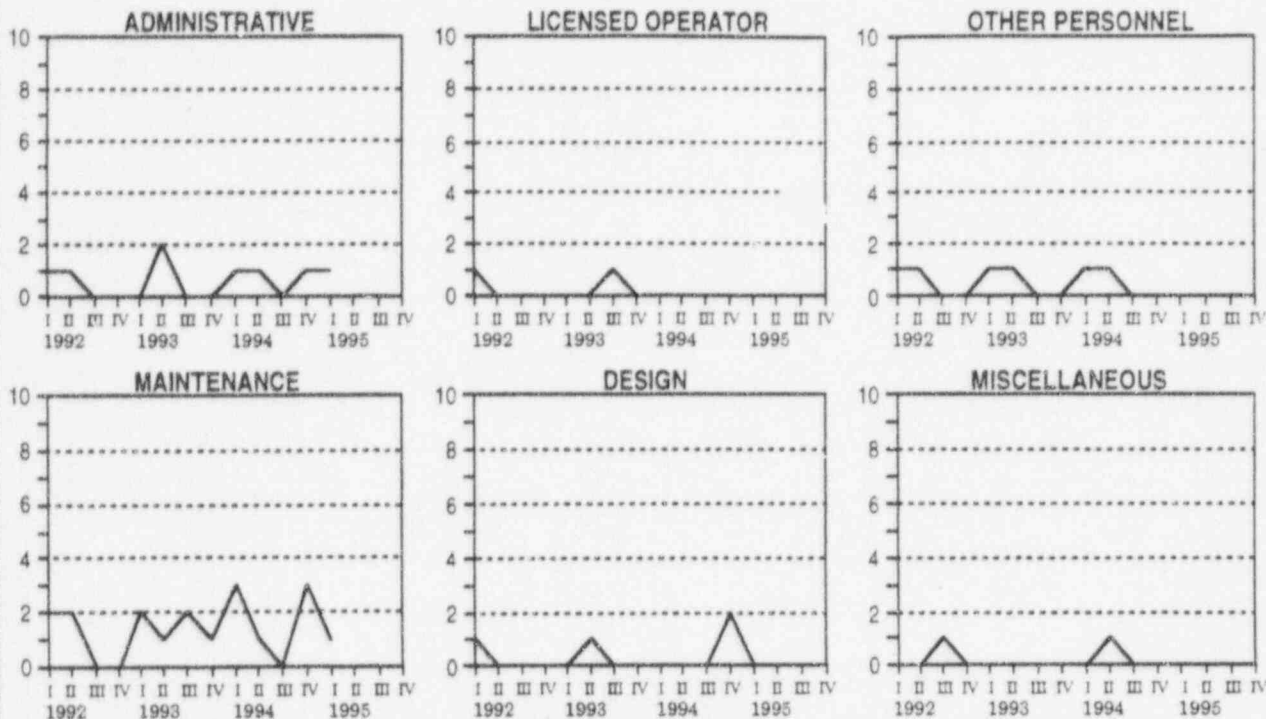
Data Source: NRC (AEOD) Quarterly Performance Indicators Report

Indicator Type

☒ NRC ☐ INPO ☐ Corp ☐ Div

LICENSEE EVENT REPORTS BY CAUSE CODES

ST. LUCIE UNIT 1



Definitions

Licensee Event Reports (LER) are submitted to the NRC by licensees to report the unusual occurrences prescribed by 10CFR50.73. Cause Codes are intended to identify possible programmatic deficiencies. The cause code trend data are developed using the NRC's Sequence Coding Search System (SCSS) database. This database is developed from all LER's and lags other performance data by as much as two quarters.

Statistical Summary

Cause Codes	Qtr 95-1	Running 4 Qtrs
Administrative	1	3
Licensed Operator	0	0
Other Personnel	0	1
Maintenance	1	5
Design	0	2
Miscellaneous	0	1
Total	2	12

Industry Performance

<u>NRC</u>	<u>Industry Average</u>	
<u>Cause Codes</u>	<u>Qtr 95-1</u>	<u>Running 4 Qtrs</u>
Administrative	0.7	5.5
Licensed Operator	0.2	1.0
Other Personnel	0.2	2.0
Maintenance	0.9	7.4
Design	0.5	3.1
Miscellaneous	<u>0.1</u>	<u>0.4</u>
Total	2.5	19.4

Performance Summary

Unit 1 performance for the 1st Quarter of 1995 was below the 1st Qtr industry average for all cause codes except Administrative and Maintenance. For the past 4 Quarters, Unit 1 performance was below the 4 Quarter Industry Average for all cause codes except Miscellaneous.

Unit 1 reported one LER in November:

#95-009-00 entitled "Missed Technical Specification Scheduled Surveillance Due to Personnel Error" dated November 18, 1995. The event occurred October 19, 1995.

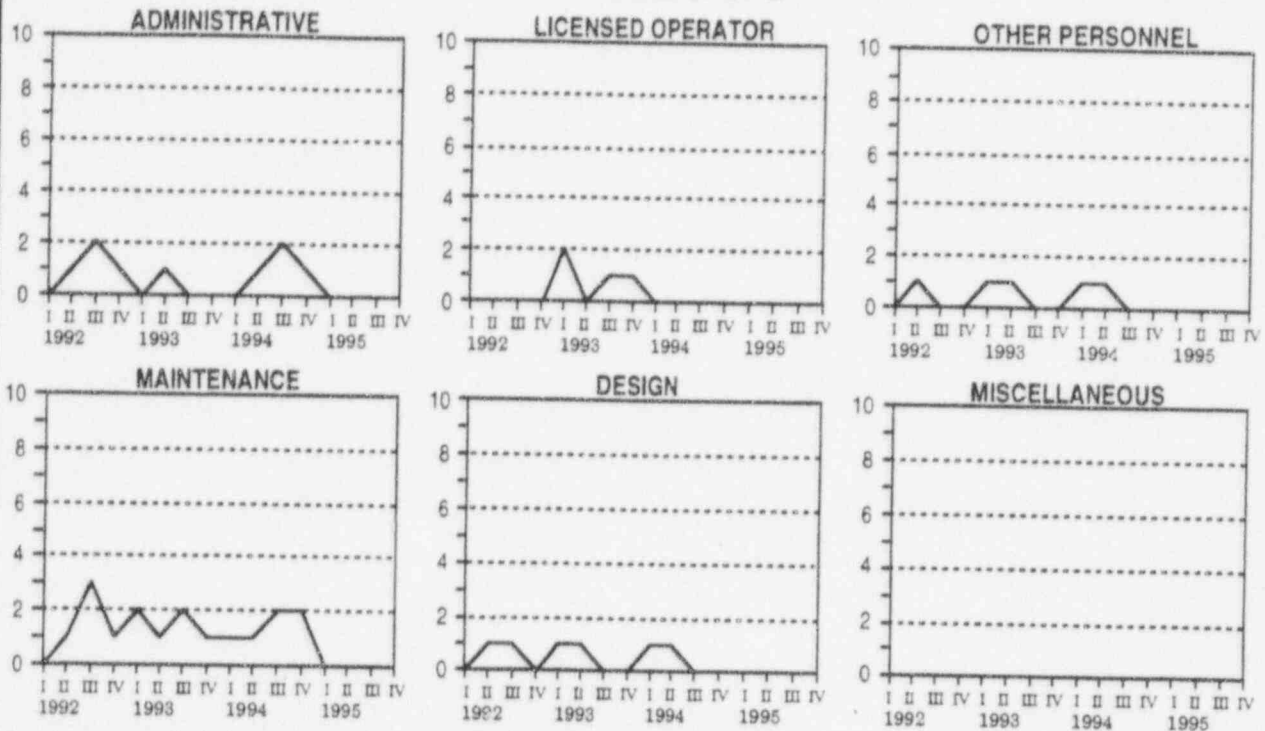
Data Source: NRC (AEOD) Quarterly Performance Indicators Report

Indicator Type

☒ NRC ☐ INPO ☐ Corp ☐ Div

LICENSEE EVENT REPORTS BY CAUSE CODES

ST. LUCIE UNIT 2



Definitions

Licensee Event Reports (LER) are submitted to the NRC by licensees to report the unusual occurrences prescribed by 10CFR50.73. Cause Codes are intended to identify possible programmatic deficiencies. The cause code trend data are developed using the NRC's Sequence Coding Search System (SCSS) database. This database is developed from all LER's and lags other performance data by as much as two quarters.

Statistical Summary

Cause Codes	Qtr 95-1	Running 4 Qtrs
Administrative	0	4
Licensed Operator	0	0
Other Personnel	0	1
Maintenance	0	5
Design	0	1
Miscellaneous	0	0
Total	0	11

Industry Performance

NRC Cause Codes	Qtr 95-1	Industry Average Running 4 Qtrs
Administrative	0.7	5.5
Licensed Operator	0.2	1.0
Other Personnel	0.2	2.0
Maintenance	0.9	7.4
Design	0.5	3.1
Miscellaneous	0.1	0.4
Total	2.5	19.4

Performance Summary

Unit 2 performance for the 1st Quarter of 1995 and Running 4 Quarters was better the Industry Average for all cause codes.

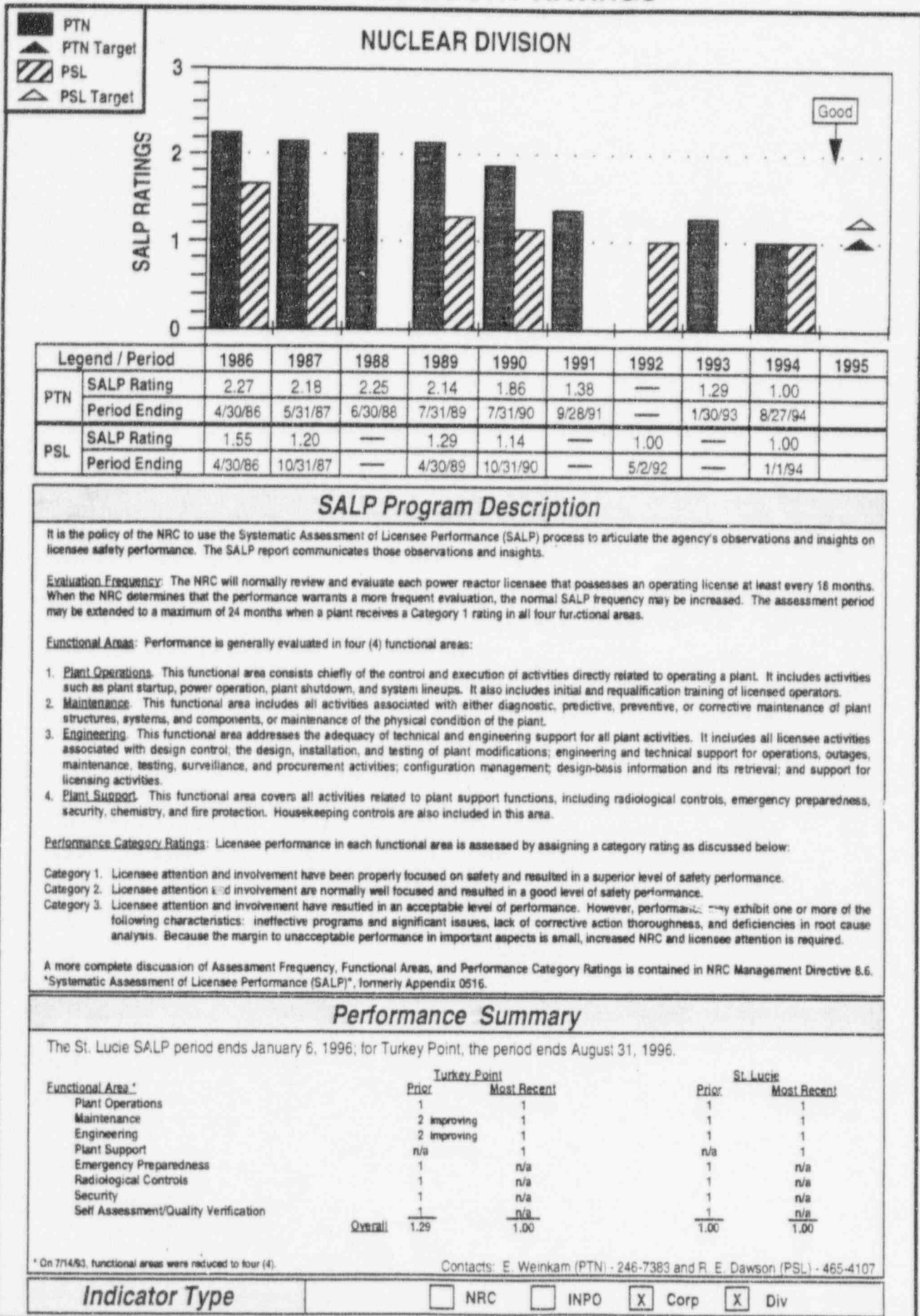
Unit 2 reported no LER's in November.

Data Source: NRC (AEOD) Quarterly Performance Indicators Report

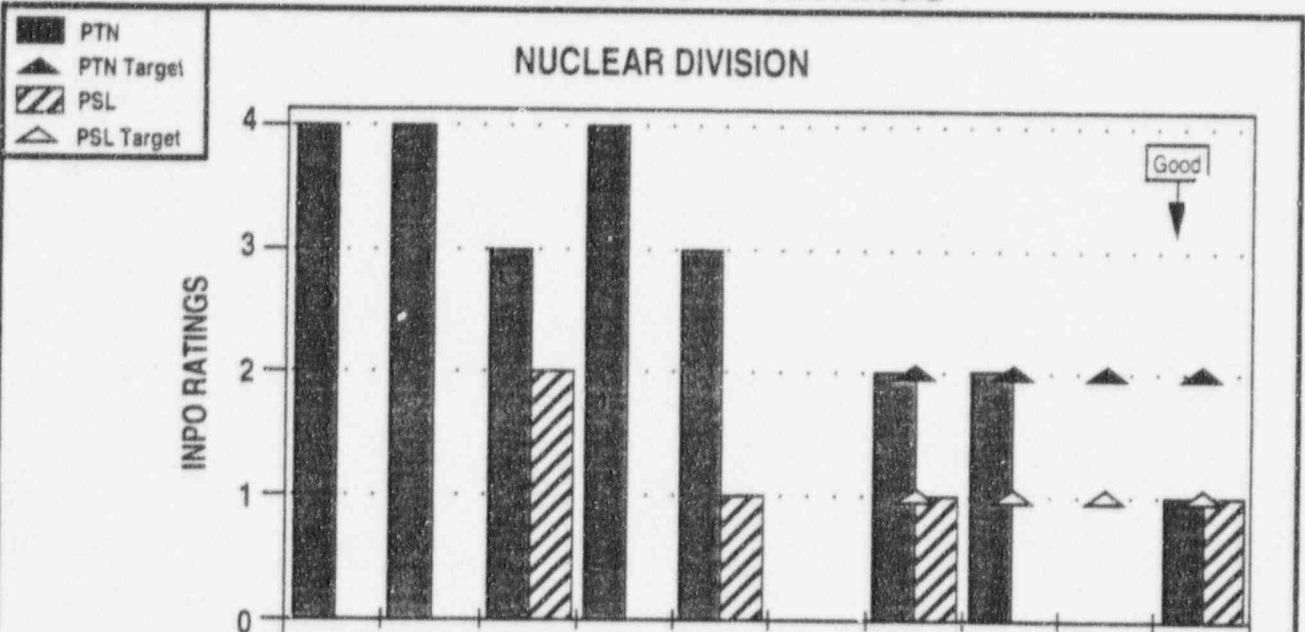
Indicator Type

☒ NRC ☐ INPO ☐ Corp ☐ Div

SALP CATEGORY RATINGS



INPO ASSESSMENT RATINGS



Legend / Period	1986	1987	1988	1989	1990	1991	1992	1993	1994	1995
PTN	4	4	3	4	3	—	2	2	—	1
PSL	—	—	2	—	1	—	1	—	—	1

INPO Assessment Program Description

The Institute of Nuclear Power Operations (INPO) conducts periodic evaluations of site activities to make an overall determination of plant safety, to evaluate management systems and controls, and to identify areas needing improvement. Information is assembled from discussions, interviews, observations, and reviews of documentation.

Evaluation Frequency: INPO's goal is to visit each plant on an average of every 18 months. However, this frequency may vary depending upon the previous assessment ratings. For instance, if a plant is rated a "1" or "2", the interval between assessments is usually 20-24 months; if a plant is rated a "4" or a "5", the assessment interval is ≤ 18 months.

Performance Category Ratings:

- Category 1: Overall performance is excellent. Industry standards of excellence are met in most areas. No significant weaknesses noted.
- Category 2: Overall performance is exemplary. Industry standards of excellence are met in many areas. No significant weaknesses noted.
- Category 3: Overall performance is generally in keeping with the high standards required in nuclear power. However, improvements are needed in a number of areas. A few significant weaknesses may exist.
- Category 4: Overall performance is acceptable, but improvements are needed in a wide range of areas. Significant weaknesses are noted in several areas.
- Category 5: Overall performance does not meet the industry standard of acceptable performance. The margin of nuclear safety is measurably reduced. Strong and immediate management action to correct deficiencies is required. Special attention, assistance, and follow-up are required.

NOTE: If a plant is found to be operating without an adequate margin of nuclear safety, INPO will request that the plant be shutdown, or not started up.

Performance Summary

Turkey Point and St. Lucie received an INPO category rating of "1" in 1995. The next evaluations for Turkey Point and St. Lucie are expected in the last quarter of 1996 and first quarter of 1997, respectively.

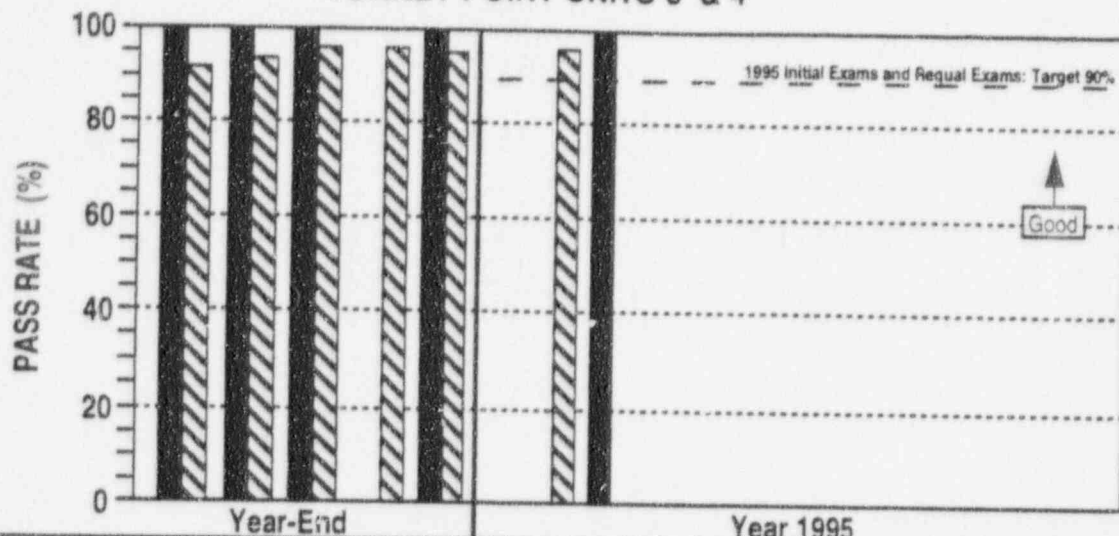
Contacts: E. Weinkam (PTN) - 246-7383 and R. E. Dawson (PSL) - 465-4107

Indicator Type

☐ NRC ☐ INPO ☐ Corp ☒ Div

OPERATOR EXAMINATION PERFORMANCE

TURKEY POINT UNITS 3 & 4



Legend/ Exams		1990	1991	1992	1993	1994	Year 1995											
							Jan	Feb	Mar	Apr	May	Jun	Jul	Aug	Sep	Oct	Nov	Dec
Initial Exams RO/SRO License	No. Taken	3	39	5	0	8			13									
	No. Passed	3	39	5	0	8			13									
	Pass Rate	100%	100%	100%	0%	100%			100%									
Operator Requal Exams	No. Taken	49	47	78	69	63		58										
	No. Passed	45	44	75	66	60		56										
	Pass Rate	92%	94%	96%	96%	95%		96.5%										

Definitions

Initial License Examination (RO/SRO) results are reported for all candidates taking an Initial License Exam as conducted by the NRC.

Operator Requalification Examination results are reported for both RO's and SRO's. This examination is administered annually by the utility and may be jointly administered by the NRC. Retests of operators who failed examinations are not included.

Statistical Summary

Initial RO/SRO License Exams			
	No. Taken	No. Passed	% Pass Rate
YTD 1995	13	13	100%
YTD 1994	8	8	100%
Operator Requal Exams			
	No. Taken	No. Passed	% Pass Rate
YTD 1995	58	56	96.5%
YTD 1994	63	60	95.0%

Industry Performance

The NRC at their last Regional Training Managers Meeting (for fiscal year 1994) provided the following data:

Initial NRC Exams		NRC Requal Exams	
RO's Pass Rate	94.6%	RO's Pass Rate	91.0%
Instant SRO's Pass Rate	94.4%	SRO's Pass Rate	85.0%
Upgrade SRO's Pass Rate	94.7%	Average Overall	88.0%
Average Overall	94.6%		

Performance Summary

No exams were given at Turkey Point in November.

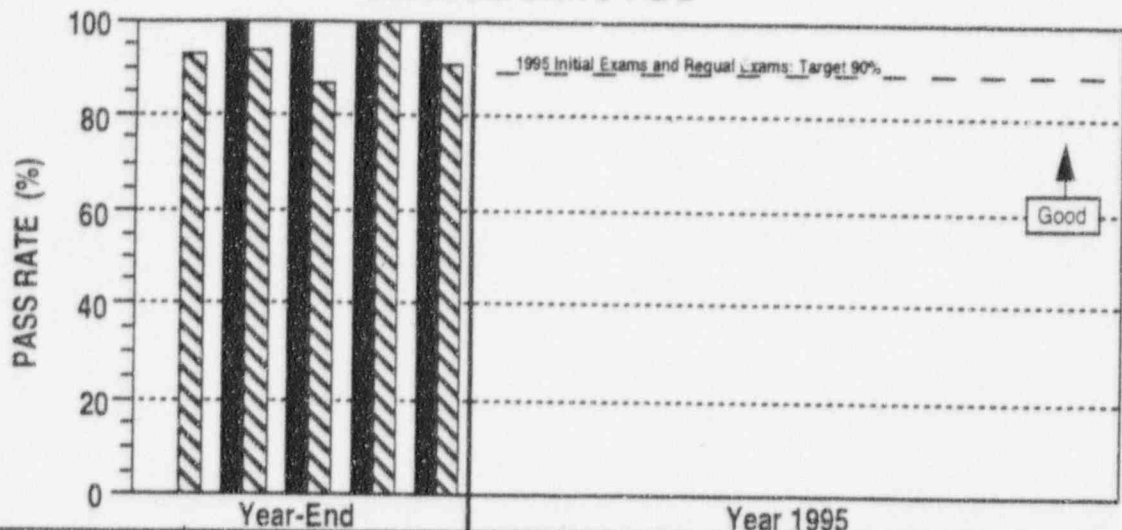
Data Source: Nuclear Training
Contact: K. E. Beatty, Manager, 694-4217

Indicator Type

☐ NRC ☐ INPO ☐ Corp ☒ Div

OPERATOR EXAMINATION PERFORMANCE

ST. LUCIE UNITS 1 & 2



Legend/ Exams		Year-End					Year 1995											
		1990	1991	1992	1993	1994	Jan	Feb	Mar	Apr	May	Jun	Jul	Aug	Sep	Oct	Nov	Dec
Initial Exams RO/SRO License	No. Taken	n/a	15	8	10	11												
	No. Passed	n/a	15	8	10	11												
	Pass Rate	n/a	100%	100%	100%	100%												
Operator Requal Exams	No. Taken	65	65	69	68	75												
	No. Passed	61	61	60	68	68												
	Pass Rate	93%	94%	87%	100%	91%												

Definitions

Initial License Examination (RO/SRO) results are reported for all candidates taking an Initial License Exam as conducted by the NRC.

Operator Requalification Examination results are reported for both RO's and SRO's. This examination is administered annually by the utility and may be jointly administered by the NRC. Retests of operators who failed examinations are not included.

Statistical Summary

Initial RO/SRO License Exams			
	No. Taken	No. Passed	% Pass Rate
YTD 1995	0	0	0%
YTD 1994	11	11	100%
<hr/>			
Operator Requal Exams			
	No. Taken	No. Passed	% Pass Rate
YTD 1995	0	0	0%
YTD 1994	75	68	91%

Industry Performance

The NRC at their last Regional Training Managers Meeting (for fiscal year 1994) provided the following data:

Initial NRC Exams

RO's Pass Rate	94.6%
Instant SRO's Pass Rate	94.4%
Upgrade SRO's Pass Rate	94.7%
Average Overall	94.6%

NRC Requal Exams

RO's Pass Rate	91.0%
SRO's Pass Rate	85.0%
Average Overall	88.0%

Performance Summary

No Initial RO/SRO License Exams were given in November at St. Lucie.

Annual Operator Requalification Exams are currently in progress. Results will be provided upon completion of the testing.

Data Source: Nuclear Training
Contact: K. E. Beatty, Manager, 694-4217

Indicator Type

☐ NRC

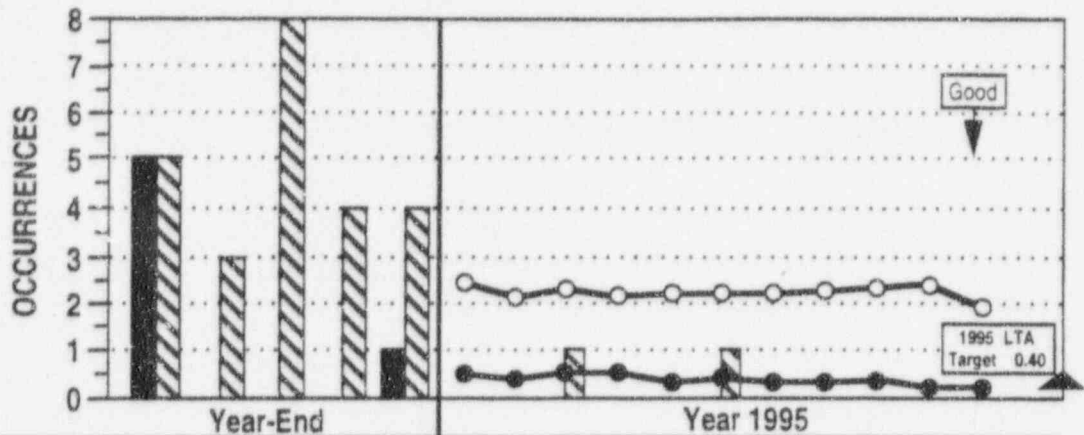
☐ INPO

☐ Corp

☒ Div

INDUSTRIAL SAFETY STATISTICS

TURKEY POINT UNITS 3 & 4



Legend / Period	1990	1991	1992	1993	1994	Jan	Feb	Mar	Apr	May	Jun	Jul	Aug	Sep	Oct	Nov	Dec
Lost Time Accidents	5	0	0	0	1	0	0	0	0	0	0	0	0	0	0	0	
Restricted Duty Cases	5	3	8	4	4	0	0	1	0	0	1	0	0	0	0	0	
OSHA Recordable Injuries	43	35	33	33*	27*	3	0	3	1	2	1	1	2	2	1	0	
INPO: LTA/Restricted Duty Cases 12-Mo. Average						.50	.40	.51	.51	.32	.43	.32	.32	.33	.22	.22	
* OSHA: Recordable Injuries Year-to-Date						2.48	2.14	2.28	2.19	2.21	2.21	2.21	2.33	2.39	2.44	1.95	

Definitions

For FPL employees permanently assigned to the site (OSHA Recordables by Payroll location only):

Lost Time Accident involves at least one full work day away from work following the day of the injury.

Restricted Duty involves at least one day of restricted or limited work activities by an injured employee.

OSHA Recordable Injury requires medical treatment beyond first aid.

INPO:

Lost Time Accident and Restricted Duty Case Rate = $\frac{\text{Lost Time Accidents and Restricted Duty} \times 200,000 \text{ Manhours}}{\text{Productive Manhours Worked}}$

OSHA:

Recordable Injury Rate = $\frac{\text{OSHA Recordable Injuries} \times 200,000 \text{ Manhours}}{\text{Productive Manhours Worked}}$

Statistical Summary

12 Mo - Ending (Nov)

LTA Rate - 1995 0.22
LTA Rate - 1994 0.47

Targets:

Year-End 1995 0.40
Year-End 1994 0.45

Industry Performance

INPO

Includes restricted duty cases:

July 1994 - June 1995 Median 0.51
1994 Average 0.64
1995 Goal 0.50

Performance Summary

Turkey Point reported no Lost Time Injuries or Restricted Duty Cases in November. Turkey Point's 12-Month Ending Lost Time Accident Rate of 0.22 was below the Year-End target and Industry Median.

* NOTE: OSHA Rate: The Turkey Point rate depicted excludes Juno Staff hours and incidents. The Corporate Safety Department publishes monthly Nuclear Division rates. The rates for this reporting period are:

Month (thru 11/95) 2.87
YTD (thru 11/95) 2.08
Corporate Target 3.10

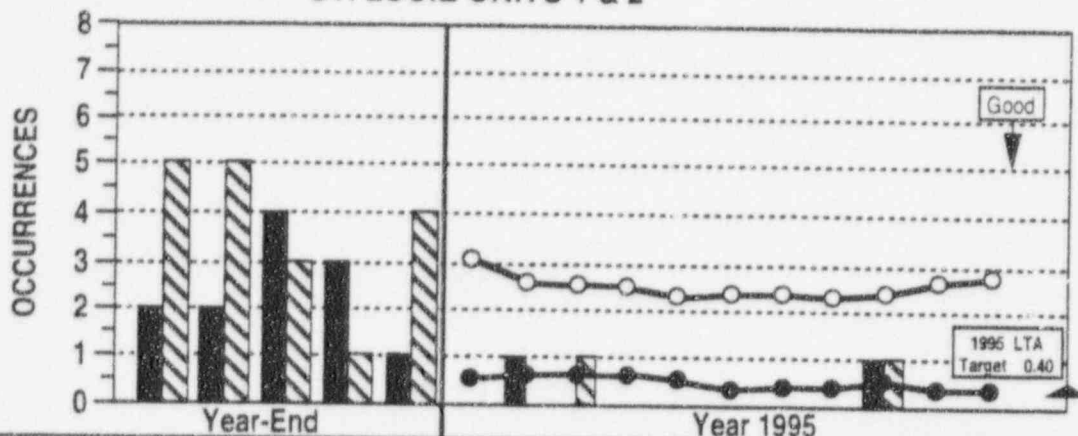
Source: Plant Safety Supervisor
Contact: W. Korte. 694-4235

Indicator Type

☐ NRC ☒ INPO ☒ Corp ☒ Div

INDUSTRIAL SAFETY STATISTICS

ST. LUCIE UNITS 1 & 2



Legend / Period	1990	1991	1992	1993	1994	Jan	Feb	Mar	Apr	May	Jun	Jul	Aug	Sep	Oct	Nov	Dec
Lost Time Accidents	2	2	4	3	1	0	1	0	0	0	0	0	0	1	0	0	
Restricted Duty Cases	5	5	3	1	4	0	0	1	0	0	0	0	0	1	0	0	
OSHA Recordable Injuries	33	20	15	19	23	2	0	2	1	1	3	1	2	3	3	7	
INPO: LTA/Restricted Duty Cases 12-Mo. Average						.51	.61	.62	.62	.52	.31	.42	.42	.51	.40	.40	
* OSHA: Recordable Injuries Year-to-Date						3.07	2.55	2.51	2.48	2.33	2.42	2.42	2.39	2.44	2.61	2.76	

Definitions

For FPL employees permanently assigned to the site (OSHA Recordables by Payroll location only):

Lost Time Accident involves at least one full work day away from work following the day of the injury.

Restricted Duty involves at least one day of restricted or limited work activities by an injured employee.

OSHA Recordable Injury requires medical treatment beyond first aid.

INPO:

Lost Time Accident and Restricted Duty Case Rate = $\frac{\text{Lost Time Accidents and Restricted Duty} \times 200,000 \text{ Manhours}}{\text{Productive Manhours Worked}}$

OSHA:

Recordable Injury Rate = $\frac{\text{OSHA Recordable Injuries} \times 200,000 \text{ Manhours}}{\text{Productive Manhours Worked}}$

Statistical Summary

12 Mo - Ending (Nov)

LTA Rate - 1995 0.40
LTA Rate - 1994 0.49

Targets:

Year End 1995 0.40
Year End 1994 0.45

Industry Performance

INPO

Includes restricted duty cases:

July 1994 - June 1995 Median 0.51
1994 Average 0.64
1995 Goal 0.50

Performance Summary

No Lost Time Accidents or Restricted Duty Cases occurred at St. Lucie during November. St. Lucie's 12-Month Average is now below the 1995 Year End target and Industry Median.

* **NOTE:** OSHA Rate: The St. Lucie rate depicted excludes Juno Staff hours and incidents. The Corporate Safety Department publishes monthly Nuclear Division rates. The rates for this reporting period are:

Month (thru 11/95) 2.87
YTD (thru 11/95) 2.08
Corporate Target: 3.10

Source: Plant Safety Supervisor
Contact: W. Korte, 694-4235

Indicator Type

☐ NRC ☒ INPO ☒ Corp ☒ Div

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