FPL NUCLEAR DIVISION

TURKEY POINT NUCLEAR STATION
ST. LUCIE NUCLEAR STATION

MONTHLY INDICATOR REPORT

February 1996

BBB| 45

Issued: March 19, 1996

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FOREWORD

The Nuclear Division Monthly Indicator Report presents a compilation of performance indicators which provide a quantitative indication of performance. Specific areas of focus include nuclear and personnel safety, plant reliability, and economic performance.

The specific indicators included in this report have been selected by senior management as key indicators of operating performance. Summaries of NRC indicator and WANO indicator performance have been incorporated in this report.

Data contained herein will be refined on the basis of feedback from data providers, of continuing quality control efforts, and of comparisons to other data sources. Each monthly report will reflect the best available data.

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OPERATING PERFORMANCE

Turkey Point Unit 3. Unit 3 achieved an EAF of 65.6%; power losses resulted from: an unplanned automatic trip (53.6 hours); Turbine Valve test (3.8 hours); waterbox and TPCW cleanings (0.6 hours); Amertap screen cleaning (4.3 hours); repair to the Feedwater Check Valve (1.5 hours); repair to 3B Feedwater Heater Pump (41.9 hours); grass intrusion (83.5 hours); Rod Control card reading failure which resulted in a manual insertion of rods for shutdown (32.3 hours), and Chemistry hold (4.7 hours). For the year, EAF was 83.3% which is below the 95.0% Y-T-D target.

Turkey Point Unit 4. Unit 4 achieved a 100.0% EAF through February, exceeding the Y-T-D target of 93.5%.

St. Lucie Unit 1. For the month, Unit 1 EAF was 88.9%. Power losses were the result of: a dropped CEA rod (64.9 hours) and waterbox cleanings (12.2 hours). For the year, EAF was 94.5% which is slightly below the Y-T-D target of 94.8%.

St. Lucie Unit 2. Unit 2 essentially operated at full power achieving an EAF of 99.8% for the month. Power losses of 1.4 hours resulted from repair to MSR Block Valves and Turbine Valve testing. Year-to-date, EAF was 87.9% which is below the target of 95.0%.

Y-T-D **Equivalent Availability** for the Nuclear Division was 91.4% which is below the 94.6% targeted through the period.

An **Unplanned Automatic Trip** occurred at Turkey Point Unit 3; on February 9th, the "B" Steam Generator Feed Pump was stopped to monitor its discharge check valve closing stroke which did not strike closed as expected. The resulting feed flow transient caused the "C" Steam Generator level to increase resulting in a turbine trip which tripped the reactor.

A summary of key plant operating statistics is summarized below.

	PTN Unit 3		PTN Unit 4	
	February	Y-T-D	February	Y-T-D
Gross Generation (MWh)	323,526	862,310	504,313	1,046,564
Net Generation (MWh)	306,886	821,685	481,644	999,512
Net Heat Rate (Btu/KWh)	11104.4	10921.9	10809.2	10786.7
Equivalent Availability	65.6%	83.3%	100.0%	100.0%
Capacity Factor	66.2%	85.7%	103.9%	104.2%
Auto Trips	1	1	0	0
Forced Outage Rate	24.3%	11.8%	0.0%	0.0%

	PSL Unit 1		PSL Unit 2	
	February	Y-T-D	February	Y-T-D
Gross Generation (MWh)	541,339	1,191,589	620,620	1,115,020
Net Generation (MWh)	510,704	1,127,346	588,788	1,051,424
Net Heat Rate (Btu/KWh)	11066.2	11063.4	10815.0	10893.6
Equivalent Availability	88.9%	94.5%	99.8%	87.9%
Capacity Factor	87.5%	93.3%	100.8%	87.0%
Auto Trips	0	0	0	0
Forced Outage Rate	9.1%	4.4%	0.0%	3.4%

MANAGEMENT SUMMARY

REGULATORY PERFORMANCE

No **NRC Violations** were received in February. However, St. Lucie reported four (4) potential violations as follows:

#96-01-01 - Temporary Procedure changes were made which involved changes of intent, without prior FRG review as required by Technical Specifications. Exit Meeting Date: 2/20/96.
#96-03-01 - Operators failed to follow procedures for boron dilution. Exit Meeting Date: 2/8/96.
#96-03-02 - Inadequate design control of RCS boron dilution procedure. Exit Meeting Date: 2/8/96.
#96-03-03 - Inadequate 50.59 safety evaluation of change to boron dilution procedure. Exit Meeting Date: 2/8/96.

COST PERFORMANCE

O&M expeditures for the month of February were $31.2 million which represented a budget underrun of $5.5 million (or 14.9%). This variance was primarily due to: St. Lucie non-outage project underruns, and underruns in materials and contracted services at Turkey Point due to outage cash flow revisions.

February O&M budget performance variances are stratified as follows:

Turkey Point Site Specific	$3.0 million (or 17.4%) below budget
St. Lucie Site Specific	$1.5 million (or 10.0%) below budget
Other Nuclear Division	$1.0 million (or 21.9%) below budget

Capital expenditures for February were $2.5 million. This represented a budget underrun of $4.6 million (or 65.0%). The favorable variance was primarily due to: an underrun in the Steam Generator Replacement Project (SGRP) due to a change in the outage start date, and during year cash flow revisions for various St. Lucie plant projects.

For the month, Capital budget performance variances are detailed as follows:

Turkey Point Site Specific	$0.3 million (or 88.2%) below budget
St. Lucie Site Specific	$4.0 million (or 60.2%) below budget
Other Nuclear Division	$0.3 million (or 307.8%) below budget

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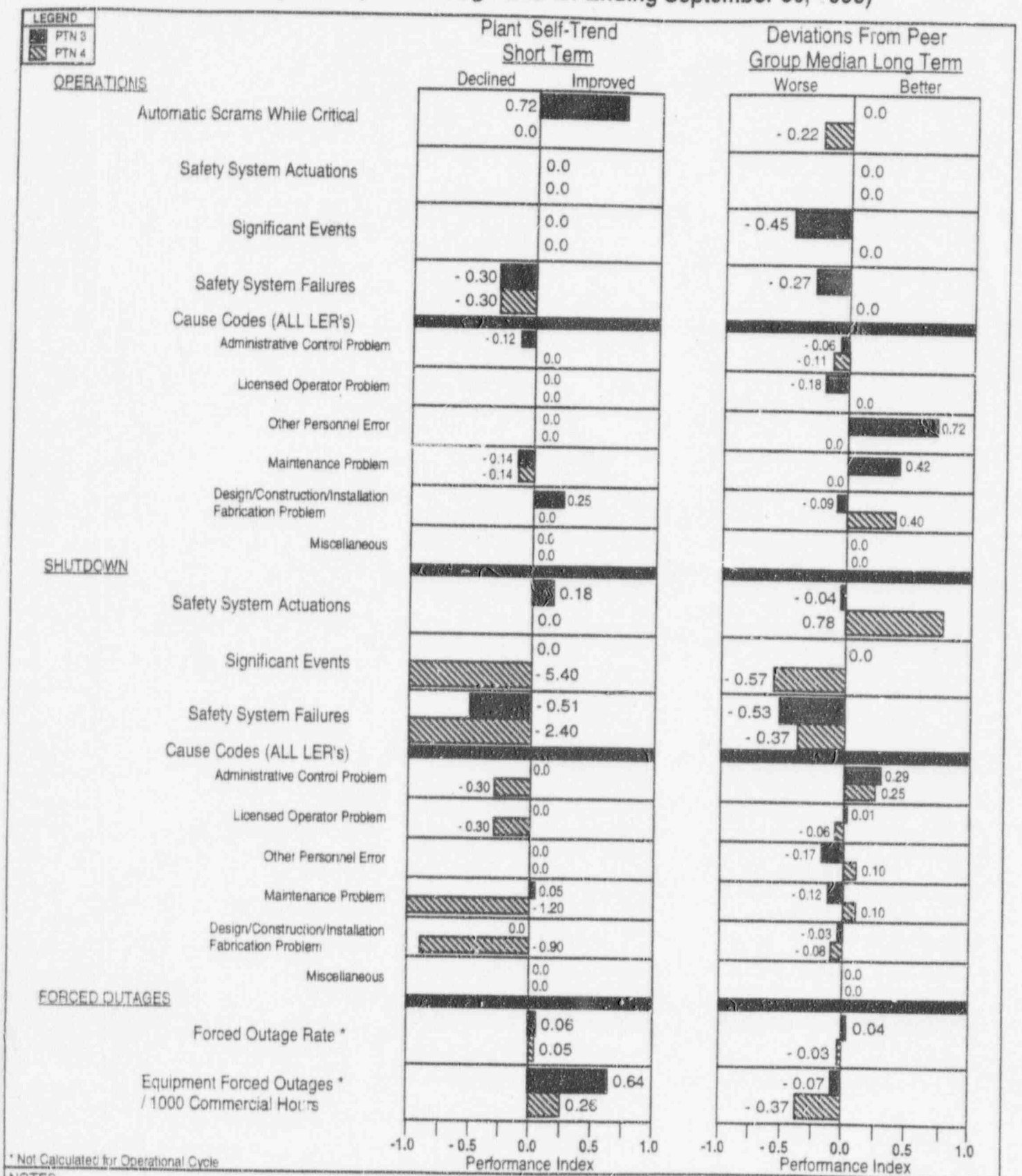
NUCLEAR DIVISION BUSINESS PLAN INDICATOR OVERVIEW

(DATA THROUGH FEBRUARY 29, 1996)

Indicator	Plant	1996 Y-E Target	1996 Y-T-D Target	1996 Y-T-D Actuals	Comments
Collective Radiation Exposure (Man-Rem)	PTN PSL	≤ 275.0 ≤ 485.0	≤10.0 ≤25.0	9.5 9.0	Turkey Point and St. Lucie were below Y-T-D Man-Rem targets.
NRC SALP Ratings	PTN PSL	1.00 1.50		1.00 1.50	The SALP for St. Lucie ending 1/6/96 was 1.50; for PTN, the period ends Aug. 31. 1996.
NRC Violations	PTN PSL	≤ 7 ≤ 7		0 0	No NRC Violations were reported in February. PSL has four potential violations for the month.
Unplanned Automatic Trips	PTN3 PTN4 PSL1 PSL2	≤ 3		1 0 0 0	Turkey Point Unit 3 experienced one automatic trip on February 9th.
Budget Performance ($ Millions)	Capital O&M Div. Total	39.2 253.0 292.2	7.1 36.7 43.8	2.5 31.2 33.7	For the year, O&M and Capital actuals were below Year-End targets; underruns were mainly due to schedule revisions and project underruns.
Equivalent Availability Factor (%)	PTN3 PTN4 PSL1 PSL2	95.0 82.0 78.5 95.0	95.0 93.5 94.8 95.0	83.3 100.0 94.5 87.9	PTN4 exceeded the Y-T-D EAF target in February. Division EAF Y-T-D was 91.4%, which is below the 94.6% Y-T-D target.
M&S Inventory Levels ($ Millions)	PTN PSL	≤ 38 ≤ 38		37.2 42.3	PSL Inventory was higher than the Y-E target due to return of $2.5M of inventory.
Cost (¢) Per KWh — Production Cost (O&M and Fuel)	PTN PSL	1.75 1.55	1.59 1.34	1.36 1.23	Production Costs (¢/KWh) were below the Y-T-D target in February.
Cost (¢) Per KWh — Total Cost (O&M, Fuel, and Capital Carrying Costs)	Div. Total	4.56		4.09	In February, Division Total Cost (¢/KWh) was below the Y-E target .
Nuclear Division Staffing Levels	FPL LT Contr Total	2045.0 463.0 2508.0		1971.0 460.0 2431.0	Total Nuclear Division Staffing Level year-to-date was below the Y-E target.
Refueling Outage Duration (Days)	PTN3 PTN4 PSL1 PSL2	NA 45 63 NA		0 0 0 5	PSL 2 returned to serviced on January 5th. Future Refueling Outage schedules are as follows: PTN3 03/08/97 PSL1 04/29/96 PTN4 03/04/96 PSL2 04/15/97
Lost Time Injuries per 200,000 Hours Worked (12 Month Running)	PTN PSL	0.30 0.30		0.22 0.30	No Lost Time Injuries or Restricted Duty Cases were reported in February.

NRC INDICATOR PERFORMANCE OVERVIEW

for Turkey Point (Data through Quarter Ending September 30, 1995)



* Not Calculated for Operational Cycle

NOTES:

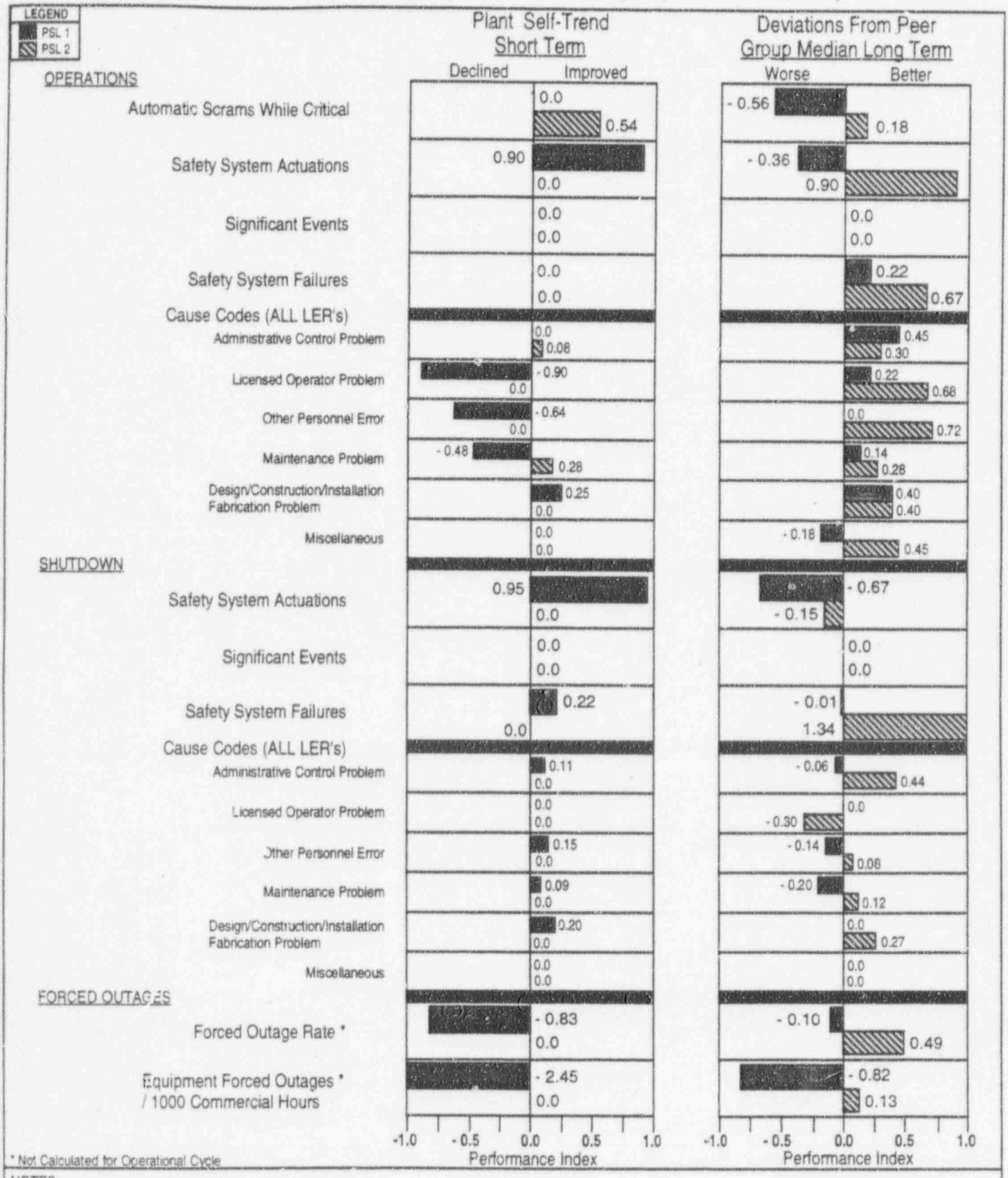
Plant Self-Trend Short Term: Based on the slope of a linear regression line plotted over each plant's data. Time intervals used in the trends are 4 Quarters for "Operations" and Forced Outages" indicators and 6 quarters for Shutdown" indicators.

Deviations from Peer Group Median Long Term: Comparisons are made of each plant to the performance of its peers over a 12 Quarter time interval.

Peer Groups: PTN 3&4 - Older Westinghouse 3-Loop.

NRC INDICATOR PERFORMANCE OVERVIEW

for St. Lucie (Data through Quarter Ending September 30, 1995)



* Not Calculated for Operational Cycle

NOTES:

Plant Self-Trend Short Term: Based on the slope of a linear regression line plotted over each plant's data. Time intervals used in the trends are 4 Quarters for "Operations" and Forced Outages" indicators and 6 quarters for Shutdown" indicators.

Deviations from Peer Group Median Long Term: Comparisons are made of each plant to the performance of its peers over a 12 Quarter time interval.

Peer Group: PSL 1&2 - Combustion Engineering with core protection calculator plants.

WANO OVERALL INDICATOR PERFORMANCE OVERVIEW

(February 29, 1996)

Performance Indicators		Unit or Station Values				Industry Median Values*	
		PTN 3	PTN 4	PSL 1	PSL 2	3-Year Distribution (7/92 - 6/95)	2000 Goals
Unit Capability Factor (Unit %, 3-Yr. Distribution Ending February '96)		88.6%	87.3%	77.8%	77.7%	79.7%	87.0%
Unplanned Capability Loss Factor (Unit %, 3-Year Distr. Ending February '96)		2.9%	4.3%	11.0%	9.4%	6.0%	3.0%
Unplanned Automatic Scrams Per 7000 Hours Critical (Per Unit, 3-Year Distribution Ending February '96)		0.6	0.9	1.6	0.7	1.0	1.0
Safety System Performance:	High Pressure Safety Injection System (Per Unit, 3-Year Distribution Ending December '95)	0.008	0.009	0.025	0.010	0.003 - 0.008	not available
	Auxiliary Feedwater System (Per Unit, 3-Year Distribution Ending December '95)	0.014	0.011	0.010	0.014	0.004 - 0.009	not available
	Emergency AC Power System (Per Unit, 3-Year Distribution Ending December '95)	0.003	0.003	0.019	0.012	0.009 - 0.014	not available
Thermal Performance (Ratio of Design to Actual Gross Heat Rate, 1-Yr. Distribution Ending February '96)		99.9%	100.1%	98.9%	98.2%	99.4% **	99.5%
Collective Radiation Exposure (Man-Rem per unit per year, 3-year running avg. ending February '96)		161	161	227	227	157	120
Volume of Low-level Solid Radioactive Waste (Cubic meters per unit per year, 3-year avg. ending February '96)		36.2	36.2	53.0	53.0	62.0 #	45.0
Chemistry Index (12-Mo. average through February '96)		1.19	1.04	1.06	1.19	1.19 **	1.10
Industrial Safety Lost-time Accident Rate (Station rate per 200,000 man-hours worked ending February '96)		0.22		0.30		0.51 **	0.40
Fuel Reliability (Unit microcuries/g, month ending February '96)		1.00E-6	1.51-5	1.69E-5	1.00E-6	Fuel Defect Reference 5.0E-04***	

NOTE: Shaded area denotes FPL performance is unfavorable to actual industry median.

Source of Industry Data:

* 1995 Mid-Year Report for Performance Indicators for the U.S. Nuclear Utility Industry (7/92-6/95 Distribution).
** 1995 Mid-Year Report for Performance Indicators for the U.S. Nuclear Utility Industry (7/94-6/95 Distribution).
*** 1995 Mid-Year Report for Performance Indicators for the U.S. Nuclear Utility Industry (4/95-6/95 Distribution).
\# 1995 Mid-Year Report for Performance Indicators for the U.S. Nuclear Utility Industry (1/92-12/94 Distribution).

Rev. 3/20/96

WANO OVERALL INDICATOR PERFORMANCE OVERVIEW
Discussion of FPL Performance Unfavorable to Industry Median
(February 29, 1996)

UNIT CAPABILITY FACTOR
(3-Years Ending 2/29/96)

St. Lucie Unit 1. The 3-year running Capability Factor for the unit was 77.8%. Capability loss is attributed to the following: Refueling Outage and extension from 3/29/93 to 6/17/93 (7.4%); Hot Leg Valve MV-3480 leak repairs from 3/29/94 to 4/2/94 (0.4%); Main Transformer trip from 6/6/94 to 6/11/94 (0.4%); Quench Tank leak repairs from 2/27/95 to 3/8/95 (0.9%); 1A2 Reactor Coolant Pump seal repairs from 8/2/95 to 8/9/95 (0.7%); inoperable Power Operated Relief Valves from 8/9/95 to 8/17/95 (0.7%); and, inadvertent Containment Spray actuation and clean-up from 8/17/95 to 9/3/95 (1.3%) 1B2 Diesel Generator failure from 9/1/95 to 9/6/95 (0.5%); 1A Diesel Generator Radiator leakage from 9/6/95 to 9/10/95 (0.4%); and, Code Safety Valve repairs and modifications from 9/11/95 to 10/13/95 (2.4%). Other miscellaneous unplanned outages and derates accounted for the remaining 7.1% Unit Capability Factor loss.

St. Lucie Unit 2. Capability Factor for the three years ending 2/29/96 was 77.7%. Capability loss is attributed to the following: 2A1 Reactor Coolant Pump repairs due to high vibration from 1/13/93 to 3/2/93 (0.2%); Pressurizer Nozzle leak repairs initiated on 3/2/93 through 4/1/93 (2.7%); dropped CEA's on 5/21/93 (0.5%); Condenser Tube leak repairs from 8/9/93 to 8/11/93 (0.2%); refueling outage from 2/13/94 to 4/22/94 (4.6%); shutdown for auto reactor trip investigation on 4/23/94 (0.3%); and, a refueling outage from 10/9/95 to 1/5/96 (7.9%). Other unplanned outages and power reductions accounted for the remaining 5.9% Unit Capability Factor loss.

UNPLANNED CAPABILITY LOSS FACTOR
(3-Years Ending 2/29/96)

St. Lucie Unit 1. The Unplanned Capability Loss Factor for the three years ending 2/29/96 was 11.0% compared to an industry median of 6.0%. Unplanned outages and power reductions contributing to this performance included: Refueling Outage extension from 6/1/93 to 6/17/93 (1.6%); Waterbox cleaning due to jelly fish intrusion from 9/18/93 to 9/29/93 (0.7%); Hot Leg Valve MV-3480 leak repairs from 3/29/94 to 4/2/94 (0.4%); Main Transformer trip from 6/6/94 to 6/11/94 (0.4%); Quench Tank leak repairs from 2/27/95 to 3/8/95 (0.9%); 1A2 Reactor Coolant Pump seal repairs from 8/2/95 to 8/9/95 (0.7%); inoperable Power Operated Relief Valves from 8/9/95 to 8/17/95 (0.7%); inadvertent Containment Spray actuation and clean-up from 8/17/95 to 9/1/95 (1.3%) 1B2 Diesel Generator failure from 9/1/95 to 9/6/95 (0.5%); 1A Diesel Generator Radiator leakage from 9/6/95 to 9/10/95 (0.4%); and, Code Safety Valve repairs and modifications from 9/11/95 to 10/31/95 (2.4%). Other miscellaneous unplanned outages and derates accounted for the remaining 1.0% Capability Loss.

St. Lucie Unit 2. The Unplanned Capability Loss Factor for the three years ending 1/31/96 was 9.4%. Major unplanned occurrences contributing to this performance included: 2A1 Reactor Coolant Pump repairs due to high vibration from 1/13/93 to 3/2/93 (0.2%); Pressurizer Nozzle leak repairs initiated on 3/2/93 through 4/1/93 (2.7%); dropped CEA's on 5/21/93 (0.5%); shutdown for auto reactor trip investigation on 4/23/94 (0.3%); and, refueling outage extension from 12/1/95 to 1/5/96 (3.2%). Other unplanned outages and power reductions accounted for the remaining 2.5% in Capability Loss.

UNPLANNED AUTOMATIC SCRAMS PER 7000 HOURS CRITICAL
(3-Years Ending 2/29/96)

St. Lucie Unit 1. Increased rate for Unit 1 was the result of five auto trips occurring on 3/18/94, 4/3/94, 6/6/94, 10/26/94 and 7/8/95.

HIGH PRESSURE SAFETY INJECTION SYSTEM
(3-Years Ending 12/31/95)

Turkey Point Unit 4. Average performance was affected due to last year's on-line replacement of the HPSI pump motors following discovery of cracked rotor bars.

St. Lucie Unit 1. Average performance for the last three years was affected by on line Motor Operated Valve testing in the 3rd Quarter of 1994 and a breaker failure on 2B HPSI pump in the 1st Quarter of 1995.

St. Lucie Unit 2. In the 1st Quarter of 1995, average performance was affected as a result of Component Cooling Water (CCW) Heat Exchanger cleaning which placed the respective HPSI pump OOS due to lack of dedicated seal cooling.

AUXILIARY FEEDWATER SYSTEM
(3-Years Ending 12/31/95)

Turkey Point. Average performance for both units was affected by the B AFW Turbine failure in the 4th Quarter of 1994 due to malfunction of the mechanical overspeed trip device; in the 3rd Quarter of 1995, performance was affected by: Part 21 repairs on the Trip and Trottle Valves, and Unit 3 outage work.

St. Lucie. Average performance for three-years was affected by failure in the 4th Quarter of 1994 of the 1C AFW Pump Governor. In the 3rd Quarter of 1995, performance was attributed to: failure of the AFW PP 2C Steam Admission Valve MV-08-13 to open, a mechanical trip linkage for AFW PP 2C when the Electrical Overspeed Solenoid was energized, and a discrepancy between field wiring and plant wiring drawing for the AFW PP 2B.

EMERGENCY DIESEL GENERATOR SYSTEM
(3-Years Ending 12/31/95)

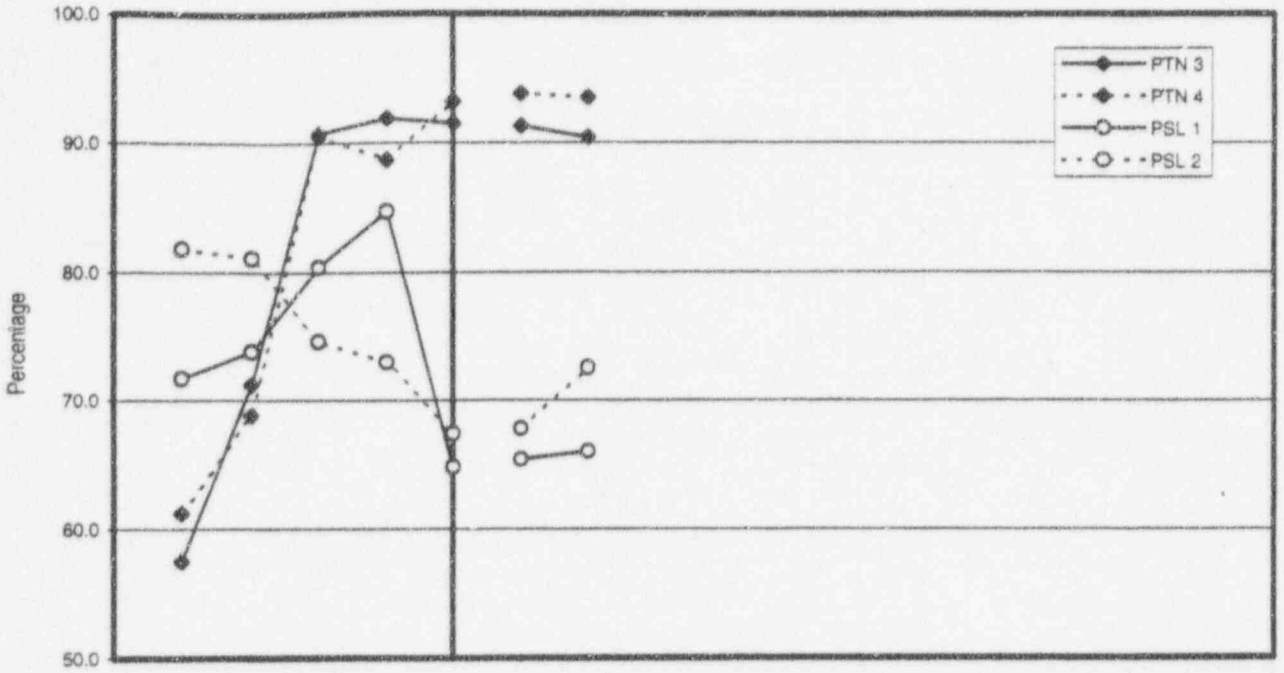
St. Lucie Unit 1. Unit 1's average performance for three-years was the result of a high water jacket temperature trip of 1A EDG and failure of the governor on 1B2 during monthly surveillance run which closed off fuel to the 12 cylinder engine in the 2nd Quarter of 1995, 1B diesel 12 cylinder engine valve failure in the 3rd Quarter of 1995, and 1B diesel due to replacement of the cooling water valves in the 4th Quarter 1995.

COLLECTIVE RADIATION EXPOSURE - MAN-REM
(3-Years Ending 2/29/96)

Turkey Point. The three-year running average Collective Radiation Exposure level for Turkey Point was 161 Man-Rem per unit which was greater than the industry median of 157 Man-Rem. Site performance was influenced by scheduled refueling outages.

St. Lucie. Collective Radiation Exposure three-year running average level for St. Lucie was 227 Man-Rem per unit which was greater than the industry median of 157 Man-Rem. Site performance was influenced by unplanned and scheduled outages.

WANO WEIGHTED OVERALL PERFORMANCE



Unit	1991	1992	1993	1994	1995	1/96	2/96	3/96	4/96	5/96	6/96	7/96	8/96	9/96	10/96	11/96	12/96
PTN 3	57.5	71.2	90.7	91.9	91.5	91.3	90.4										
PTN 4	61.2	68.8	90.5	86.7	93.2	93.8	93.5										
PSL 1	71.7	73.8	80.4	84.7	64.8	65.4	66.0										
PSL 2	81.8	81.1	74.6	73.0	67.4	67.8	72.5										

DEFINITION

The WANO Overall Performance Index is a composite indicator utilized to trend nuclear station performance.
The index is a weighted combination of the following 10 individual performance indicators:

1. Unit Capability Factor (16%)
2. Unplanned Capability Loss Factor (12%)
3. High Pressure Safety Injection (9%)
4. Auxiliary Feedwater System (9%)
5. Emergency AC Power (9%)
6. Unplanned Auto-Scrams (8%)
7. Collective Radiation Exposure (8%)
8. PWR Fuel Reliability (7%)
9. Thermal Performance (6%)
10. Chemistry Indicator (6%)
11. Low-level Radwaste Volume (5%)
12. Industrial Safety Accidents (5%)

STATISTICAL SUMMARY

INDUSTRY PERFORMANCE

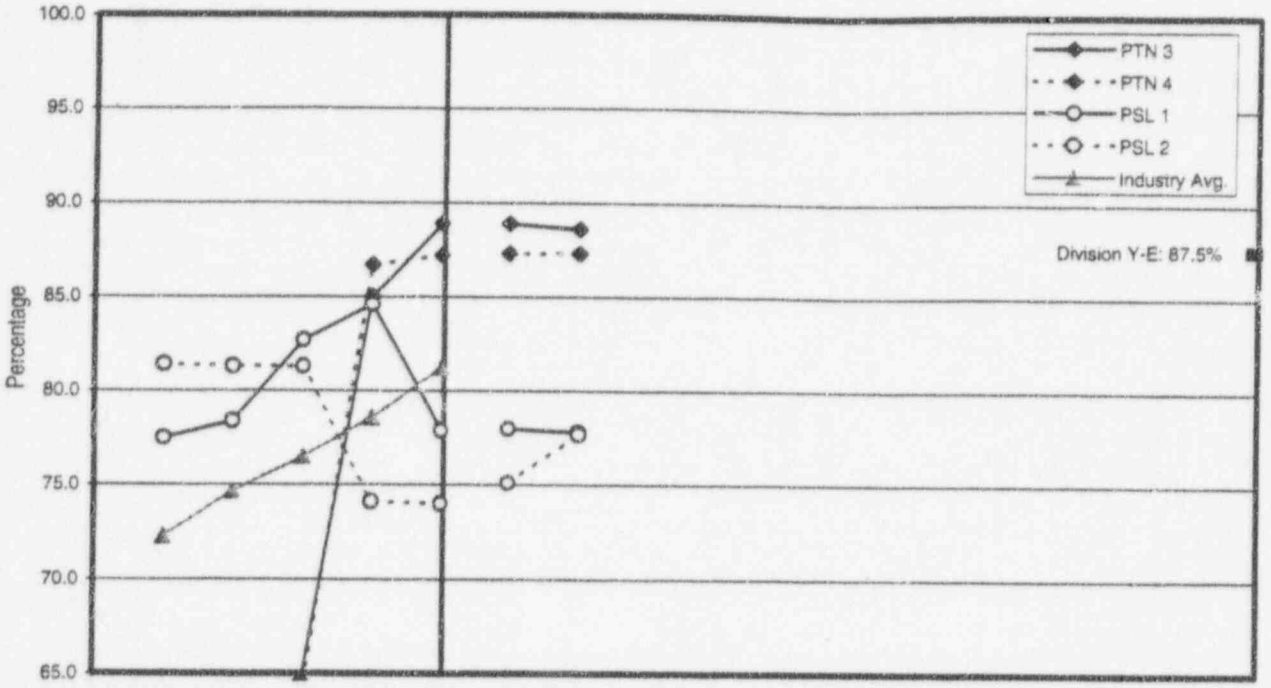
WANO:

Through 2nd Quarter 1995 - Industry Median: 81.0%

PERFORMANCE SUMMARY

UNIT CAPABILITY FACTOR

(3-Year Running Average)



Unit	1991	1992	1993	1994	1995	1/96	2/96	3/96	4/96	5/96	6/96	7/96	8/96	9/96	10/96	11/96	12/96
PTN 3	47.9	52.7	64.6	85.0	88.9	86.9	88.6										
PTN 4	42.2	61.8	63.7	86.7	87.2	87.3	87.3										
PSL 1	77.5	78.4	82.7	84.6	77.9	78.0	77.8										
PSL 2	81.4	81.3	81.3	74.1	74.0	75.1	77.7										
Industry Avg.	72.3	74.6	76.5	78.6	81.2												

DEFINITION

Unit Capability Factor is the ratio of the available energy generation over a given time period to the reference energy generation over the same time period, expressed as a percentage with both energy generation terms determined relative to reference ambient conditions. Available energy generation is the energy that could have been produced under reference ambient conditions considering only limitations within control of plant management, i.e., plant equipment and personnel performance, and work control. Reference energy generation is the energy that could be produced if the unit were operated continuously at full power under reference ambient conditions throughout the period. Reference ambient conditions are environmental conditions representative of the annual mean (or typical) ambient conditions for the unit.

STATISTICAL SUMMARY

	Feb	Y-T-D	3-yr Running	Y-E Target
PTN 3	65.6%	83.3%	88.6%	≥ 95.0%
PTN 4	100.0%	100.0%	87.3%	≥ 82.0%
PSL 1	88.9%	94.5%	77.8%	≥ 78.0%
PSL 2	99.8%	87.9%	77.7%	≥ 95.0%

INDUSTRY PERFORMANCE

WANO

July 1992 - June 1995 Median	79.7%
12-mo. thru July 1995 Median	84.5%
1995 Goal	80.0%

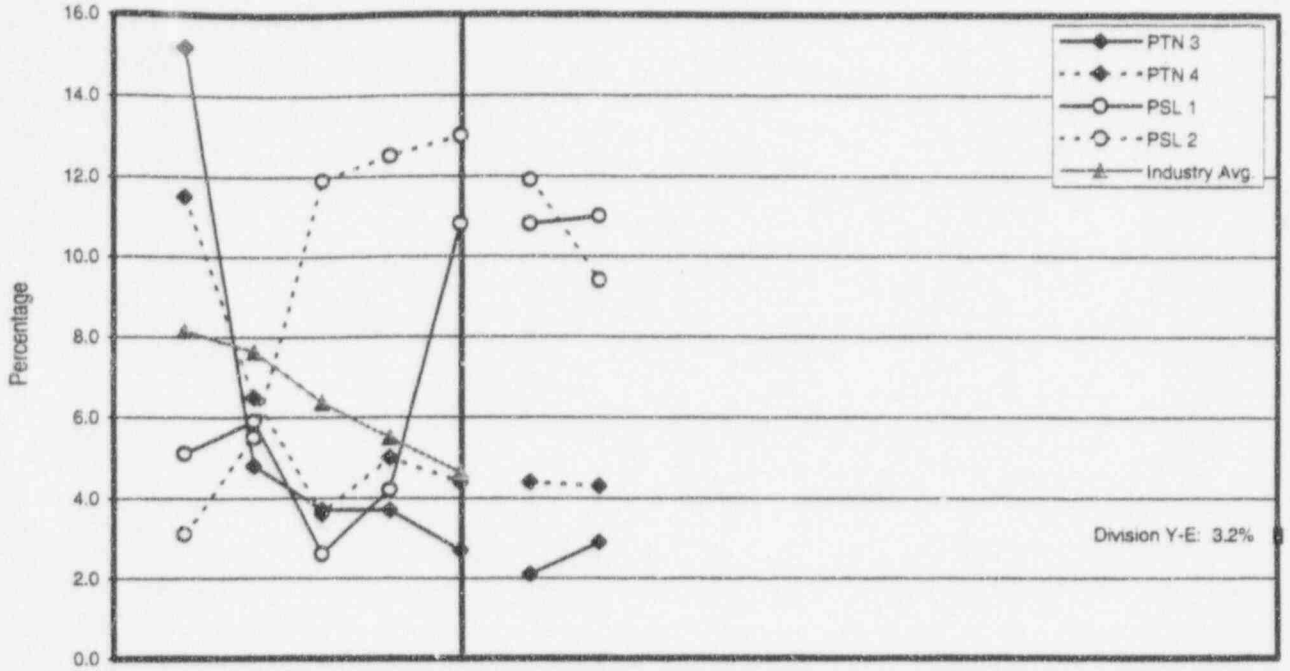
PERFORMANCE SUMMARY

St. Lucie Unit 1and 2's Unit Capability Factor (3-yr. running average) performance through February 1996 was below WANO's 3-yr. running industry median.

Data Source: GADS Report

UNPLANNED CAPABILITY LOSS FACTOR

(3-year Running Average)



Unit	1991	1992	1993	1994	1995	1/96	2/96	3/96	4/96	5/96	6/96	7/96	8/96	9/96	10/96	11/96	12/96
PTN 3	15.2	4.8	3.7	3.7	2.7	2.1	2.9										
PTN 4	11.5	6.5	3.6	5.0	4.4	4.4	4.3										
PSL 1	5.1	5.9	2.6	4.2	10.8	10.8	11.0										
PSL 2	3.1	5.5	11.9	12.5	13.0	11.9	9.4										
Industry Avg.	8.2	7.6	6.4	5.5	4.6												

DEFINITION

Unplanned Capability Loss Factor is defined as the ratio of the unplanned energy losses during a given period of time to the reference energy generation, expressed as a percentage. Unplanned energy loss is energy that was not produced during the period because of unplanned shutdowns, outage extensions, or unplanned load reductions due to causes under plant management control. Causes of energy losses are considered to be unplanned if they are not scheduled at least four weeks in advance. Reference energy generation is the energy that could be produced if the unit were operated continuously at full power under reference ambient conditions throughout the period. Reference ambient conditions are environmental conditions representative of the annual mean (or typical) ambient conditions for the unit.

STATISTICAL SUMMARY

	3-yr Running Avg
PTN 3	2.9%
PTN 4	4.3%
PSL 1	11.0%
PSL 2	9.4%

1996 Target (all units): ≤ 3.2%

INDUSTRY PERFORMANCE

WANO

July 1992 - June 1995 Median	6.0%
12-Mo. ending June 1995 Median	4.2%
1995 Goal	4.5%

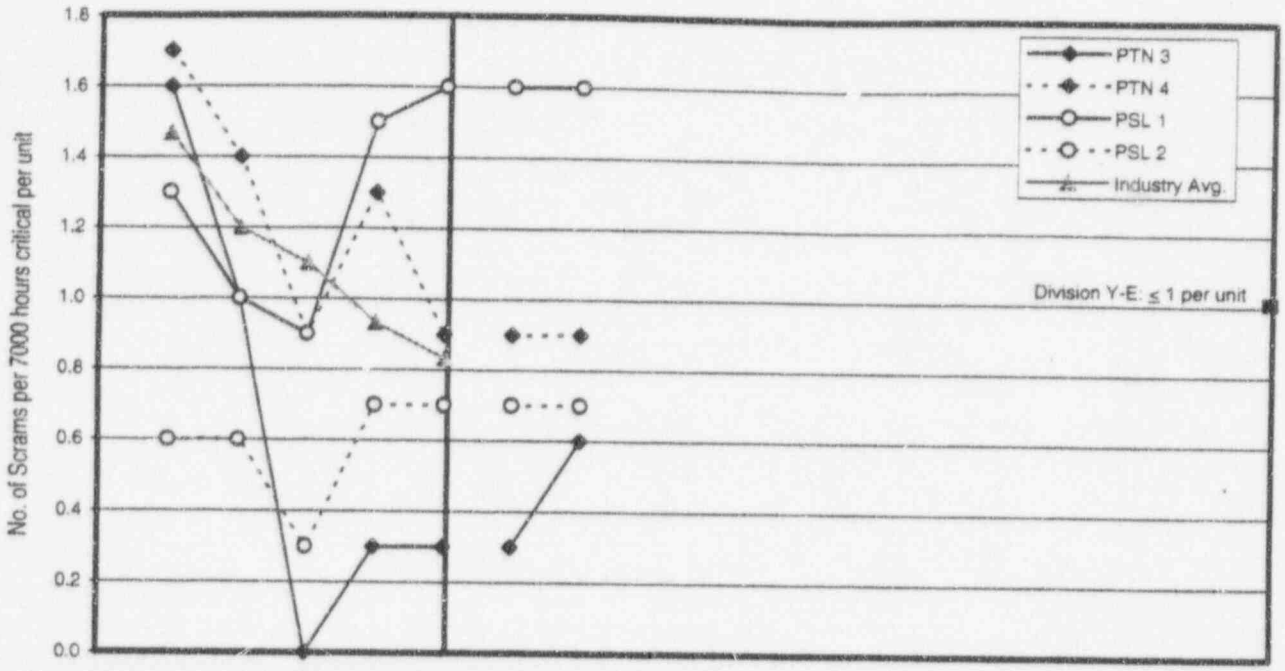
PERFORMANCE SUMMARY

St. Lucie Units 1 and 2's Unplanned Capability Loss Factor performance for 3-yr running through February 1996 was higher than the WANO 3-year running industry median.

Data Source: GADS Report

UNPLANNED AUTO TRIPS PER 7000 HOURS CRITICAL

(3-year Running Average)



Unit	1991	1992	1993	1994	1995	1/96	2/96	3/96	4/96	5/96	6/96	7/96	8/96	9/96	10/96	11/96	12/96
PTN 3	1.6	1.0	0.0	0.3	0.3	0.3	0.6										
PTN 4	1.7	1.4	0.9	1.3	0.9	0.9	0.9										
PSL 1	1.3	1.0	0.9	1.5	1.6	1.6	1.6										
PSL 2	0.6	0.6	0.3	0.7	0.7	0.7	0.7										
Industry Avg.	1.5	1.2	1.1	0.9	0.8												

DEFINITION

Unplanned Automatic Scrams per 7000 Hours Critical is defined as the number of unplanned automatic scrams that occur per 7000 hours of critical operation. Unplanned means that the scram was not an anticipated part of a planned test. Scram means the automatic shutdown of the reactor by a rapid insertion of negative reactivity (by control rods, liquid injection shutdown system, etc.) caused by actuation of the reactor protection system. The scram signal may have resulted from exceeding a setpoint or may have been spurious. Automatic means that the initial signal that caused actuation of the reactor protection system logic was provided from one of the sensors monitoring plant parameters and conditions rather than from the manual scram switches or, in certain cases described in INPO 94-009, from manual turbine trip switches provided in the main control room. Critical means that during the steady-state condition prior to the scram, the effective reactor multiplication factor was essentially equal to one.

STATISTICAL SUMMARY

Through February

	12-mo.	36-mo.
Division Totals:		
1996	0.5	0.9
1995	2.1	1.0

Division Target: ≤ 3.0 auto trips

INDUSTRY PERFORMANCE

WANO

July 1992 - June 1995 Median	1.0
12-Mo. ending June 1995 Median	0.8
1995 Goal	1.0

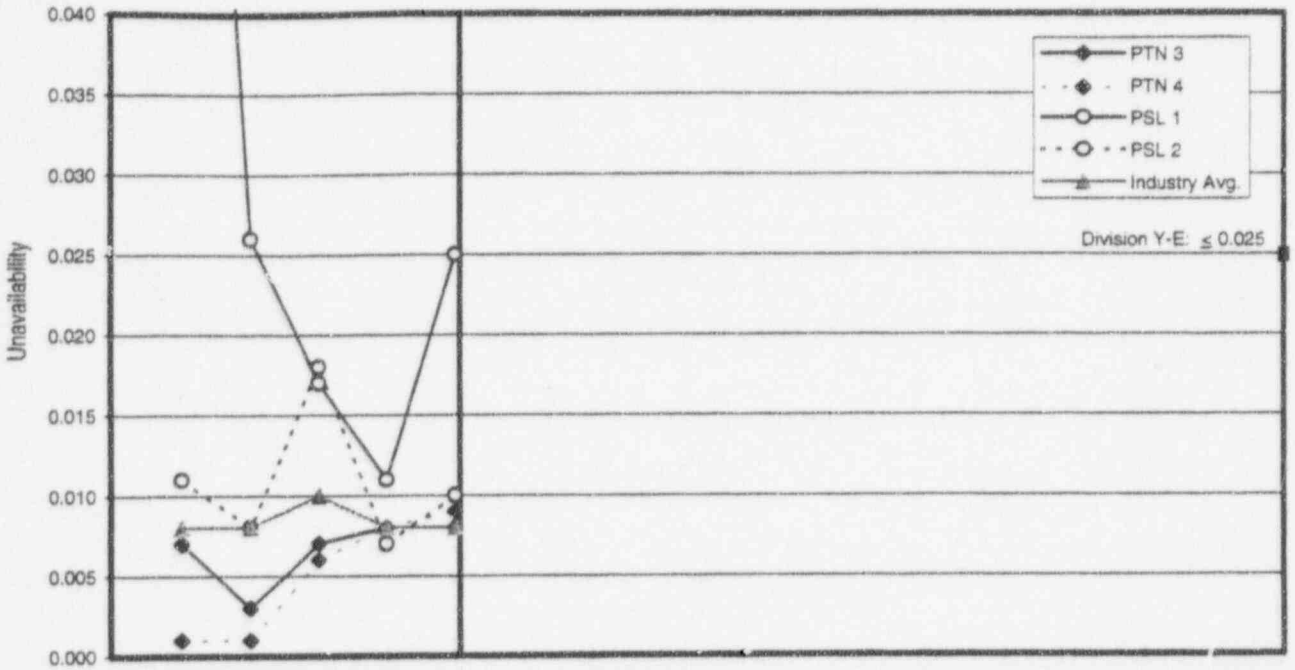
PERFORMANCE SUMMARY

- Turkey Point Unit 3 experienced an automatic trip on February 9th. The "B" Steam Generator Feed Pump was stopped to monitor its discharge check valve closing stroke which did not strike closed as expected. The resulting feed flow transient caused the "C" Steam Generator level to increase resulting in a turbine trip which tripped the reactor. Unit 3's 3-year running average of 0.6 was below the industry median.
- St. Lucie Unit 1's 3-year running average of 1.6 exceeded WANO's 3-year running average of 1.0 as a result of four trips occuring in 1994 and one in 1995.

Data Providers: (PTN) Jim Knorr 246-6757 and (PSL) Mike Snyder 467-7036

HIGH PRESSURE AND SAFETY INJECTION SYSTEM PERFORMANCE

(3-yr Running Average)



Unit	1991	1992	1993	1994	1995			3/96			6/96			9/96			12/96
PTN 3	0.007	0.003	0.007	0.008	0.008												
PTN 4	0.001	0.001	0.006	0.008	0.009												
PSL 1	0.091	0.026	0.017	0.011	0.025												
PSL 2	0.011	0.008	0.018	0.007	0.010												
Industry Avg.	0.008	0.008	0.010	0.008	0.008												

DEFINITION

This Safety System Performance indicator monitors the readiness of the Safety Injection (SI) System at Turkey Point and the High Pressure Safety Injection (HPSI) System at St. Lucie to respond to off-normal events or accidents. The indicator is determined from the unavailabilities, due to all causes, of the components in the system during a time period, divided by the number of trains in the system. The definition is further explained: component unavailability is the ratio of the hours the component was unavailable (unavailable hours) to the hours the system was required to be available for service. Data is reported on a quarterly basis.

$$\text{Unavailability} = \frac{\text{(Known Unavailable Hours) + (Estimated Unavailable Hours)}}{\text{(Hours System Required) x (Number of Trains)}}$$

STATISTICAL SUMMARY

	4th Qtr 1995	Yr 1995	3-yr Avg Ending 1995
PTN 3	0.002	0.003	0.008
PTN 4	0.001	0.003	0.009
PSL 1	0.000	0.023	0.025
PSL 2	0.000	0.008	0.010

Targets:	PTN	PSL
1995 Y-E Target	≤ 0.016	≤ 0.023
1996 Y-E Target	≤ 0.025	≤ 0.025

INDUSTRY PERFORMANCE

WANO	Unavailability
July 1992 - June 1995 Median (PWR)	0.003 - 0.008
1995 Goal	0.020

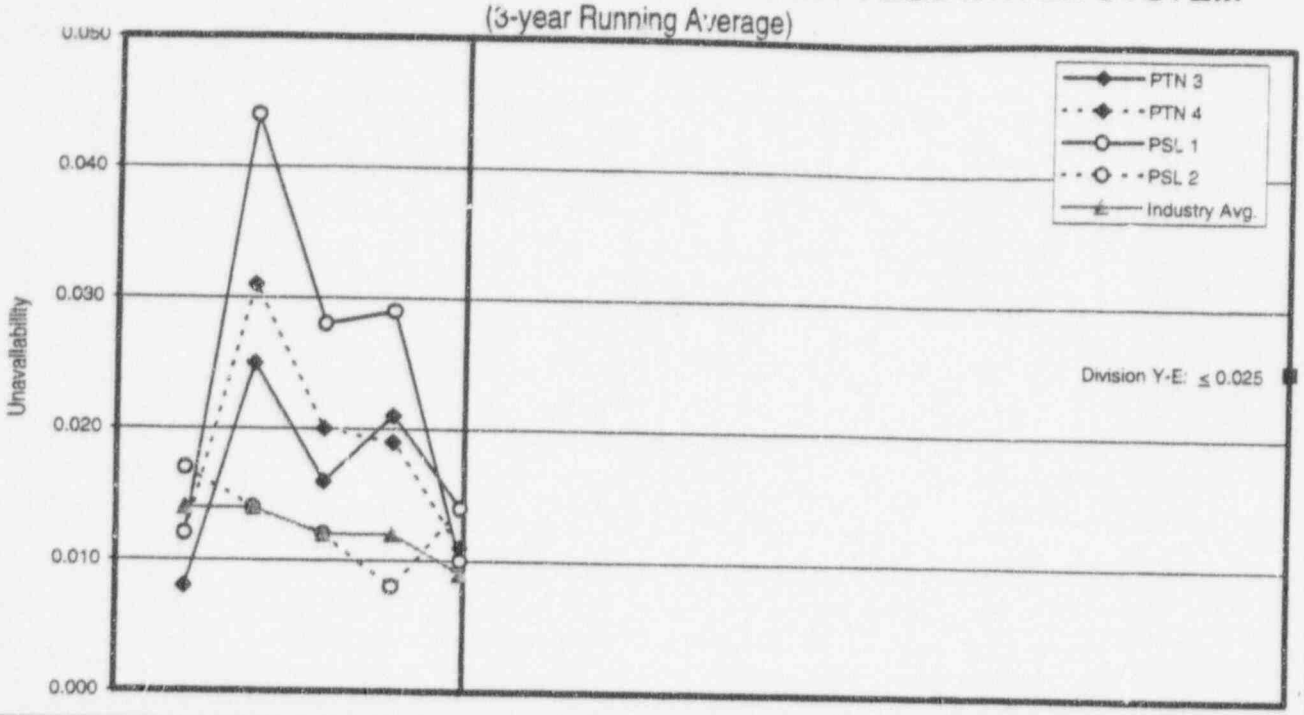
PERFORMANCE SUMMARY

- Turkey Point Unit 4's 3-year running average for Safety Injection System performance was slightly above the industry median.
- St. Lucie Units 1 & 2's High Pressure Injection System performance for 3-years running was higher than the industry median.

Data Providers: (PTN) Ed Lyons 246-6967/Carlos Melchor 246-6964
(PSL) Chuck Wood 467-7034/Catherine Swiatek 467-7081

SAFETY SYSTEM PERFORMANCE - AUXILIARY FEEDWATER SYSTEM

(3-year Running Average)



Unit	1991	1992	1993	1994	1995			3/96			6/96			9/96			12/96
PTN 3	0.008	0.025	0.016	0.021	0.014												
PTN 4	0.012	0.031	0.020	0.019	0.011												
PSL 1	0.012	0.044	0.028	0.029	0.010												
PSL 2	0.017	0.014	0.012	0.008	0.014												
Industry Avg.	0.014	0.014	0.012	0.012	0.009												

DEFINITION

This Safety System Performance indicator monitors the readiness of the Auxiliary Feedwater (AFW) System to respond to off-normal events or accidents. The indicator is determined from the unavailabilities, due to all causes, of the components in the system during a time period, divided by the number of trains in the system. This definition is further explained: component unavailability is the ratio of hours the component was unavailable (unavailable hours) to the hours the system was required to be available for service.

AFW Unavailability = (Known Unavailable Hours) + (Estimated Unavailable Hours)
(Hours System Required) x (Number of Trains)

STATISTICAL SUMMARY

	4th Qtr 1995	Yr 1995	3-yr Avg Ending 1995
PTN 3	0.008	0.009	0.014
PTN 4	0.013	0.011	0.011
PSL 1	0.002	0.003	0.010
PSL 2	0.000	0.029	0.014

Targets:	PTN	PSL
1995 Y-E Target	≤ 0.020	≤ 0.021
1996 Y-E Target	≤ 0.025	≤ 0.025

INDUSTRY PERFORMANCE

WANO	Unavailability
July 1992 - June 1995 Median (PWR)	0.004 - 0.009
1995 Goal	0.025

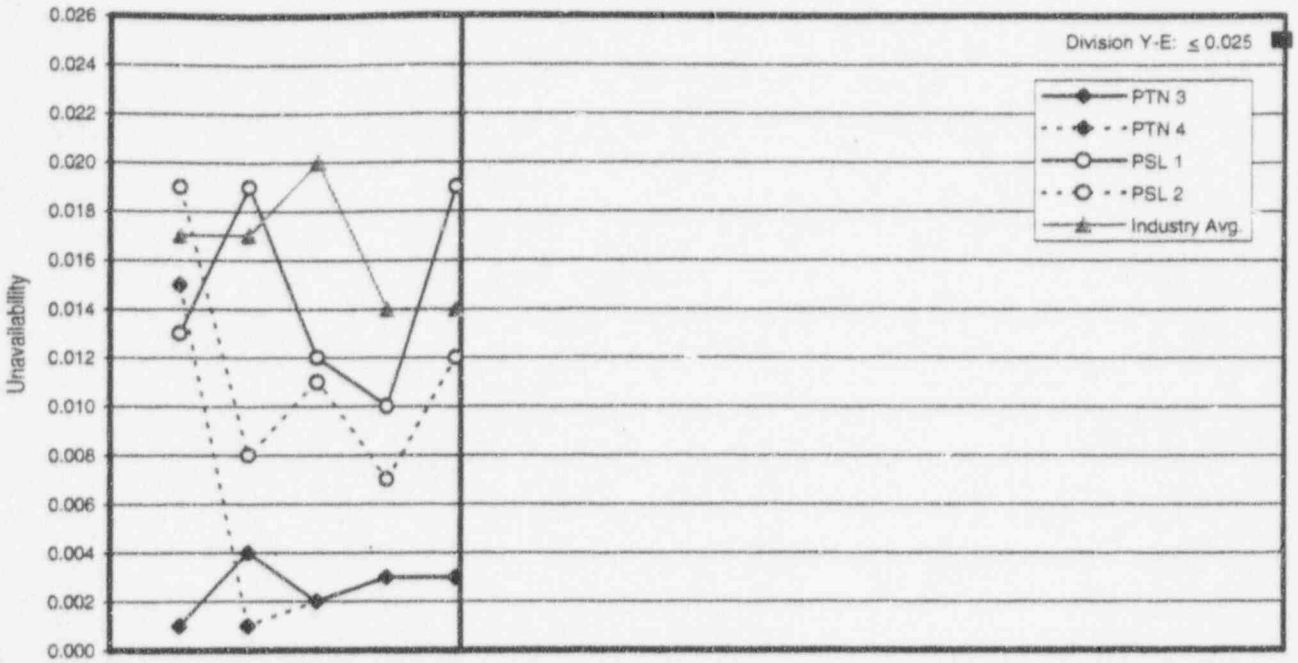
PERFORMANCE SUMMARY

- Turkey Point Units 3 & 4 Auxiliary Feedwater System performance for 3-years running through 1995 was lower than the year-end target and higher than the industry median.
- St. Lucie Units 1 & 2 Auxiliary Feedwater System performance for 3-years running ending 1995 was below the year-end target and higher than the industry median.

Data Providers: (PTN) Jose Donis 246-6008/Woody Raasch 246-6527
(PSL) Chuck Wood 467-7034/Mark Wolaver 467-7083

SAFETY SYSTEM PERFORMANCE - EMERGENCY AC POWER SYSTEM

(3-year Running Average)



Unit	1991	1992	1993	1994	1995			3/96			6/96			9/96			12/96
PTN 3	0.001	0.004	0.002	0.003	0.003												
PTN 4	0.015	0.001	0.002	0.003	0.003												
PSL 1	0.013	0.019	0.012	0.010	0.019												
PSL 2	0.019	0.003	0.011	0.007	0.012												
Industry Avg.	0.017	0.017	0.020	0.014	0.014												

DEFINITION

Emergency AC Power System is defined as the sum of the emergency diesel generator unavailabilities divided by the number of emergency generators at a station. Data is collected at the train level. The emergency generator includes subsystems such as air start, lube oil, fuel oil, cooling water, etc. However, for this safety system performance indicator, unavailable hours are counted only when the emergency generator is unavailable to start or load-run. For example, if a component fails in one train of a redundant support system the emergency generator is still operable, and no unavailable hours are counted.

STATISTICAL SUMMARY

	4th Qtr 1995	Yr 1995	3-yr Avg Ending 1995
PTN 3A&B	0.007	0.004	0.003
PTN 4A&B	0.007	0.003	0.003
PSL 1A&B	0.049	0.046	0.019
PSL 2A&B	0.004	0.002	0.012

Targets:	PTN	PSL
1995 Y-E Target	≤ 0.015	≤ 0.016
1996 Y-E Target	≤ 0.025	≤ 0.025

INDUSTRY PERFORMANCE

WANO	Unavailability
July 1992 - June 1995 Median (PWR)	0.009 - 0.014
1995 Goal	0.025

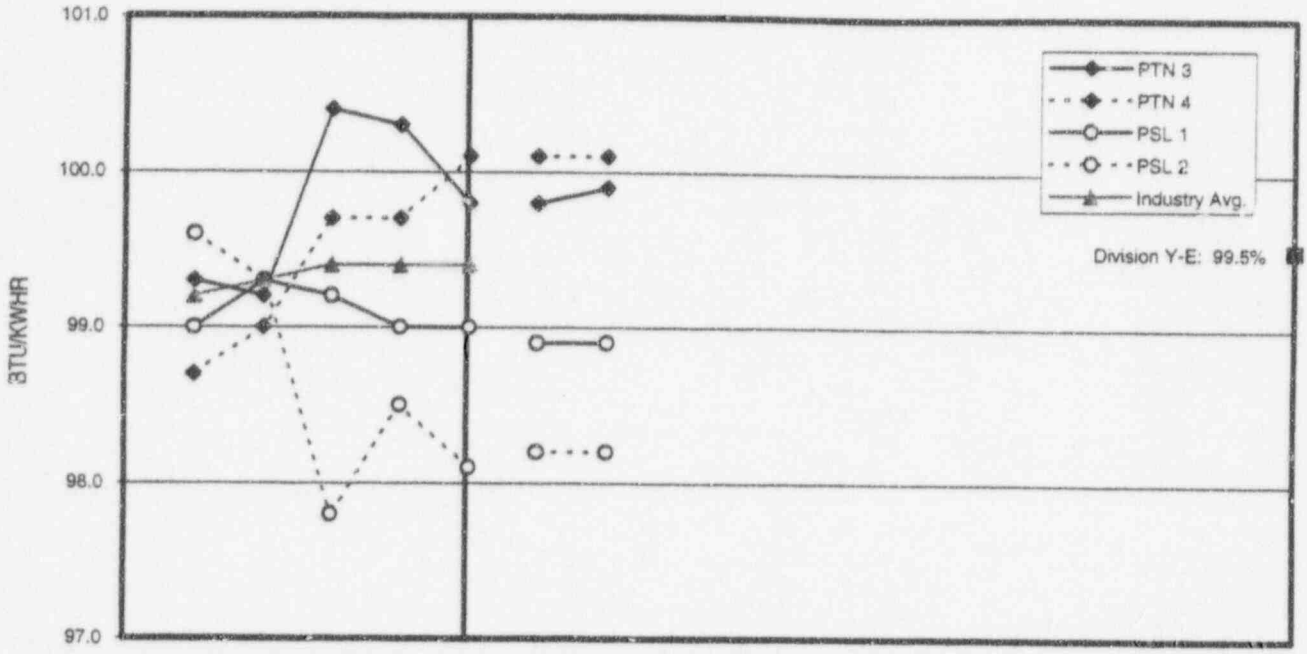
PERFORMANCE SUMMARY

- Turkey Point Units 3 & 4 Emergency AC Power performance for 3-years running ending 1995 was below the year-end target and industry median.
- St. Lucie Unit 1 Emergency AC Power performance was higher than the year-end target and industry median for 3-years running through 1995. Unit 2 performance was below the year-end target and industry median for this period.

Data Providers: (PTN) Dan Tomaszewski 246-6158/Jim Freyre 246-6539
(PSL) Chuck Wood 467-7034/Roger Kulavich 467-7080

THERMAL PERFORMANCE

(12-Month Running Average)



Unit	1991	1992	1993	1994	1995	1/96	2/96	3/96	4/96	5/96	6/96	7/96	8/96	9/96	10/96	11/96	12/96
PTN 3	99.3	99.2	100.4	100.3	99.8	99.8	99.9										
PTN 4	98.7	99.0	99.7	99.7	100.1	100.1	100.1										
PSL 1	99.0	99.3	99.2	99.0	99.0	[illegible]9	98.9										
PSL 2	99.6	99.3	97.8	98.5	98.1	[illegible]2	98.2										
Industry Avg.	99.2	99.3	99.4	99.4	99.4												

DEFINITION

Thermal Performance is the ratio of the design gross heat rate (corrected) to the adjusted actual gross heat rate. Gross heat rate is defined as the ratio of total thermal energy produced by the reactor core to the total gross electrical energy produced by the generator during a given time period. Design gross heat rate (corrected) is the minimum theoretical heat rate that can be attained at design operating conditions for 100 percent power, expressed in British thermal units (BTUs) per kilowatt-hour(electric). Adjusted actual gross heat rate is the gross heat rate attained in the normal equipment lineup during one 24-hour period each month, expressed in BTUs per kilowatt-hour(electric) - power level should be greater than 80 percent.

STATISTICAL SUMMARY

	Feb	12-Mo. Average
PTN 3	100.0%	99.9%
PTN 4	100.0%	100.1%
PSL 1	99.2%	98.9%
PSL 2	99.3%	98.2%

1996 Target (all units): 99.5%

INDUSTRY PERFORMANCE

WANO

July 1994 - July 1995 Median	99.4%
1995 Goal	99.5%

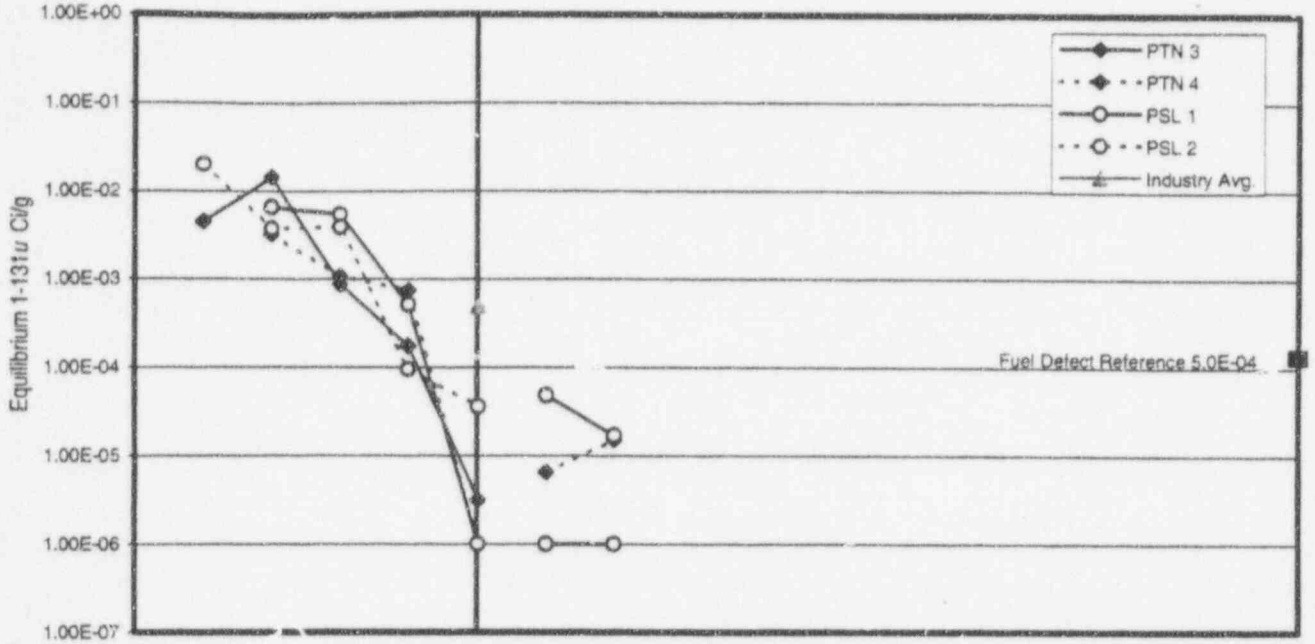
PERFORMANCE SUMMARY

- Turkey Point Units 3 & 4 Thermal Performance for February 1996 was higher than the year-end target. For 12-mo. running average, Units 3 and 4 performance was higher than WANO's 12-mo. running industry average.
- St. Lucie Units 1& 2 Thermal Performance for the month was below the year-end target. Performance for 12-mo. running was below WANO's 12-mo. running average.

Contacts: Jennifer Nicholson (PTN) 246-6827
Marty Smit (PSL) 467-7257

FUEL RELIABILITY

(Fuel Cycle/Monthly)



Unit	Previous Cycles					1/96	2/96	3/96	4/96	5/96	6/96	7/96	8/96	9/96	10/96	11/96	12/96
PTN 3	4.48E-3	1.44E-2	8.70E-4	1.75E-4	3.16E-6	1.00E-6	1.00E-6										
PTN 4		3.24E-3	1.08E-3	7.43E-4	1.00E-6	6.56E-6	1.51E-5										
PSL 1		6.54E-3	5.41E-3	5.17E-4	1.00E-6	4.82E-5	1.69E-5										
PSL 2	2.01E-2	3.71E-3	3.92E-3	9.50E-5	3.62E-5	1.00E-6	1.00E-6										
Industry Avg.					5.00E-4												

DEFINITION

This indicator is defined as the steady-state primary coolant iodine-131 activity (microcuries/gram) corrected for the tramp contribution and power level, and normalized to a common purification rate and average linear heat generation rate. The indicator value is calculated based on the average of the three monthly values for the most recent quarter of steady-state operation above 85 percent power. Steady state is defined as continuous operation for at least three days at a power level that does not vary more than ± 5 percent.

Note: If a calculated monthly value for a unit is less than 1.0E-6 microcuries per gram, the value is replaced by 1.0E-6 microcuries per gram.

STATISTICAL SUMMARY

	Feb	Cycle No.
PTN 3	1.00E-06	15
PTN 4	1.51E-05	15
PSL 1	1.69E-05	13
PSL 2	1.00E-06	9

Division Target for each unit: ≤ 4.50E-03

INDUSTRY PERFORMANCE

WANO

1994 Fuel Defect Reference Threshold (PWR)	5.00 E-04
1994 Median	7.36 E-05
1994 Best Quartile	3.00 E-06

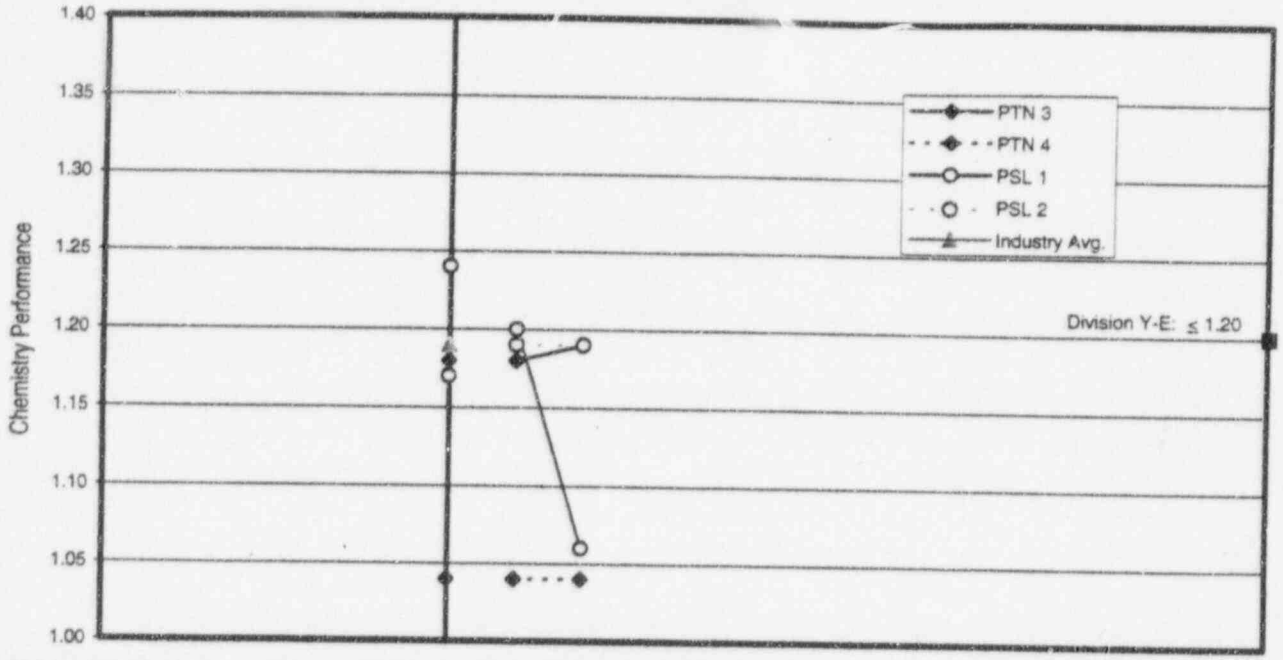
PERFORMANCE SUMMARY

- Unit 3 and 4's Fuel Reliability continues to indicate zero fuel defects in Cycle 15.
- For Unit 1, Reactor Coolant System radioisotopic data and spiking iodine following a reactor shutdown at the end of February and in July indicate the presences of one third-cycle failed fuel rod in the current Cycle 13.
- Unit 2 Fuel Reliability continues to indicate zero fuel defects in Cycle 9.

Data Source: Modesto Jimenez 694-3323

CHEMISTRY INDEX

(12-Months Running Average)



Unit	1991	1992	1993	1994	1995	1/96	2/96	3/96	4/96	5/96	6/96	7/96	8/96	9/96	10/96	11/96	12/96
PTN 3					1.18	1.18	1.19										
PTN 4					1.04	1.04	1.04										
PSL 1					1.24	1.20	1.06										
PSL 2					1.17	1.19	1.19										
Industry Avg.					1.19												

DEFINITION

The Chemistry Index compares the concentration of selected impurities to the limiting values for those impurities. Each impurity value is divided by the limiting value for the impurity, and the sum of these ratios is normalized to 1.0. The limiting values are the "achievable values" defined by international industry-accepted values.

STATISTICAL SUMMARY

	Feb	Y-T-D	12-Mo. Ending
PTN 3	1.05	1.04	1.19
PTN 4	1.04	1.04	1.04
PSL 1	1.07	1.04	1.06
PSL 2	1.16	1.22	1.19

1996 Target for each unit: ≤ 1.20

INDUSTRY PERFORMANCE

WANO

(PWR's with Recirculating Steam Generators not on Molar Ratio Control)

July 1994 - June 1995 Median	1.19
1995 Lowest Chemistry Index Value Attainable	1.00

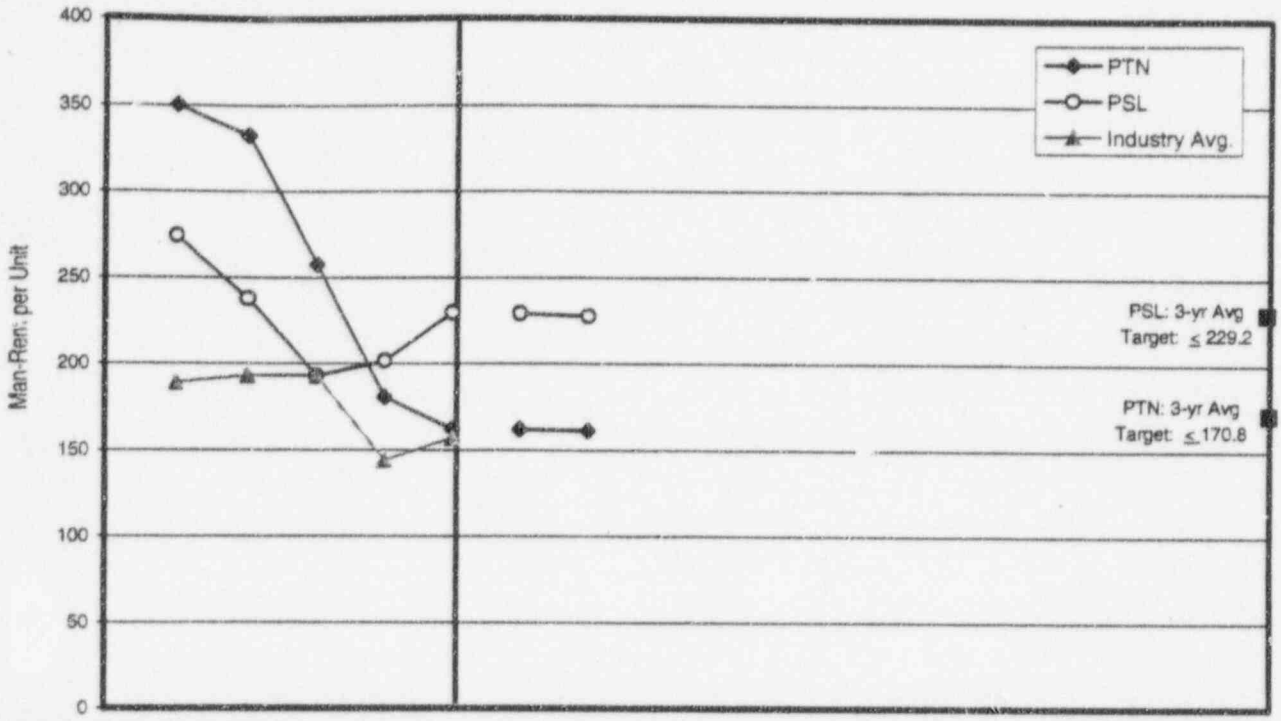
PERFORMANCE SUMMARY

- Turkey Point and St. Lucie's Chemistry Index performance for 12-mo. running was below the year-end target of ≤ 1.20 and the industry median.

Data Sources:
Bob Frechette (PTN) 246-6118
R. Steinke (PSL) 465-3213

RADIATION EXPOSURE

(3-year Running Average)



Plant	1991	1992	1993	1994	1995	1/96	2/96	3/96	4/96	5/96	6/96	7/96	8/96	9/96	10/96	11/96	12/96
PTN	350	332	257	181	162	162	161										
PSL	274	238	193	202	230	229	227										
Industry Avg.	189	193	193	144	157												

DEFINITION

Collective Radiation Exposure is the total external whole-body dose received by all personnel (including contractors and visitors) coming on site during a time period, as measured by the primary dosimeter, thermoluminescent dosimeter (TLD) or film badge. Exposure measured by direct reading dosimeters should be included only for those periods or situations when more accurate data is not available to the utility from TLD's or film badges. In order to correlate this indicator with the new 10 CFR 20 reporting guidelines, U.S. utilities report deep dose equivalent (DDE) and the total effective dose equivalent (TEDE).

STATISTICAL SUMMARY

3-yr Running per Unit

PTN	161.2
PSL	227.4

Division Targets:	3-yr. Avg
PTN: Y-E thru 1996	≤ 170.8
PSL: Y-E thru 1996	≤ 229.2

INDUSTRY PERFORMANCE

WANO

	Man-Rem (per Unit)
(PWR's) Median 3-yr Distribution (7/92 - 6/95)	157
1995 Goal	185

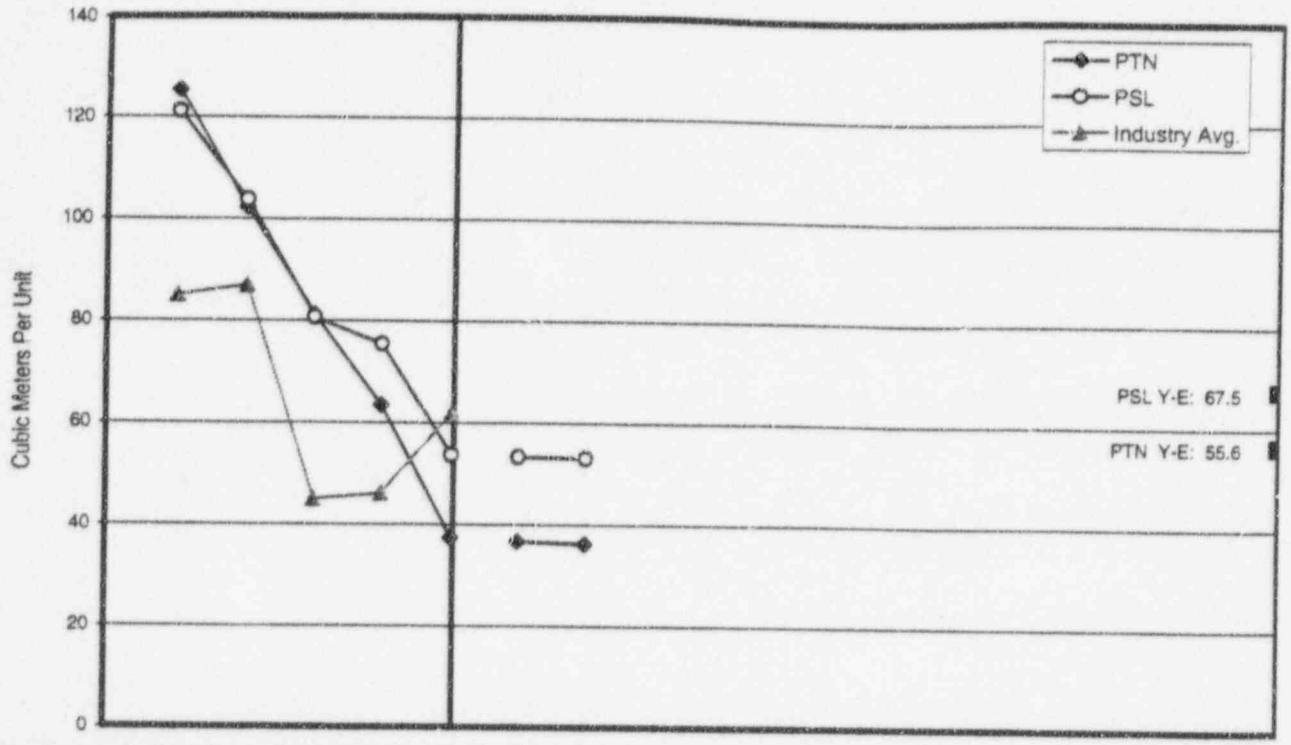
PERFORMANCE SUMMARY

- Turkey Point Collective Radiation Exposure for 3-years running through February 1996 was 161.2; performance was slightly higher than WANO's 3-yr. running median of 157.
- St. Lucie's Collective Radiation Exposure for 3-years running through February 1996 of 227.4 was higher than WANO's 3-yr. running median.

Data Providers: (PTN) John Lindsey 246-6548 (PSL) H. M. Mercer 467-7302

LOW-LEVEL SOLID RADIOACTIVE WASTE

(3-year Running Average)



Plant	1991	1992	1993	1994	1995	1/96	2/96	3/96	4/96	5/96	6/96	7/96	8/96	9/96	10/96	11/96	12/96
PTN	125.3	102.3	81.2	63.4	37.4	36.7	36.2										
PSL	121.1	103.8	80.7	75.4	53.7	53.3	53.0										
Industry Avg.	85.0	87.0	45.0	46.0	62.0												

DEFINITION

This indicator is defined as the volume of low-level solid radioactive waste that has been processed and is in final form (for example, compacted or solidified) ready for disposal during a given period. It is calculated using the amount of waste in final form, including the container, actually shipped for disposal from both on-site and off-site facilities, plus the change in inventory of final-form waste in storage at both on-site and off-site facilities.

STATISTICAL SUMMARY

3-yr Running thru Feb (per unit)

PTN	36.2
PSL	53.0

Division Targets:

PTN 1996 Y-E	67.5 cu. mtr. per unit
PSL 1996 Y-E	55.6 cu. mtr. per unit

INDUSTRY PERFORMANCE

WANO

Monthly Cu. Ft. (2 Unit Site)

Median 3-yr Distribution Median (1992 - 1994)	62.0
1995 12-Mo. average ending June 1995	29.0
1995 Goal	110.0

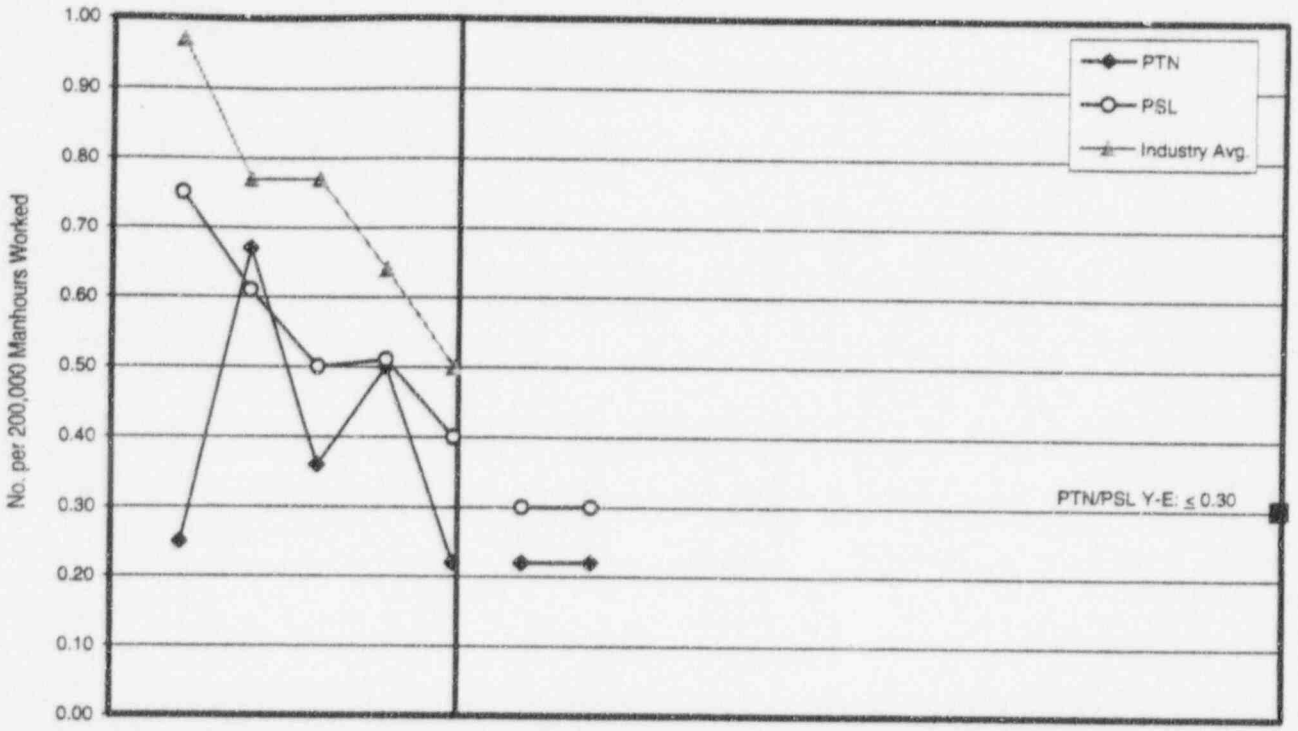
PERFORMANCE SUMMARY

- Turkey Point and St. Lucie's Solid Waste Disposal performance for 3-yrs. running was below WANO's 3-yr. industry median.

Data Sources: Bob Schuber 246-7227
Bruce Somers 467-7305

INDUSTRIAL SAFETY ACCIDENT RATE

(12-Month Running Average)



Plant	1991	1992	1993	1994	1995	1/96	2/96	3/96	4/96	5/96	6/96	7/96	8/96	9/96	10/96	11/96	12/96
PTN	0.25	0.67	0.36	0.50	0.22	0.22	0.22										
PSL	0.75	0.61	0.50	0.51	0.40	0.30	0.30										
Industry Avg.	0.97	0.77	0.77	0.64	0.50												

DEFINITION

Industrial Safety Accident Rate is defined as the number of accidents per 200,000 man-hours worked for all utility personnel permanently assigned to the station that result in any of the following: (1) one or more days of restricted work (excluding the day of the accident); (2) one or more days away from work (excluding the day of the accident); and, (3) fatalities. Contractor personnel are not included for this indicator.

Industrial Safety Accident Rate = (number of restricted-time + lost-time accidents + fatalities) x 200,000
(number of station man-hours worked)

STATISTICAL SUMMARY

	12-Mo -Ending (Feb)
PTN	0.22
PSL	0.30
Division Targets:	
PTN 1996 Year-End	0.30
PSL 1996 Year-End	0.30

INDUSTRY PERFORMANCE

WANO

July 1994 - June 1995 Median	0.51
1994 Average	0.64
1995 Goal	0.50

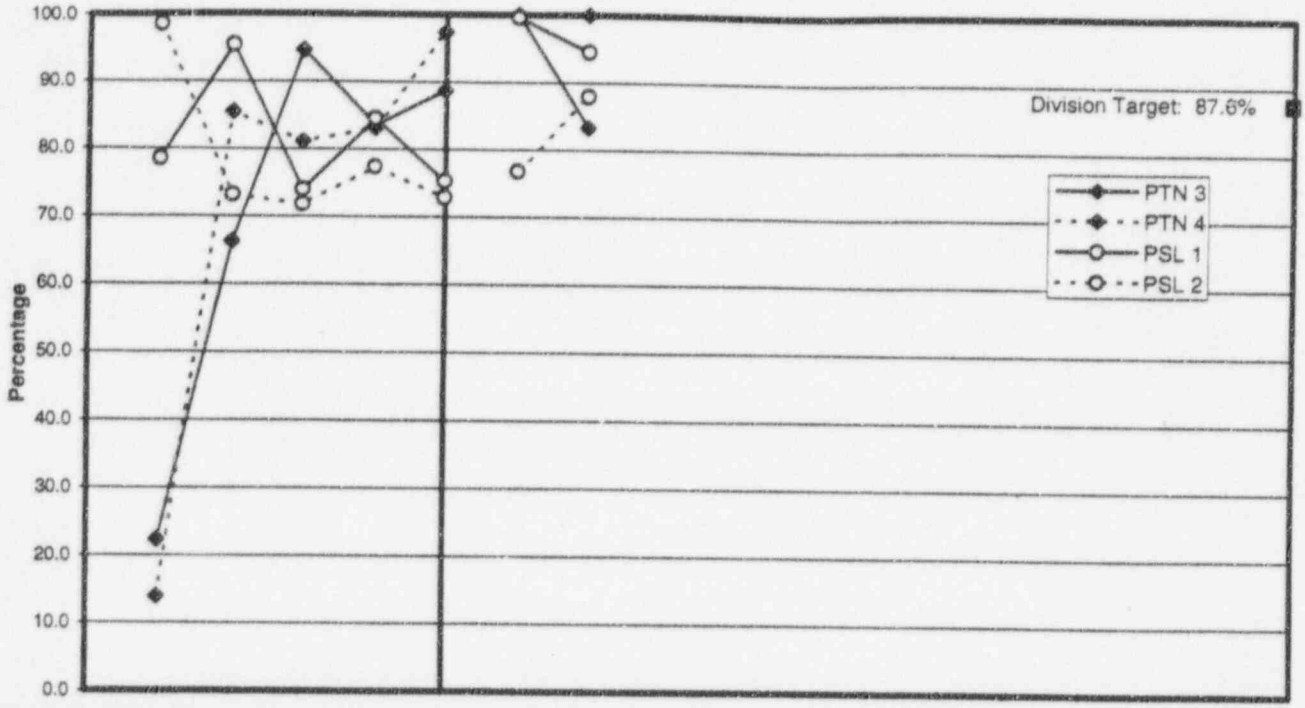
PERFORMANCE SUMMARY

- Turkey Point's Industrial Safety Accident Rate for 12-mo. running through February 1996 was below the year-end target and industry median.
- St. Lucie's rate of 0.30 for 12-mo. running through February 1996 was equal to the year-end target and below the industry median.

Contact: W. Korte 694-4235

EQUIVALENT AVAILABILITY FACTOR

(Year-to-Date)



Unit	1991	1992	1993	1994	1995	1/96	2/96	3/96	4/96	5/96	6/96	7/96	8/96	9/96	10/96	11/96	12/96
PTN 3	22.3	66.3	94.6	83.5	86.7	99.9	83.3										
PTN 4	13.9	85.5	81.1	83.1	97.4	100.0	100.0										
PSL 1	78.5	95.4	74.0	84.5	75.3	99.6	94.5										
PSL 2	96.5	73.2	71.8	77.4	72.9	76.8	87.9										

DEFINITION

Equivalent Availability Factor (EAF) is the ratio of the actual energy production capability to the energy that would be produced operating at full power for the same period expressed as a percent. Equivalent Availability provides an indication of the effectiveness of plant programs and practices in maximizing electrical generation.

$$EAF\% = \frac{\text{Available Hours - (Equivalent Unit Derated Hours)}}{\text{Period Hours}} \times 100\%$$

STATISTICAL SUMMARY

	Feb	Y-T-D	Y-T-D Target	3-yr Run. Avg
PTN 3	65.6%	83.3%	95.0%	88.6%
PTN 4	100.0%	100.0%	93.5%	87.3%
PSL 1	88.9%	94.5%	94.8%	77.8%
PSL 2	99.8%	87.9%	95.0%	77.7%
Division	88.6%	91.4%	94.6%	82.8%

1996 Division Y-E Target: 87.6%
1996 Year-End Forecast: 87.1%

INDUSTRY PERFORMANCE

NERC/GADS	
1994 (PWR's)	77.8%
1994 (All types)	74.1%
1990-1994 (PWR's)	74.4%
1990 - 1994 (All types)	71.2%
WANO	
July 1994 - July 1995 Median	82.4%
July 1994 - July 1995 Average	77.4%

PERFORMANCE SUMMARY

Major contributors to February 1996 Equivalent Availability loss for the Division was as follows:

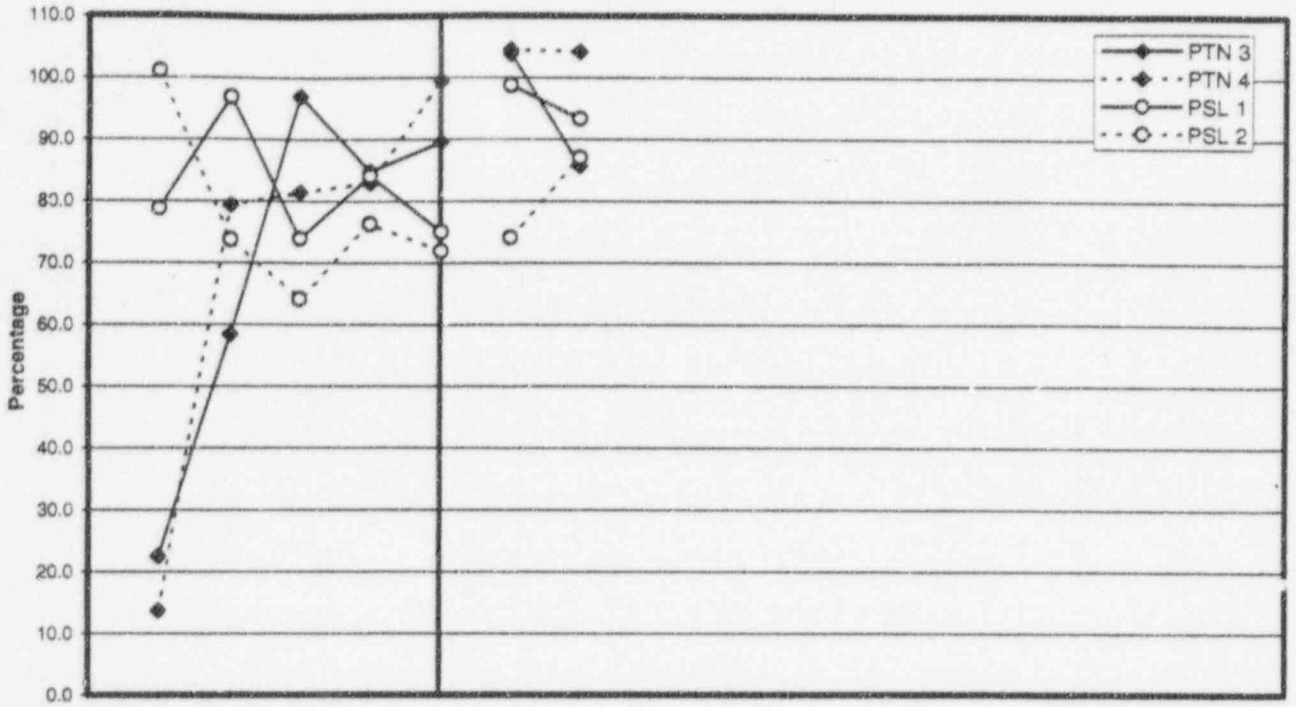
PTN Unit 3: Reactor Trip Steam Generator High on 2/9/96 (54 hours); Off-line grass influx (83 hours); and Rod Control System failure (32 hours).

PSL Unit 1: Dropped CEA #47 on 2/23/96 (64 hours).

Data source: GADS Report

CAPACITY FACTOR (MDC NET)

(Year-to-Date)



Unit	1991	1992	1993	1994	1995	1/96	2/96	3/96	4/96	5/96	6/96	7/96	8/96	9/96	10/96	11/96	12/96
PTN 3	22.5	58.4	97.0	84.8	89.5	103.9	85.7										
PTN 4	13.7	79.3	81.4	83.0	99.5	104.5	104.2										
PSL 1	76.8	96.9	73.9	84.1	75.0	98.8	93.3										
PSL 2	101.1	73.7	64.1	76.3	71.9	74.1	87.0										

DEFINITION

Capacity Factor (CF) is the index of the actual electrical energy produced by the unit with respect to its potential.

$$\text{Capacity Factor} = \frac{\text{Net Electrical Generation}}{\text{Maximum Dependable Capacity (839 or 666) X Period Hours}} \times 100$$

STATISTICAL SUMMARY

	Feb	Y-T-D	12-mo. ending
PTN 3	66.2%	85.7%	86.6%
PTN 4	103.9%	104.2%	99.5%
PSL 1	87.5%	93.3%	74.2%
PSL 2	100.8%	87.0%	70.8%
Division	89.6%	92.6%	82.8%

INDUSTRY PERFORMANCE

NERC/GADS	
1994 Avg PWR's	76.7%
1994 Avg All Types	72.8%
1990-1994 PWR's	72.8%
1990-1994 All Types	69.5%
WANO	
July 1994 - July 1995 Median	82.4%
July 1994 - July 1995 Average	77.4%

PERFORMANCE SUMMARY

Major contributors to February 1996 Capacity Factor loss for the Division was as follows:

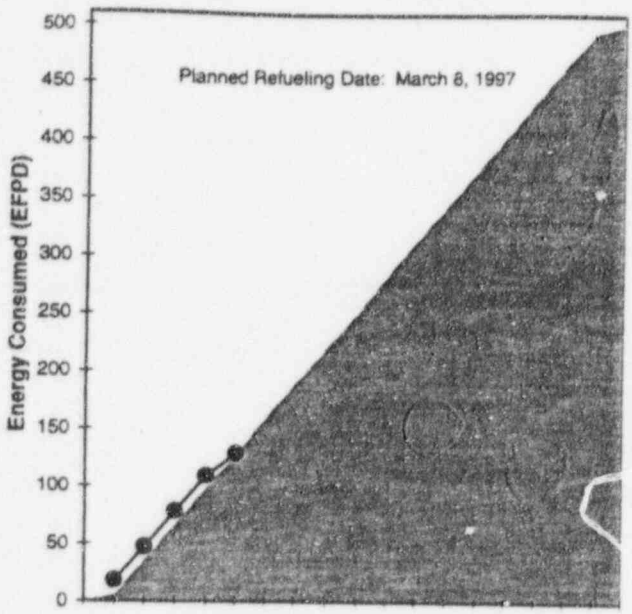
PTN Unit 3: Reactor Trip Steam Generator High on 2/9/96 (54 hours); Off-line grass influx (83 hours); and Rod Control System failure (32 hours).

PSL Unit 1: Dropped CEA #47 on 2/23/96 (64 hours).

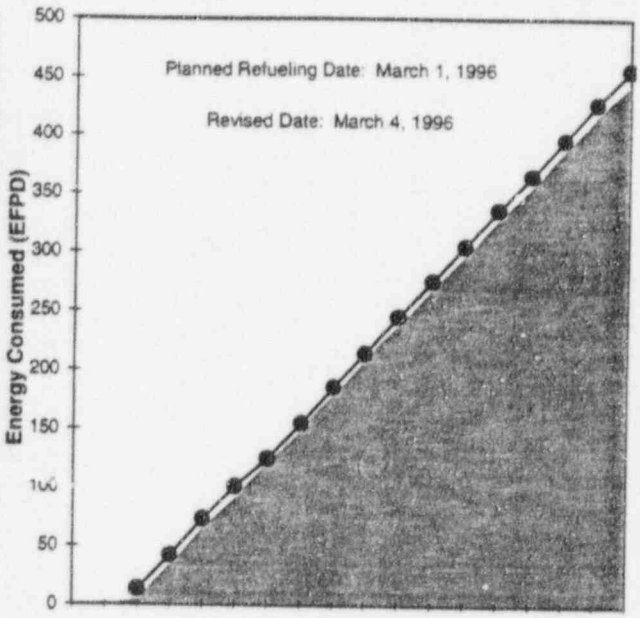
Data source: 1192 Report

PTN FUEL UTILIZATION FACTOR

Turkey Point Unit #3, Cycle 15



Turkey Point Unit #4, Cycle 15



PTN #3	Oct	Nov	Dec	Jan	Feb	Mar	Apr	May	Jun	Jul	Aug	Sep	Oct	Nov	Dec	Jan	Feb	Mar
Planned	5	35	66	98	127	158	188	219	249	281	312	342	373	403	435	466	494	501
Actual	18	47	78	109	128													
Variance (+/-)	13	12	12	11	1													
PTN #4																		
Planned		4	34	64	91	121	150	180	209	239	269	298	328	357	387	417	445	
Actual		13	42	73	101	125	155	186	215	246	276	306	337	366	397	426	457	
Variance (+/-)		9	8	9	10	4	5	6	6	7	7	8	9	9	10	11	12	

DEFINITION

Fuel utilization plots the amount of nuclear energy used during the current fuel cycle. The amount of nuclear energy loaded into the core is expressed in effective full power days (EFPD). One EFPD is the equivalent of operating the reactor at maximum thermal rating (2200 at PTN or 2700 at PSL) for a 24 hour period. Planned energy is compared to actual energy used during the cycle. Fuel utilization is directly related to plant performance. The significance of variance EFPD(+/-) is the difference between planned and actual energy consumption. Fuel utilization can be used to project longer or shorter operating fuel cycles.

Actual Energy / Calendar Days = Fuel Utilization Factor

Fuel Cycle Operating Assumptions

PTN 3: In accordance with the April 26, 1995 Approved Operating Schedule (AOS), Unit 3, Cycle 15 is scheduled to run 498 calendar days (October 27, 1995 to March 8, 1997) with design energy to run 501 (EFPD) days. NOTE: Due to the 34-day outage, startup began on October 7, 1995, 20 days earlier than planned.

PTN 4: Unit 4, Cycle 15 ended on March 4, 1996.

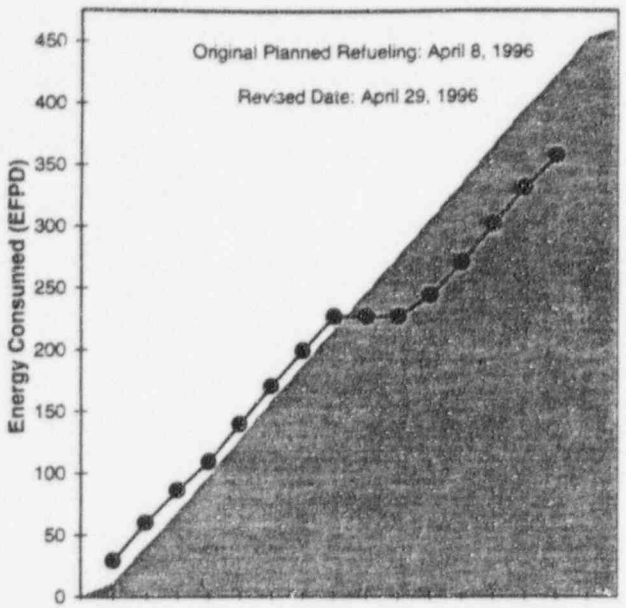
PERFORMANCE SUMMARY

- Unit 3 achieved a Fuel Utilization Factor of 65.0% and operated for 454 effective full power hours in the month of February. For the period October 27, 1995 to February 29, 1996, the Fuel Utilization Factor was 102.0%.
- Unit 4 achieved a Fuel Utilization Factor of 99.6% and operated for 693 effective full power hours in the month of February. For the period of November 27, 1994 to February 29, 1996, the Fuel Utilization Factor was 102.7%.

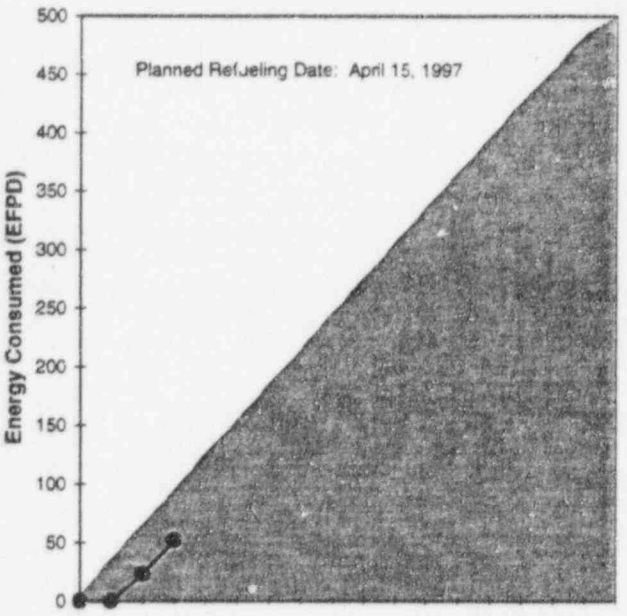
Data provider: Jimmie Perryman 694-3330

PSL FUEL UTILIZATION FACTOR

St. Lucie Unit #1, Cycle 13



St. Lucie Unit #2, Cycle 9



PSL #1	Nov	Dec	Jan	Feb	Mar	Apr	May	Jun	Jul	Aug	Sep	Oct	Nov	Dec	Jan	Feb	Mar	Apr
Planned		11	41	68	98	127	158	187	217	247	276	306	335	366	396	423	453	461
Actual		29	60	86	109	139	170	199	227	227	227	244	271	302	331	357		
Variance (+/-)		18	19	18	11	12	12	12	10	-20	-49	-62	-64	-64	-65	-66		
PSL #2																		
Planned	5	36	67	95	126	155	186	215	247	276	306	336	366	397	427	455	485	500
Actual	0	0	23	52														
Variance (+/-)	-5	-36	-44	-43														

DEFINITION

Fuel utilization plots the amount of nuclear energy used during the current fuel cycle. The amount of nuclear energy loaded into the core is expressed in effective full power days (EFPD). One EFPD is the equivalent of operating the reactor at maximum thermal rating (2200 at PTN or 2700 at PSL) for a 24 hour period. Planned energy is compared to actual energy used during the cycle. Fuel utilization is directly related to plant performance. The significance of variance EFPD(+/-) is the difference between planned and actual energy consumption. Fuel utilization can be used to project longer or shorter operating fuel cycles.

Actual Energy / Calendar Days = Fuel Utilization Factor

Fuel Cycle Operating Assumptions

PSL 1: In accordance with the October 28, 1994 Approved Operating Schedule (AOS), Unit 1, Cycle 13 was scheduled to begin operation December 20, 1994. This provided for a cycle of 475 calendar days with design energy to run 461 effective full power days (EFPD). Unit 1, Cycle 13 actually began operation November 29, 1994 and is currently scheduled to refuel April 29, 1996. NOTE: Per the revised AOS dated February 8, 1996, the unit will shutdown 21 days later than previously scheduled.

PSL 2: In accordance with the April 26, 1995 AOS, Unit 2, Cycle 9 was scheduled to begin operation November 24, 1995. This provided for a cycle of 508 calendar days with design energy to run 500 EFPD. Unit 2, Cycle 9 actually began operation January 5, 1996 and is currently scheduled to refuel April 15, 1997.

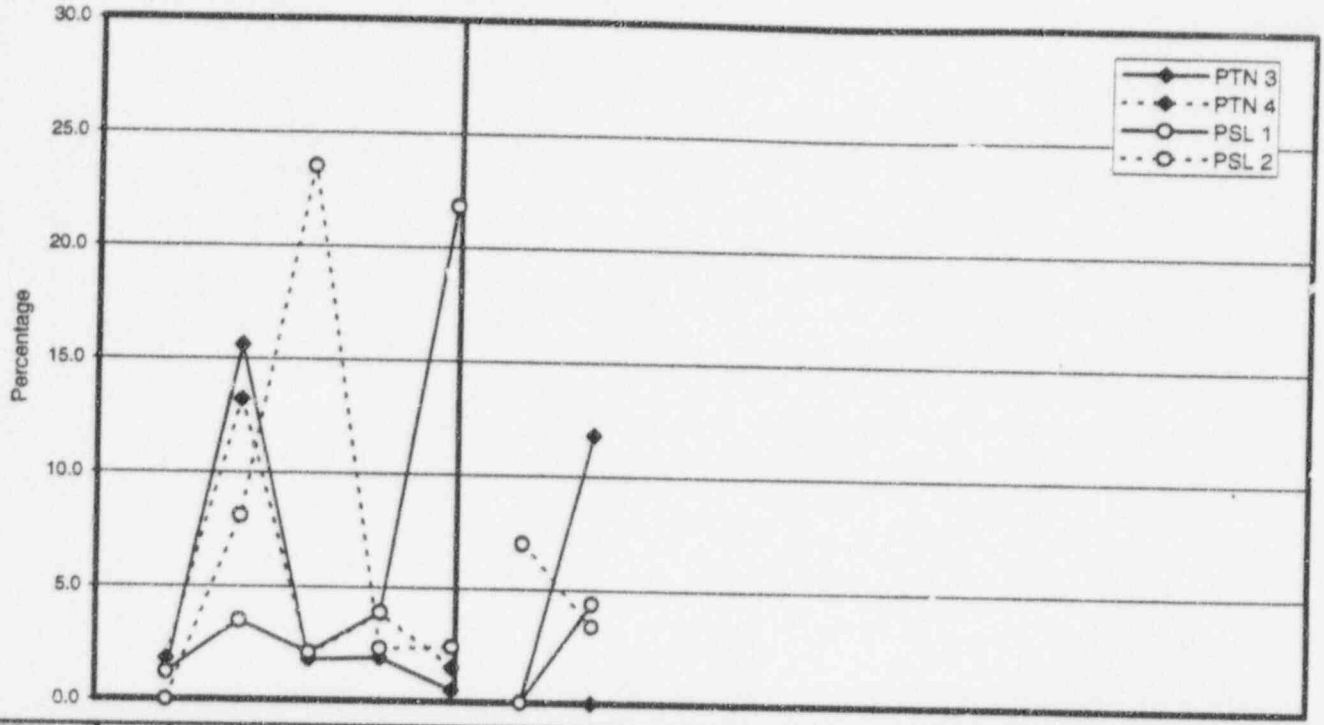
PERFORMANCE SUMMARY

- Unit 1 achieved a Fuel Utilization Factor of 81.2% and operated at 565 effective full power hours in the month of February. For the period of December 20, 1995 through February 29, 1996, the Fuel Utilization Factor was 78.1%.
- Unit 2 achieved a Fuel Utilization Factor of 99.3% and operated at 691 effective full power hours in the month of February. For the period of November 24, 1995 through February 29, 1996, the Fuel Utilization Factor was 54.7%.

Data provider: Ruben Rodriquez 694-3345

FORCED OUTAGE RATE

(Year-to-Date)



Unit	1991	1992	1993	1994	1995	1/96	2/96	3/96	4/96	5/96	6/96	7/96	8/96	9/96	10/96	11/96	12/96
PTN 3	1.2	15.6	1.8	1.9	0.5	0.0	11.8										
PTN 4	1.8	13.2	2.0	3.9	1.5	0.0	0.0										
PSL 1	1.2	3.5	2.1	3.9	21.8	0.0	4.4										
PSL 2	0.0	8.1	23.5	2.3	2.4	7.0	3.4										

DEFINITION

Forced Outage Rate is the percentage of time that the unit was unavailable due to forced events compared to the time planned for full power operation. A forced outage exists when the unit requires immediate removal from service, i.e. the unit is not synchronized to the grid. This type of outage usually results from immediate mechanical/electrical/hydraulic controls systems and operator-initiated trips in response to unit alarms.

$$\text{Forced Outage Rate} = \frac{\text{Forced Outage Hours}}{\text{Forced Outage Hours} + \text{Service Hours}} \times 100\%$$

STATISTICAL SUMMARY

	Feb	Y-T-D	12 Mo. Ending
PTN 3	24.3%	11.8%	2.6%
PTN 4	0.0%	0.0%	1.5%
PSL 1	9.1%	4.4%	22.5%
PSL 2	0.0%	3.4%	2.0%

INDUSTRY PERFORMANCE

NERC/GADS

1994 (PWR's)	8.1%
1994 (All types)	10.6%
1990-1994 (PWR's)	7.6%
1990 - 1994 (All types)	10.8%

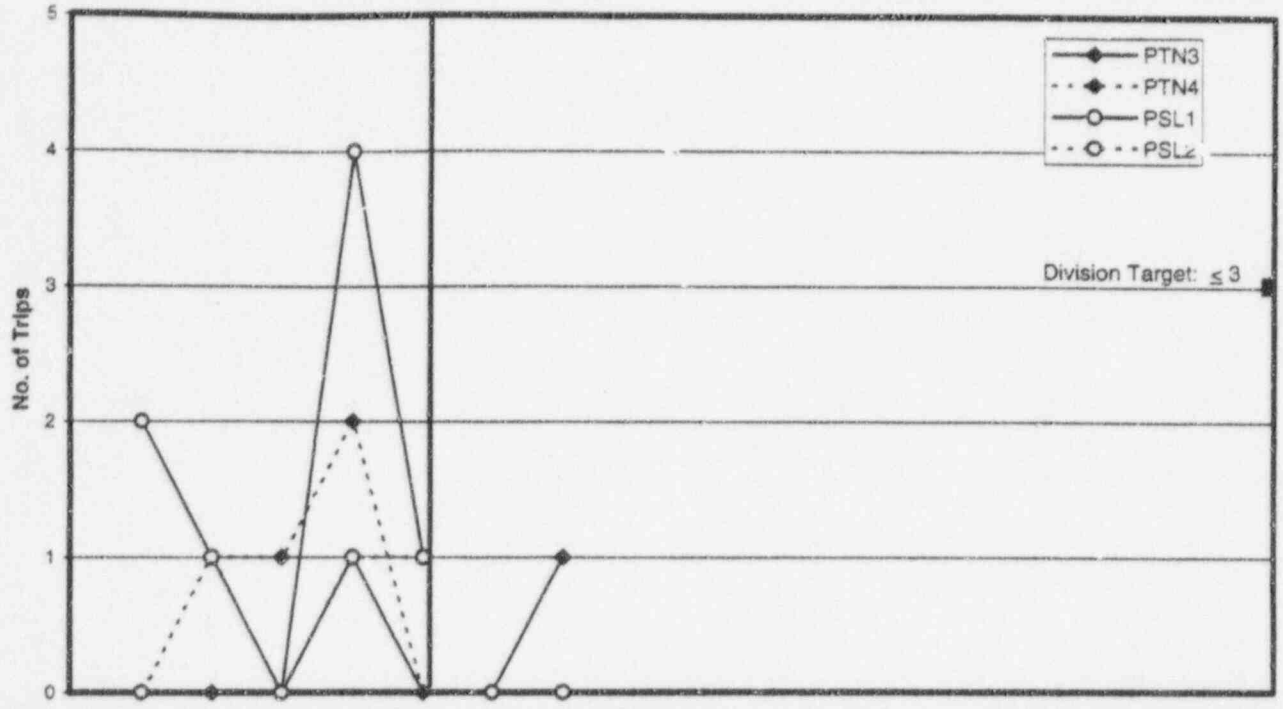
PERFORMANCE SUMMARY

- Turkey Point Unit 3 Forced Outage Rate for February resulted from an automatic trip, repair to the Feedwater Check Valve, repair to 3B Feedwater Heater Pump, and grass intrusion. Unit 3's 12-month ending average was below industry median.
- Unit 1 Forced Outage Rate for February was the result of a dropped CEA rod. Unit 1's 12-month ending average of 22.5% was higher than the industry median.

Data Source: GADS Report

UNPLANNED AUTOMATIC TRIPS WHILE CRITICAL

(Year-to-Date)



Unit	1991	1992	1993	1994	1995	1/96	2/96	3/96	4/96	5/96	6/96	7/96	8/96	9/96	10/96	11/96	12/96
PTN3	0	0	0	1	0	0	1										
PTN4	0	1	1	2	0	0	0										
PSL1	2	1	0	4	1	0	0										
PSL2	0	1	0	1	1	0	0										

DEFINITION

An Uplanned Automatic Scram is a non-manual actuation of the reactor protection system that results in a scram signal any time the unit is critical. Scrams that are planned as part of special evaluations or tests are not included in this definition. This indicator provides an indication of success in improving plant safety by reducing the number of undesirable and unplanned thermal-hydraulic and reactivity transients requiring reactor scrams.

STATISTICAL SUMMARY

	Feb	Y-T-D	12-Month Ending
PTN 3	1	1	1
PTN 4	0	0	0
PSL 1	0	0	1
PSL 2	0	0	0
Division Total:	1	1	2

1996 Division Target: ≤ 3

INDUSTRY PERFORMANCE

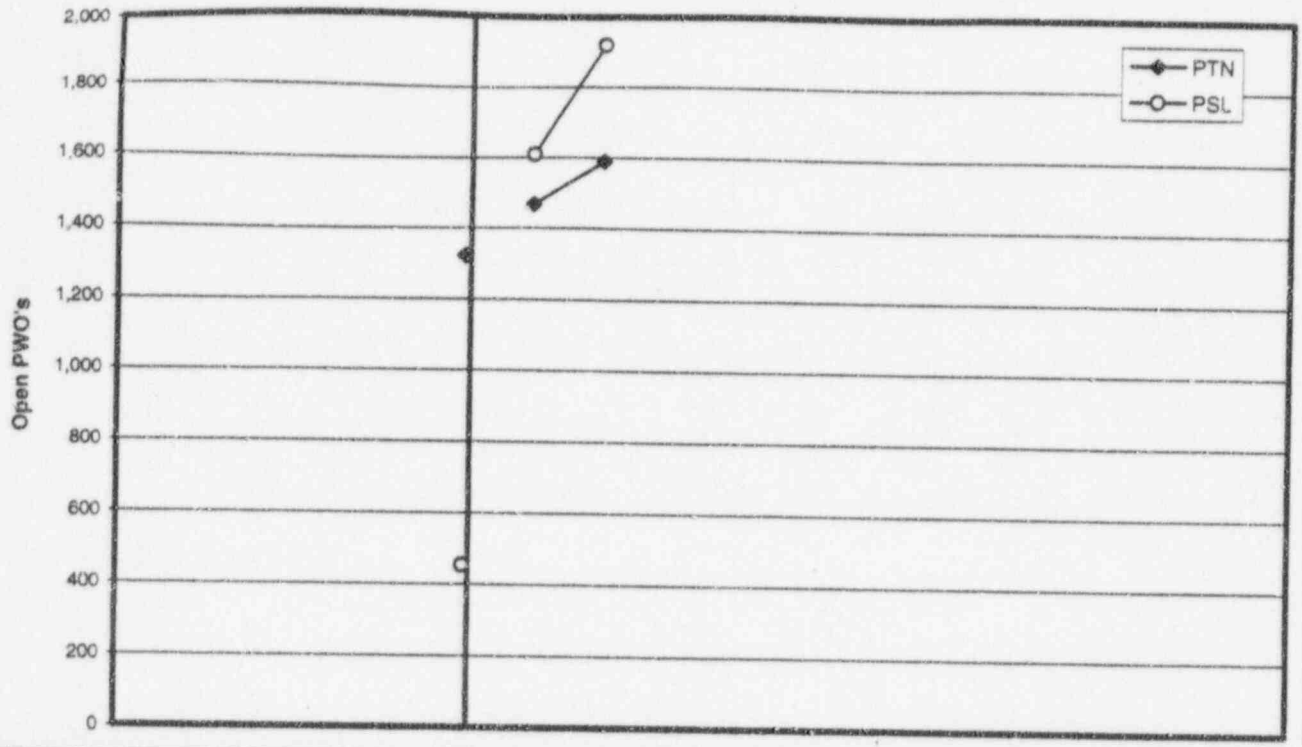
INPO	Trips per unit
Trips per 7000 Critical Hours	
3-yr Distribution Median (7/92-6/95)	1.0
1994 Median	0.8
1995 Goal	1.0
NRC (1995 3rd Qtr Performance Indicator Rpt)	
Quarterly Trips Annualized	1.0
Quarterly Trips per 7000 Critical Hours	1.2

PERFORMANCE SUMMARY

- Turkey Point Unit 3 experienced an automatic trip on February 9th; the "B" Steam Generator Feed Pump was stopped to monitor its discharge check valve closing stroke which did not strike closed as expected. The resulting feed flow transient caused the "C" Steam Generator level to increase resulting in a turbine trip which tripped the reactor.

Data Providers:
PTN: J. Knorr 246-6757
PSL: K. Korth 467-7054

OPEN PWO'S



Unit	1991	1992	1993	1994	1995	1/96	2/96	3/96	4/96	5/96	6/96	7/96	8/96	9/96	10/96	11/96	12/96
PTN					1,321	1,466	1,589										
PSL					454	1,608	1,920										

DEFINITION

This indicator includes Work Type 1 (Planned Miscellaneous), Work Type 3 (Projects), WorkType 5 (Trouble & Breakdown) and includes all hold codes, status 22 through 48.

STATISTICAL SUMMARY

INDUSTRY PERFORMANCE

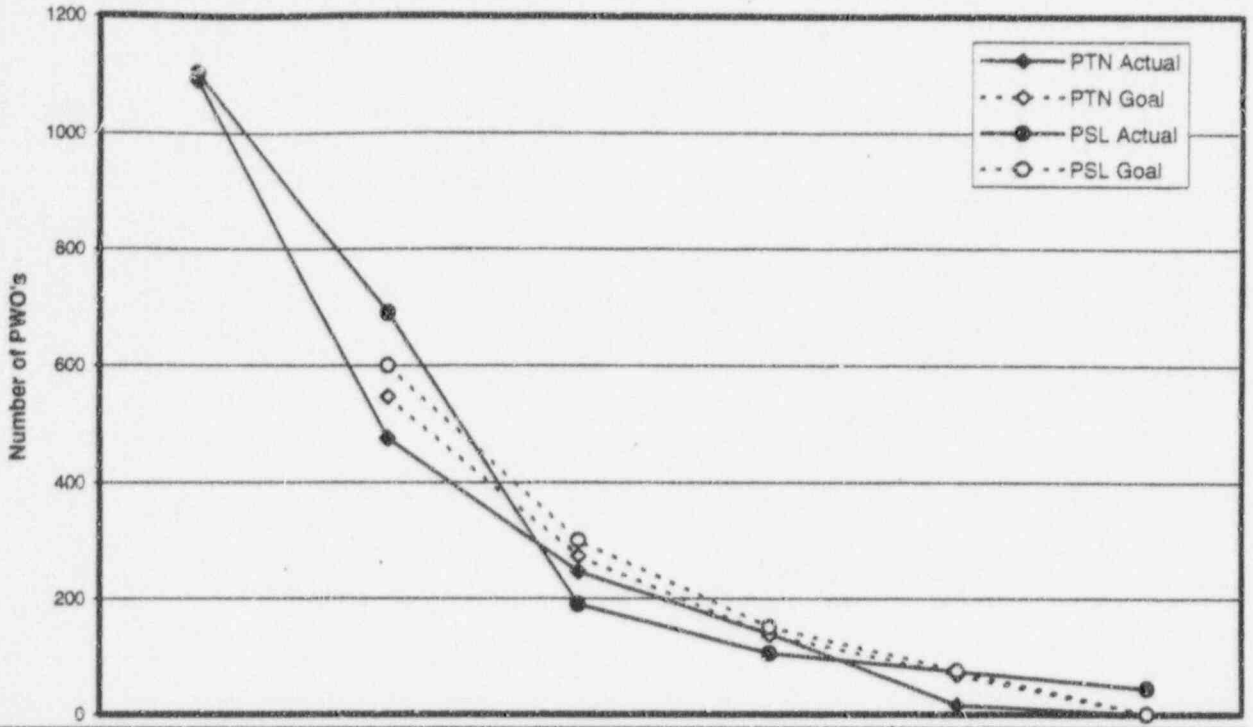
PERFORMANCE SUMMARY

Prior to 1996, each site was counting Open Pwo's differently. St. Lucie was counting only Work Type 5 non-outage corrective and Turkey Point was counting Work Types 1 and 5. Each site now agrees to report per the above definition.

St. Lucie numbers have increased due to intensive system walkdowns by Operations and Tech based on upper management directives. Also the February numbers now include 232 work type "3" which were excluded from the Jan report.

Data Providers:
PSL: Joe Marchese 467-7107
PTN: Greg Heisterman 246-6796

PWO AGING CURVE



Unit	TOTALS	3	6	9	12	Over 12
PTN Actual	1091	475	246	139	16	0
PTN Goal		546	273	137	68	0
PSL Actual	1103	690	190	105	74	44
PSL Goal		600	300	150	75	0

DEFINITION

PWO Aging curves includes Work Types 1 (Planned Misc) Work Type 3 (Projects) and Work Type 5 (Trouble & Breakdown), and excludes short notice outages (SNO, SNW), HC2-Startup, HC3-Hot Standby, HC4-Hot Shutdown, HC5-Cold Shutdown, and HC6-Refueling. This only includes status 22 through 48.

Goal: To halve the backlog every three months.

STATISTICAL SUMMARY

INDUSTRY PERFORMANCE

PERFORMANCE SUMMARY

Data providers:
PSL: Joe Marchese 467-7107
PTN: Greg Heisterman 246-6796

PWO'S GREATER THAN 12 MONTHS
(GOLDEN OLDIES)

TURKEY POINT

REF	PWO #	PWO TITLE	Orig. Date	Wks on List
1	95001652	CV-3-3729, Valve failed open (AWP/ENG)	01/13/95	
2	95005744	CV-3-6347B, Valve leaks	01/20/95	
3	95003154	4-821, Spindle assy. needs to be	01/26/95	
4	95005513	HSC-3-152, Brittle/crack insul. on wires	01/27/95	
5	95005512	HSC-3-153, Brittle/crack insul. on wires (AWP hold)	01/27/95	
6	95005514	HSC-3-151, Brittle/crack insul. on wires (AWP hold)	01/27/95	
7	9500[illegible]227	CV-3-6347B, Assist Mech. with valve repair	02/28/95	
8	9[illegible]6427	Various doors throughout plant, install drip flashings on doors	03/01/95	
9	95006428	D096-1 Door, Water intrusion	03/01/95	
10	95006604	DPC-3-3728, Adjust D/P	03/02/95	

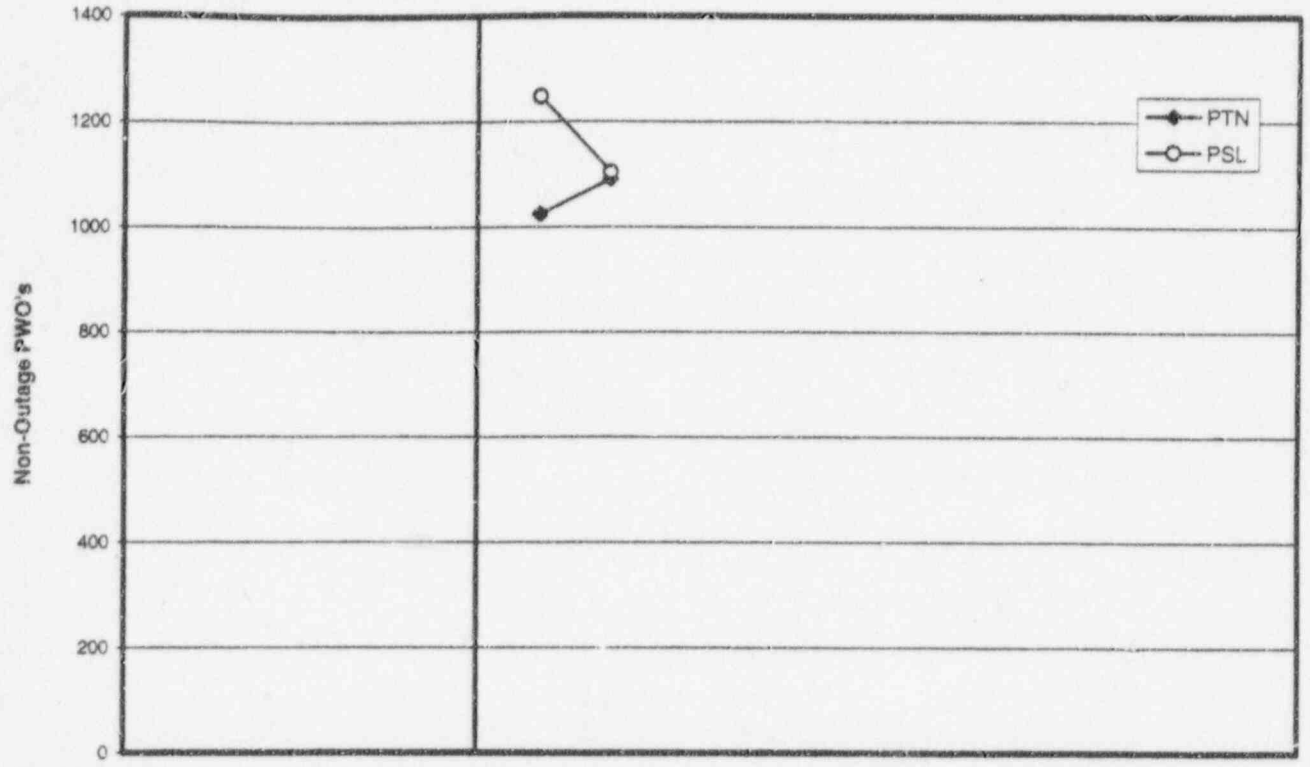
ST. LUCIE

1	91036158	LINE I-8-CW-99 HAS SEVERE PITS	11/29/91	
2	93021318	REPLACE MOTOR HEATERS.HEATERS DEFECTIVE	8/4/93	
3	93034478	OVERHAUL SPARE HVS-1A MOTOR.	12/30/93	
4	94006444	INSTALL REPLACEMENT VLV DURING SL1-14	3/19/94	
5	94015819	REPLACE LIS-07-2C AND PIS-07-2C WITH	6/23/94	
6	94018623	REBUILD SPARE COMPRESSOR MOTOR	7/25/94	
7	94018561	PUMP DEGRADING/OVERHAUL	7/25/94	
8	94020286	PLUG ON SEAL LEAKING	8/11/94	
9	94024396	DISASSEMBLE AND INSPECT REPAIR AS NEEDE	9/26/94	
10	94024431	DISASSEMBLE AND INSPECT REPAIR AS NEEDE	9/26/94	
11	94025159	REPLACE VALVES REMOVE DRIP PAN.	10/4/94	
12	94025282	UPPER LEFT COOLING FAN INOPERABLE.	10/5/94	
13	94025253	METER MOVEMENT STICKY AND OUT OF CAL	10/5/94	
14	94025159	REPLACE VALVES REMOVE DRIP PAN.	10/17/94	
15	94026357	U/2 XFMR COOLING FAN MOTOR REPAIR	10/20/94	
16	94027706	LAMP TEST/RELAY TEST SWITCH	11/5/94	
17	94028901	TEMP. REPAIR WAS WORKED ON W/O 94026229	11/19/94	
18	94029456	BROKEN "LB" AND CONDUIT 1031SG 1-1/2	11/25/94	
19	94029484	REPLACE SWITCH	11/26/94	
20	94029464	REPLACE BISTABLE MODULE 206 PRESSURIZER	11/26/94	
21	94029653	A-7 GAUGE GLASS AND ISO VALVE HAVE BEEN	11/28/94	
22	94029654	THE A-7 GAUGE GLASS IS REMOVED ON PWO	11/28/94	
23	94031063	REPAIR STEAM LEAK-REMOVE DRIP-PAN	12/13/94	
24	95000079	ASSIST MECHANICAL MAINTENANCE	1/3/95	
25	95000584	BROKEN ANNUNCIATOR WINDOWS	1/5/95	
26	95000764	REPLACE VALVE.	1/9/95	
27	95001003	INSTALL LEVEL INDICATOR FOR POOL LEVEL	1/11/95	
28	95001138	NO SAFE ACCESS TO THE TROLLEY	1/12/95	
29	95001552	EDGEWISE CONNECTOR(J1) BAD. REPLACE&TES	1/17/95	
30	95001726	GAUGE IS BUSTED	1/18/95	
31	95002431	U-1 GNTY TROLLY/BRIDGE MTR OH	1/25/95	
32	95002359	VALVE LEAKS BY SEAT/REPLACE	1/25/95	
33	95002361	VALVE LEAKS BY SEAT/REPAIR VALVE	1/25/95	
34	95002363	VALVE LEAKS BY SEAT/REPLACE	1/25/95	
35	95002356	VALVE LEAKS BY SEAT/REPAIR VALVE	1/25/95	
36	94018623	REBUILD SPARE COMPRESSOR MOTOR	2/1/95	
37	95003580	VENT FANS AGING/ERDADS COMPUTER CABINET	2/3/95	
38	95003560	FANS ARE AGING.REPLACE FANS.	2/3/95	
39	95003699	U-2 DSG 2A ANN. 6.2 ALARM	2/6/95	
40	95004053	REPAIR VALVE LEAK-REMOVE DRIPPAN	2/9/95	
41	95004054	OIL LEAK FROM PUMP TO FLOOR	2/9/95	
42	95004363	MOTOR SUPPORT PLATE IS NOT FLAT, CAUSIN	2/13/95	
43	95004362	VLV LEAKS BY SEAT	2/13/95	
44	95005489	ANNUCIATOR C-26 CAME IN AT HIGHER VALUE	2/23/95	
45	95005523	CHECK THE LIMITS ON SOLENOID.	2/23/95	
46	95005529	GAUGE PEGGED LOW/RECALIBRATE GAUGE	2/23/95	
47	95005529	GAUGE PEGGED LOW/RECALIBRATE GAUGE	2/23/95	

PWO Greater than 12 month's, Work Types 1 through 5, excluding outage hold codes (SNO, SNW, HC2, HC3, HC4, HC5, HC6) status 22-48.

Data Providers: (PTN) Greg Heisterman 246-6796 (PSL) Joe Marchese 467-7107

NON-OUTAGE PWO'S



Unit	1991	1992	1993	1994	1995	1/96	2/96	3/96	4/96	5/96	6/96	7/96	8/96	9/96	10/96	11/96	12/96
PTN						1024	1091										
PSL						1247	1103										

DEFINITION

Non-Outage PWO's includes Work Types 1 (Planned Misc) Work Type 3 (Projects) and Work Type 5 (Trouble & Breakdown), and excludes all outage hold codes. Short notice outages (SNO, SNW), HC2-Startup, HC3-Hot Standby, HC4-Hot Shutdown, HC5-Cold Shutdown, HC6-Refueling and includes status 22-48.

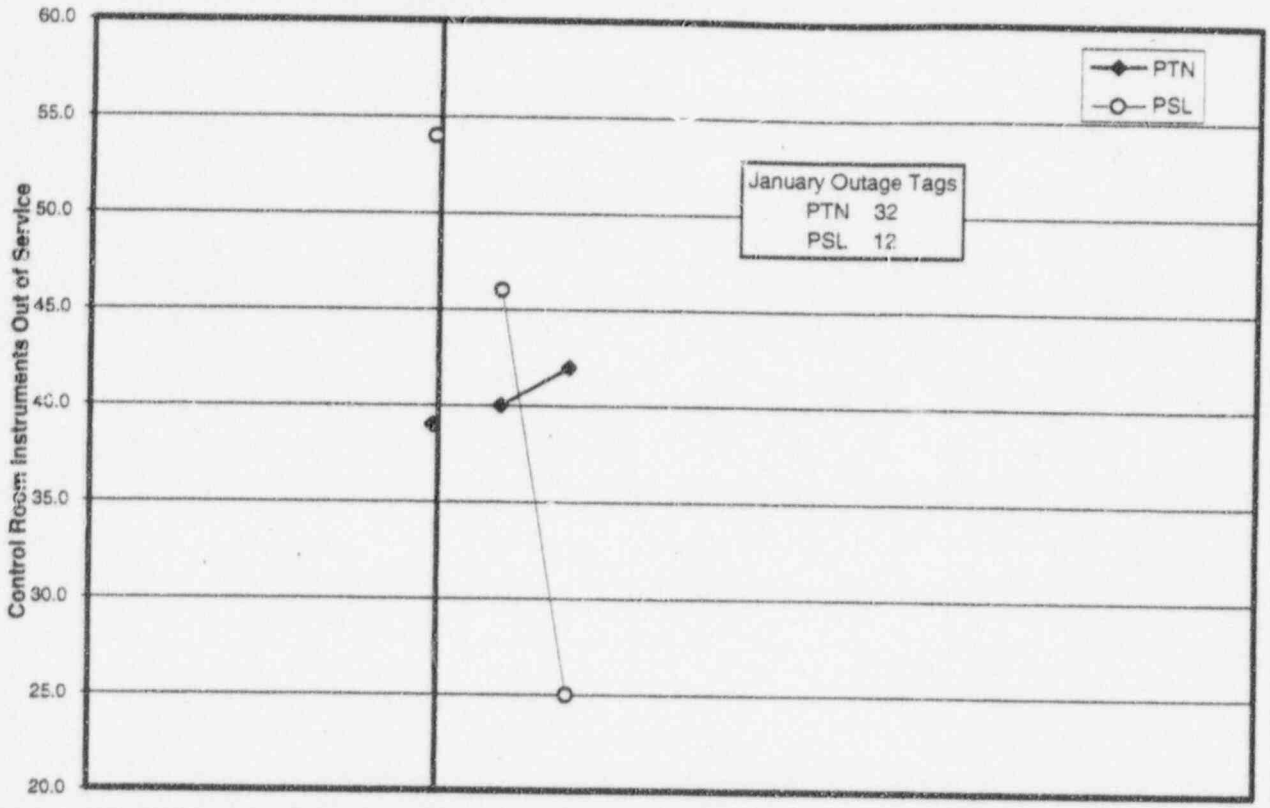
STATISTICAL SUMMARY

INDUSTRY PERFORMANCE

PERFORMANCE SUMMARY

Data providers:
PSL: Joe Marchese 467-7107
PTN: Greg Heisterman 246-6796

CONTROL ROOM INSTRUMENTS OUT-OF-SERVICE



Unit	1991	1992	1993	1994	1995	1/96	2/96	3/96	4/96	5/96	6/96	7/96	8/96	9/96	10/96	11/96	12/96
PTN					39	40	42										
PSL					54	46	25										

DEFINITION

This indicator defines the number of control room instruments for each unit that cannot perform their design function, regardless of the reason. Instruments on the control room back panels are readily available for use by the control room crews and are included; however, instruments in adjoining areas where operators are not normally stationed (such as computer rooms) are not included. Count deficency tags that are in status 05 to 48.

STATISTICAL SUMMARY

INDUSTRY PERFORMANCE

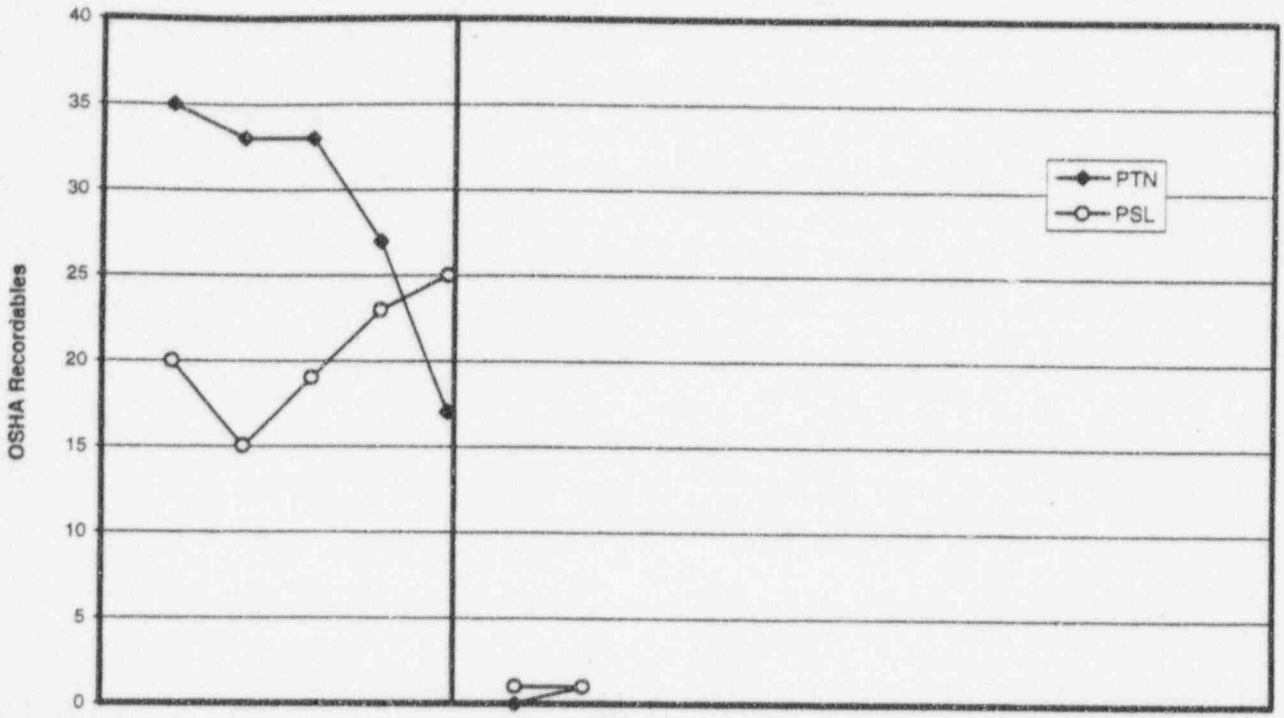
PERFORMANCE SUMMARY

A correction has been made to PTN data for Year End 1995 and January. The data reflects the corrected numbers.

Data providers:
PSL: Joe Marchese 467-7107
PTN: Greg Heisterman 246-6796

OSHA RECORDABLES

(Year-to-Date)



Unit	1991	1992	1993	1994	1995	1/96	2/96	3/96	4/96	5/96	6/96	7/96	8/96	9/96	10/96	11/96	12/96
PTN	35	33	33	27	17	0	1										
PSL	20	15	19	23	25	1	1										

DEFINITION

The definition by Occupational Safety Health Administration (OSHA) is an injury occurring on the job that requires medical treatment beyond first aid as defined by 29 CFR 1904.

STATISTICAL SUMMARY

	Month	Y-T-D
PTN	1	1
PSL	0	0

Nuclear Division Y-E Target: 1.75
Corporate Y-E Target: 3.75

INDUSTRY PERFORMANCE

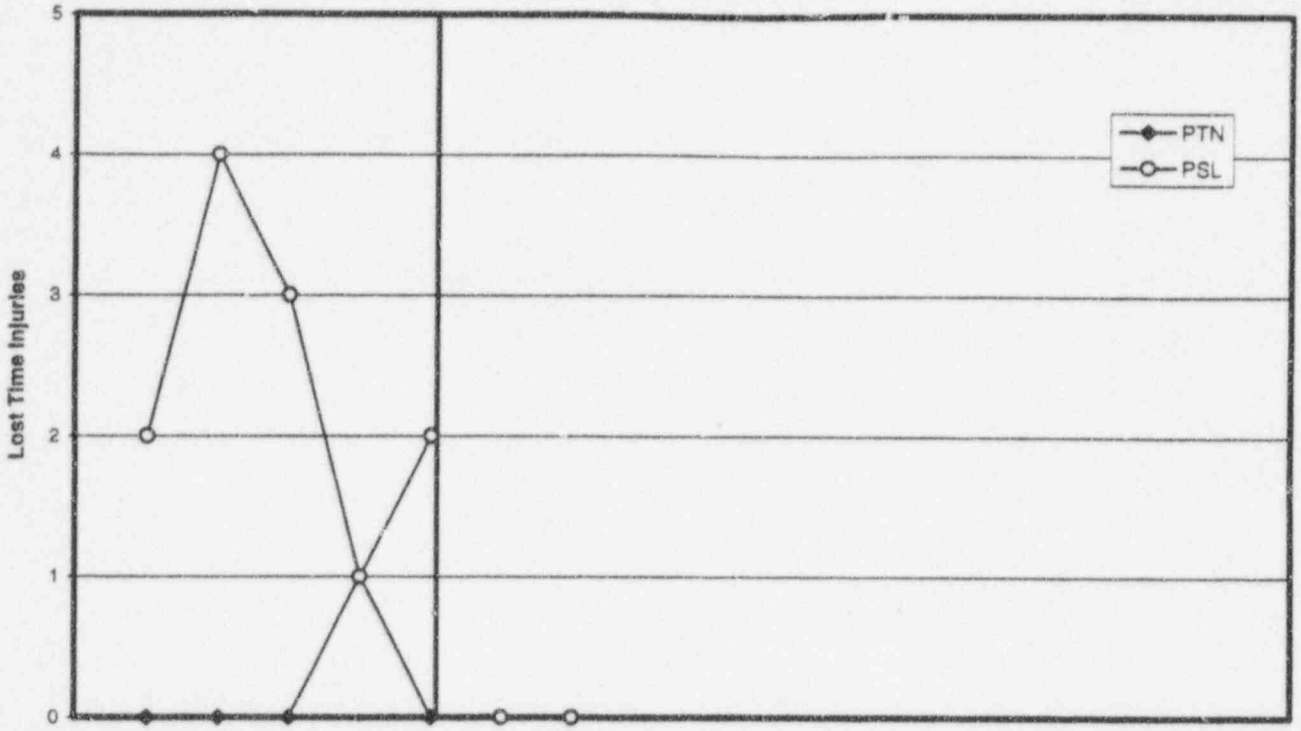
PERFORMANCE SUMMARY

One OSHA Recordable Injury was reported at Turkey Point; a mechanic cut his finger requiring sutures when the crane window on the cask crane closed on his finger.

Data provider: W. Korte 694-4235

LOST TIME INJURIES

(Year-to-Date)



Unit	1991	1992	1993	1994	1995	1/96	2/96	3/96	4/96	5/96	6/96	7/96	8/96	9/96	10/96	11/96	12/96
PTN	0	0	0	1	0	0	0										
PSL	2	4	3	1	2	0	0										

DEFINITION

A Lost Time Injury as defined by Occupational Safety Health Administration (OSHA) is an occupational injury that requires an employee to miss a full day (8 hour shift) beyond the day of injury.

STATISTICAL SUMMARY

	Month	Y-T-D
PTN	0	0
PSL	0	0

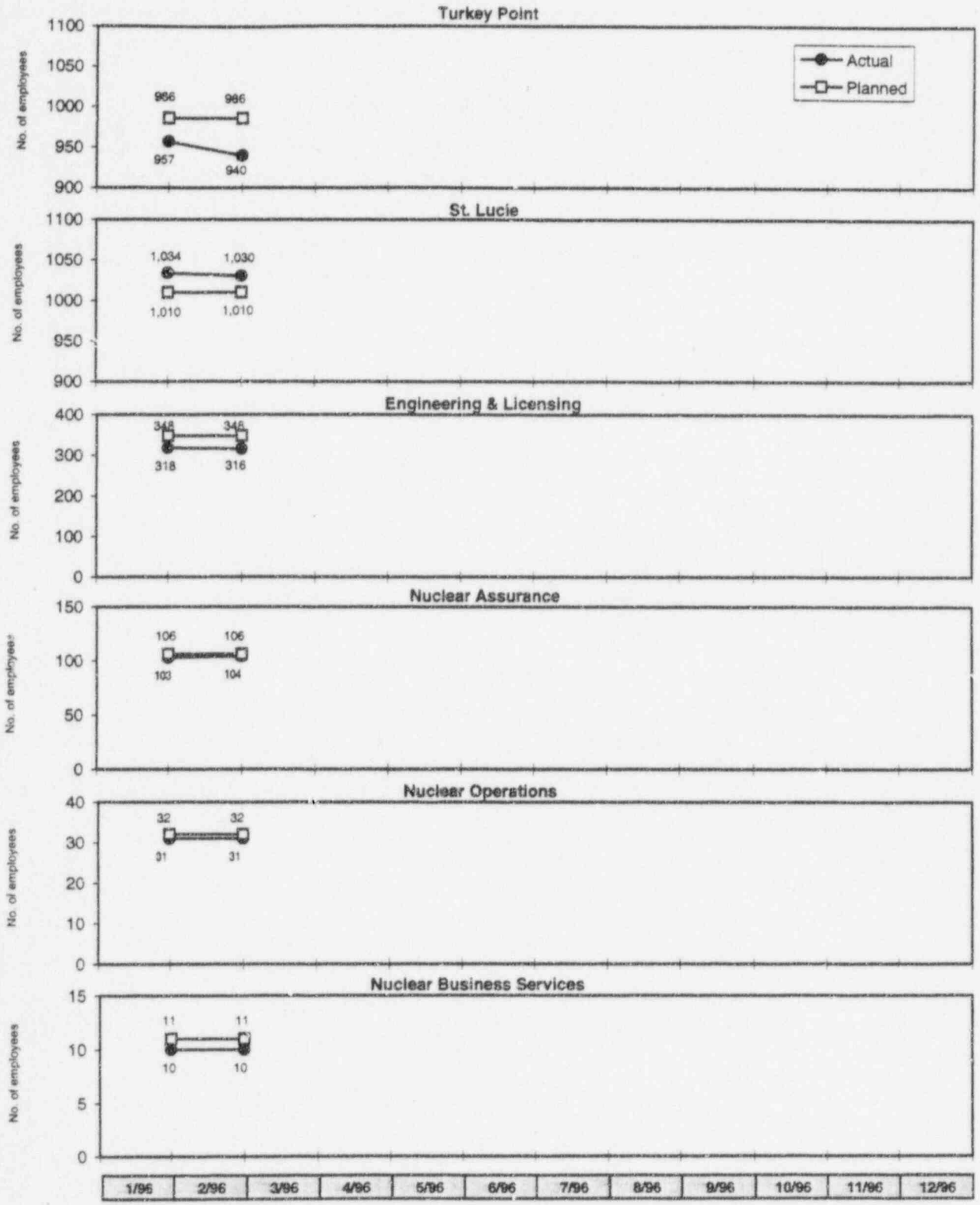
INDUSTRY PERFORMANCE

PERFORMANCE SUMMARY

No Lost Time Injuries were reported in February 1996.

Data provider: W. Korte 694-4235

REGULAR STAFFING
FPL Employees and Long-Term Contractors by Department



Data Provider: Alicia Simpson 694-3275
Data Source: ND Staffing Report

OPERATOR EXAMINATION PERFORMANCE
Turkey Point Units 3 & 4

Exams:	1991	1992	1993	1994	1995	1/96	2/96	3/96	4/96	5/96	6/96	7/96	8/96	9/96	10/96	11/96	12/96
RO/SRO Taken	39	5	-	8	13	-	-										
Passed	39	5	-	8	13	-	-										
RO/SRO Pass %	100.0%	100.0%		100.0%	100.0%												
Oper Req Taken	47	78	69	63	58	-	64										
Passed	44	75	66	60	56	-	64										
Oper Req Pass %	94.0%	96.0%	96.0%	95.0%	96.5%		100.0%										

DEFINITION

Initial License Examination (RO/SRO) results are reported for all candidates taking an Initial License Exam as conducted by the NRC.

Operator Requalification Examination results are reported for both RO's and SRO's. This examination is administered annually by the utility and may be jointly administered by the NRC. Retests of operators who failed examinations are not included.

STATISTICAL SUMMARY

Initial RO/SRO License Exams

	No. Taken	No. Passed	Pass Rate %
YTD 1996	0	0	0.0%
YTD 1995	13	13	100.0%

Operator Requal Exams

	No. Taken	No. Passed	Pass Rate %
YTD 1996	64	64	100.0%
YTD 1995	58	56	96.5%

INDUSTRY PERFORMANCE

The NRC at their last Regional Training Managers Meeting (for fiscal year 1994) provided the following data:

Initial NRC Exams		NRC Requal Exams	
RO's Pass Rate	94.6%	RO's Pass Rate	91.0%
Instant SRO's Pass Rate	94.4%	SRO's Pass Rate	85.0%
Upgrade SRO's Pass Rate	94.7%	Average Overall	88.0%
Average Overall	94.6%		

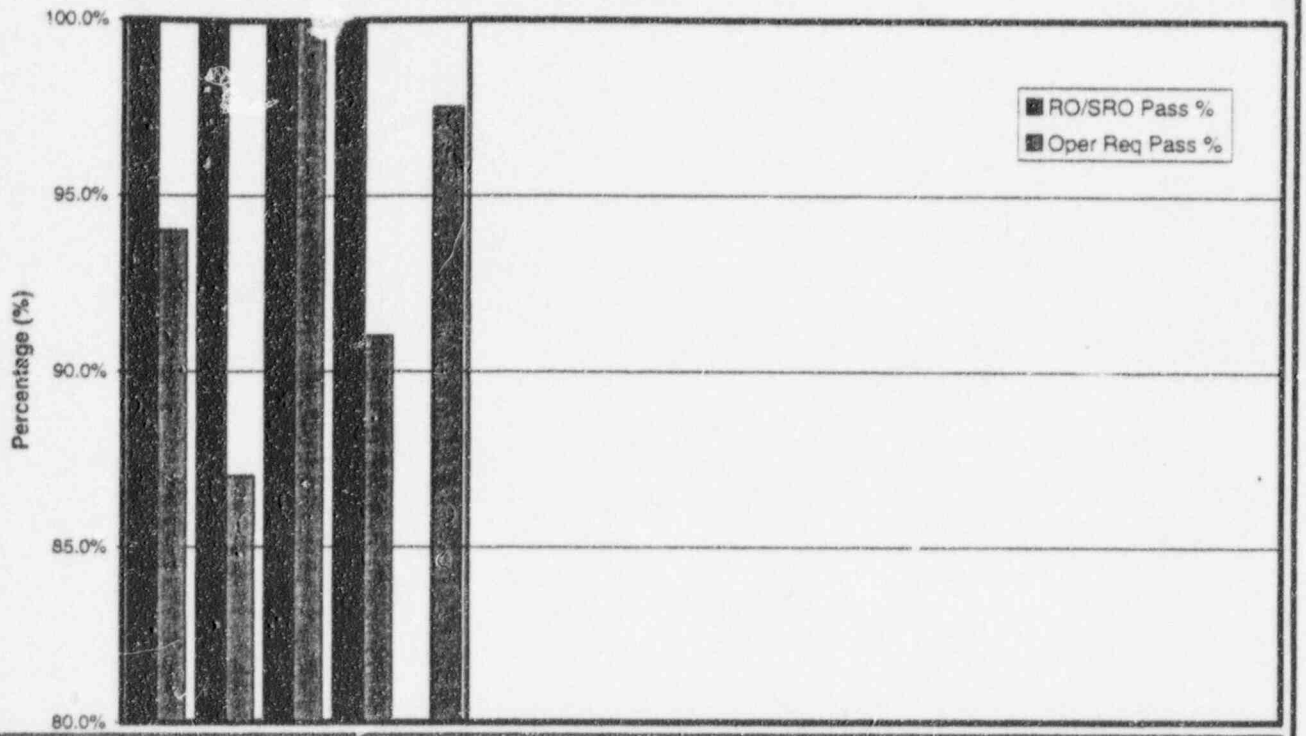
PERFORMANCE SUMMARY

No Initial RO/SRO License Exams were given in February 1996.

Operator Requalification Exams were given in January and February. Of the 64 operators tested, 64 passed.

Data provider: K. E. Beatty 694-4217

OPERATOR EXAMINATION PERFORMANCE
St. Lucie Units 1 & 2



Exams:	1991	1992	1993	1994	1995	1/96	2/96	3/96	4/96	5/96	6/96	7/96	8/96	9/96	10/96	11/96	12/96
RO/SRO Taken	15	8	10	11	-	-	-										
Passed	15	8	10	11	-	-	-										
RO/SRO Pass %	100.0%	100.0%	100.0%	100.0%	-												
Oper Req Taken	65	69	68	75	73	-	-										
Passed	61	60	68	68	71	-	-										
Oper Req Pass %	94.0%	87.0%	100.0%	91.0%	97.5%												

DEFINITION

Initial License Examination (RO/SRO) results are reported for all candidates taking an Initial License Exam as conducted by the NRC.

Operator Requalification Examination results are reported for both RO's and SRO's. This examination is administered annually by the utility and may be jointly administered by the NRC. Retests of operators who failed examinations are not included.

STATISTICAL SUMMARY

Initial RO/SRO License Exams

	No. Taken	No. Passed	Pass Rate %
YTD 1996	0	0	0.0%
YTD 1995	0	0	0.0%

Operator Requal Exams

	No. Taken	No. Passed	Pass Rate %
YTD 1996	0	0	0.0%
YTD 1995	73	71	97.5%

INDUSTRY PERFORMANCE

The NRC at their last Regional Training Managers Meeting (for fiscal year 1994) provided the following data:

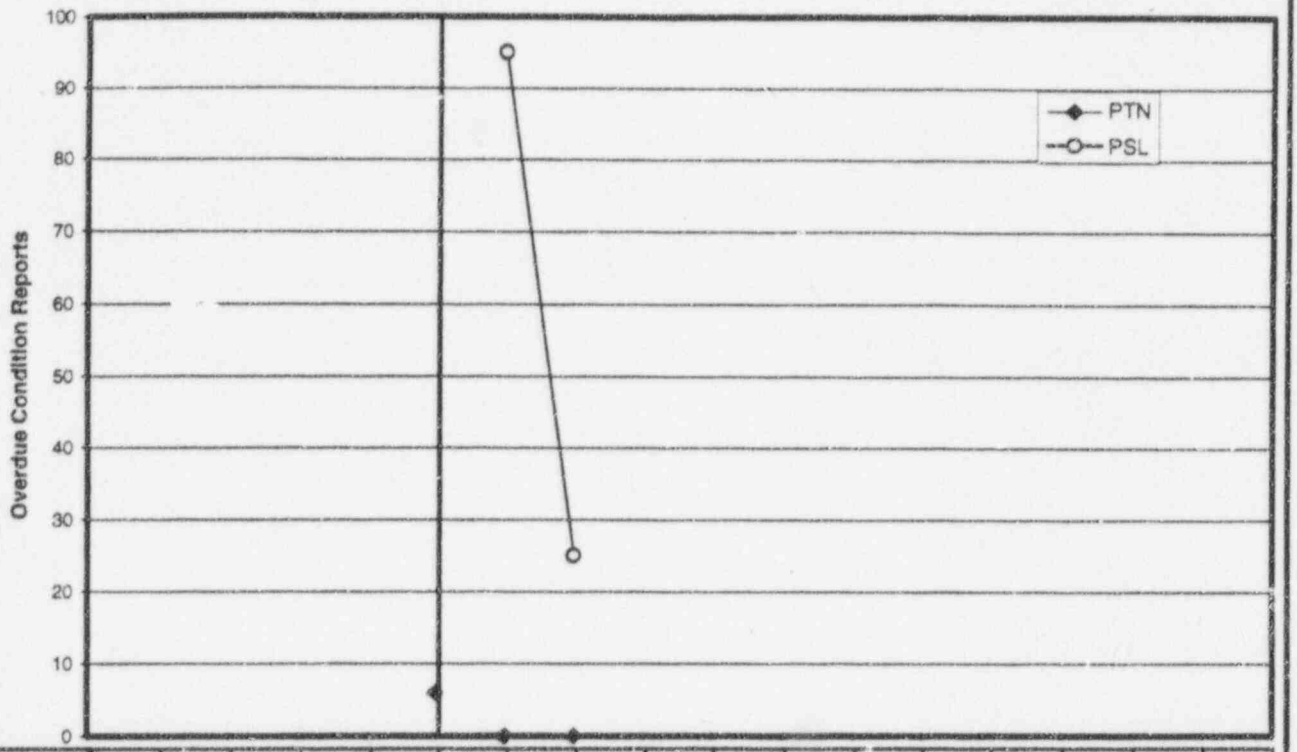
Initial NRC Exams		NRC Requal Exams	
RO's Pass Rate	94.6%	RO's Pass Rate	91.0%
Instant SRO's Pass Rate	94.4%	SRO's Pass Rate	85.0%
Upgrade SRO's Pass Rate	94.7%	Average Overall	88.0%
Average Overall	94.6%		

PERFORMANCE SUMMARY

No Initial RO/SRO License or Operator Requalification Exams were given in February 1996. Initial RO/SRO License Exams are scheduled to be given in March.

Data provider: K. E. Beatty 694-4217

OVERDUE CONDITION REPORTS



Unit	1991	1992	1993	1994	1995	1/96	2/96	3/96	4/96	5/96	6/96	7/96	8/96	9/96	10/96	11/96	12/96
PTN					6	0	0										
PSL						95	25										

DEFINITION

Currently, the Nuclear Division is transitioning to a consistent Condition Report. The project is scheduled to be completed by May 1, 1996 at all locations. This graph shows Condition Reports that exceed assigned priority timeframe.

Severity Levels are as follows:

A = 3 working days
B = 10 calendar days
C = 30 calendar days

STATISTICAL SUMMARY

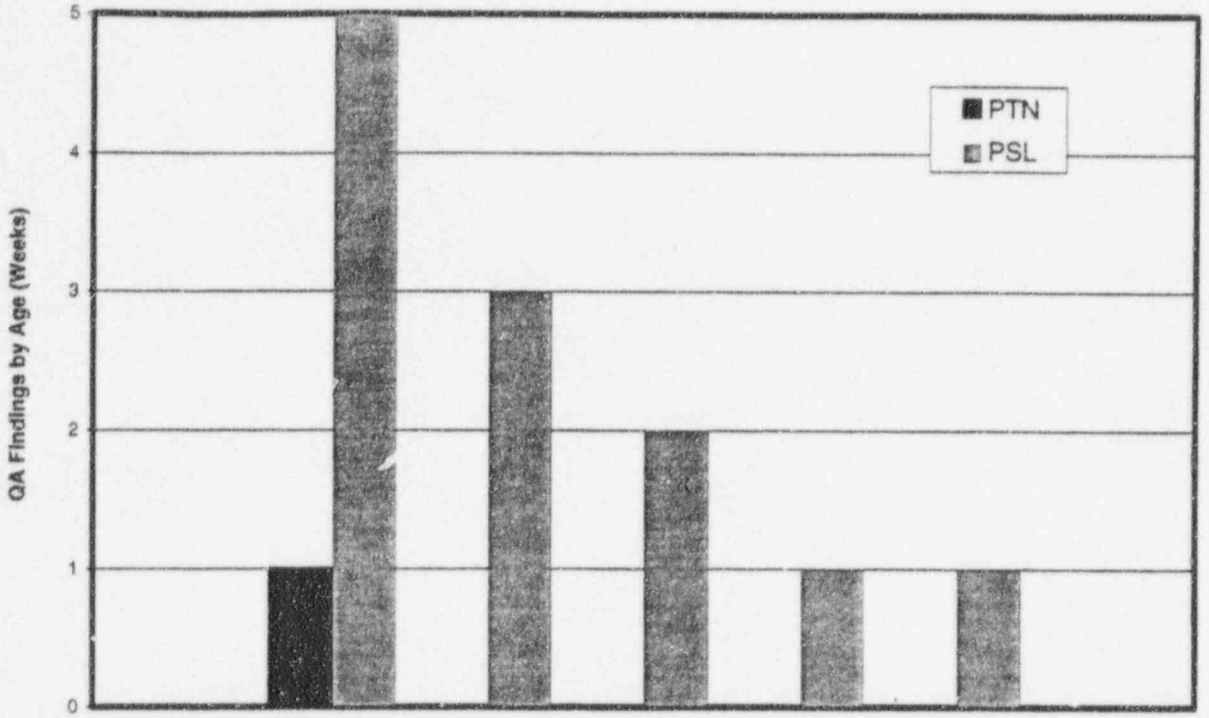
INDUSTRY PERFORMANCE

PERFORMANCE SUMMARY

St Lucie has 25 priority 1, 2 or 3 STAR's overdue as of 2/29/96. The STAR database includes both STAR's and PMAI's. St. Lucie is currently transferring to the CR process(by April 1). There are 740 STAR's remaining that predate the priority process that are being converted to CR's or PMAI's.

Data Providers: (PTN) Julio Balaguero 246-6971 and (PSL) Bob Dawson 467-7154

QA FINDINGS



Unit	0-3	4-6	7-9	10-12	13-15	16-18	19-Up
PTN	0	1	0	0	0	0	0
PSL	0	5	3	2	1	1	0

DEFINITION

This indicator represents the time it takes for verification of implementation of corrective action for audit findings. The clock for the QA group starts when the audited organization notifies QA that implementation of audit finding corrective actions are complete and ready for verification. The process is complete when QA closes the audit finding and notifies the audited organization that the audit finding corrective action is complete.

STATISTICAL SUMMARY

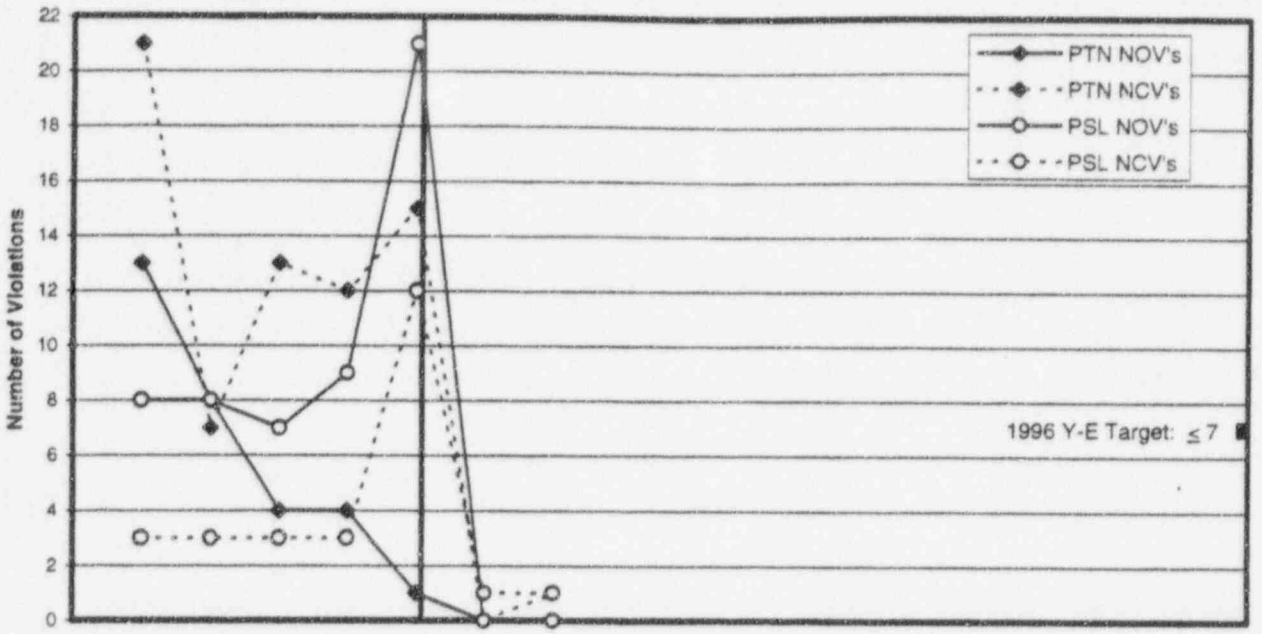
INDUSTRY PERFORMANCE

PERFORMANCE SUMMARY

Data Providers: (PTN) Tom Abbatiello 246-6[illegible] and (PSL) Wes Bladow 467-7111

NRC VIOLATIONS Cited and Non-Cited

(Year-to-Date)



Unit	1991	1992	1993	1994	1995	1/96	2/96	3/96	4/96	5/96	6/96	7/96	8/96	9/96	10/96	11/96	12/96
PTN NOV's	13	8	4	4	1	0	0										
PTN NCV's	21	7	13	12	15	0	1										
PSL NOV's	8	8	7	9	21	0	0										
PSL NCV's	3	3	3	3	12	1	1										

DEFINITION

Violations are categorized in terms of five levels of severity to show their relative importance. Severity Levels I and II are violations that involve actual or high potential impact on the public. Severity Level III Violations are cause for significant concern. Severity Level IV violations are less serious, but are of more than minor concern; i.e., if left uncorrected, could lead to a more serious concern. Severity Level V violations are minor safety or environmental concern. Violations are counted on the date of the inspection exit meeting. Violations are now counted with respect to the date of occurrence (using the date of the inspection exit meeting) instead of the date of the inspection report, as was done in the past.

STATISTICAL SUMMARY

	Feb	YTD	Year End Target
PTN Cited	0	0	7
PTN Non-Cited	0	1	
PSL Cited	0	0	7
PSL Non-Cited	0	1	

INDUSTRY PERFORMANCE

NRC Violations	
1994 IBG Group Mean	16.2
1994 Region II Mean	13.1
1994 IBG Top Quartile Mean	9.0
1994 Region II Top Quartile Entry	8.0

PERFORMANCE SUMMARY

Turkey Point reported no NRC Violations for February; one non-cited violation was reported as follows:

#96-01-01: Use of contaminated tools in clean system. Exit Meeting date: 2/12/96.

St. Lucie reported no NRC Violations for the month: however, there are four potential violations pending investigation:

#96-01-01: Temporary Procedure changes were made which involved changes of intent, without prior FRG review as required by Technical Specifications. Exit Meeting Date: 2/20/96.

#96-03-01: Operators failed to follow procedures for boron dilution. Exit Meeting Date: 2/8/96.

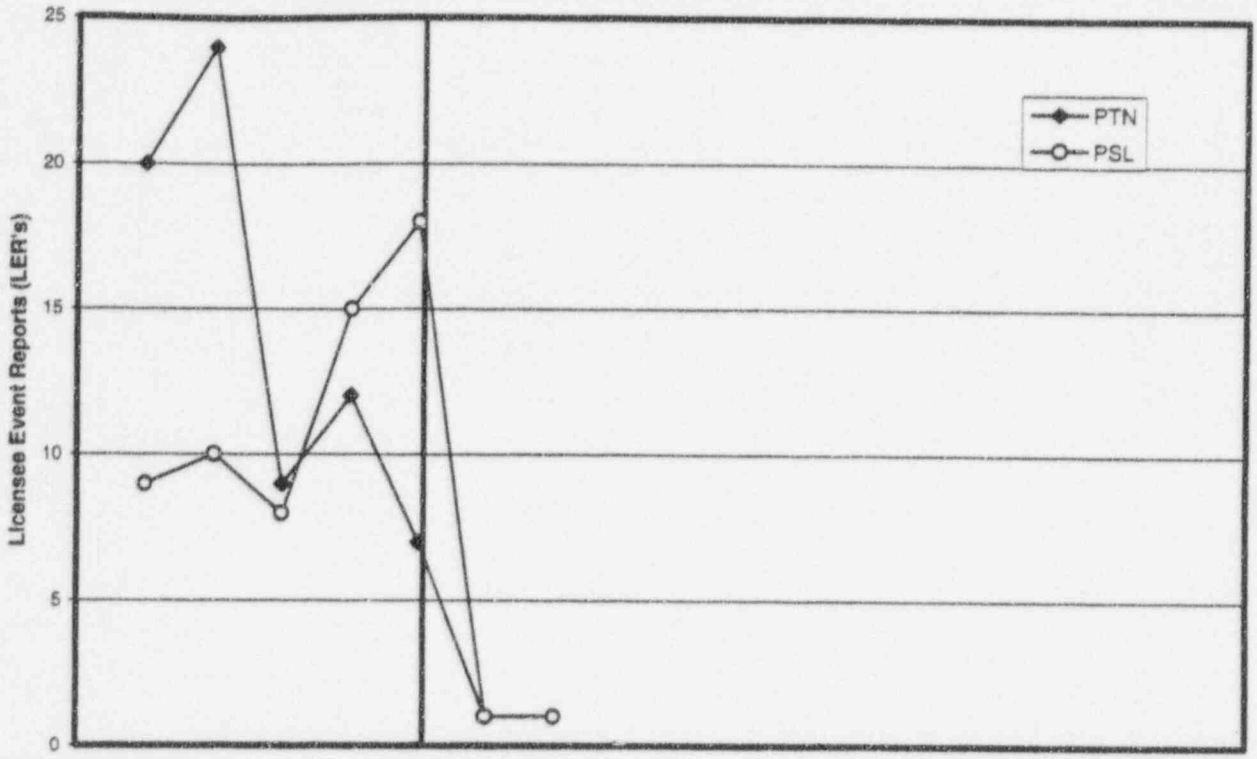
#96-03-02: Inadequate design control of RCS boron dilution procedure. Exit Meeting Date: 2/8/96.

#96-03-03: Inadequate 50.59 safety evaluation of change to boron dilution procedure. Exit Meeting Date: 2/8/96.

Data Providers: (PTN) Gary Hollinger 246-6078 and (PSL) Ed Weinkam 467-7162

LICENSEE EVENT REPORTS

(Year-to-Date)



Unit	1991	1992	1993	1994	1995	1/96	2/96	3/96	4/96	5/96	6/96	7/96	8/96	9/96	10/96	11/96	12/96
PTN	20	24	9	12	7	1	1										
PSL	9	10	8	15	18	1	1										

DEFINITION

License Event Reports (LER) are submitted to the NRC by the licensee to report unusual occurrences prescribed by 10CFR50.73.

STATISTICAL SUMMARY

	Feb	YTD
PTN	0	1
PSL	0	1

INDUSTRY PERFORMANCE

PERFORMANCE SUMMARY

Turkey Point reported one Voluntary Reportable Event as follows:

LER #96-001-00: "Intake Cooling Water System Flow Rates Found With Potential to be Less Than Required by Design Basis" dated March 1, 1996. The event occurred January 31, 1996.

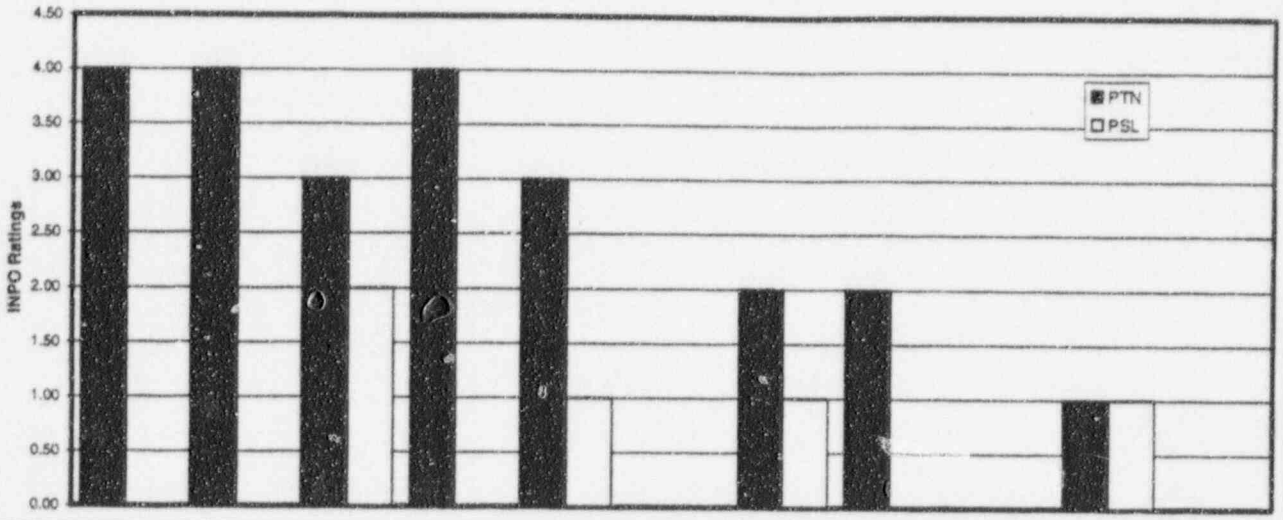
St. Lucie has reported one LER for the year:

LER #96-001-00: "Manual Reactor Trip due to High Main Generator Cold Gas Temperature" dated January 24, 1996. The event occurred January 5, 1996.

No LER's have been reported in February 1996.

Data Providers: (PTN) Gary Hollinger 246-6078 and (PSL) Ed Weinkam 467-7162

INPO ASSESSMENT RATINGS



Unit	1986	1987	1988	1989	1990	1991	1992	1993	1994	1995	1996
PTN	4.00	4.00	3.00	4.00	3.00	-	2.00	2.00	-	1.00	-
PSL	-	-	2.00	-	1.00	-	1.00	-	-	1.00	-

INPO ASSESSMENT PROGRAM DESCRIPTION

The Institute of Nuclear Power Operations (INPO) conducts periodic evaluations of site activities to make an overall determination of plant safety, to evaluate management systems and controls, and to identify areas needing improvement. Information is assembled from discussions, interviews, observations, and reviews of documentation.

Evaluation Frequency: INPO's goal is to visit each plant on an average of every 18 months. However, this frequency may vary depending upon the previous assessment ratings. For instance, if a plant is rated a "1" or "2", the interval between assessments is usually 20-24 months; if a plant is rated a "4" or a "5", the assessment interval is $\leq$ 18 months.

Performance Category Ratings:

Category 1: Overall performance is excellent. Industry standards of excellence are met in most areas. No significant weaknesses noted.
Category 2: Overall performance is exemplary. Industry standards of excellence are met in most areas. No significant weaknesses noted.
Category 3: Overall performance is generally in keeping with the high standards required in nuclear power. However, improvements are needed in a number of areas. A few significant weaknesses may exist.
Category 4: Overall performance is acceptable, but improvements are needed in a wide range of areas. Significant weaknesses are noted in several areas.
Category 5: Overall performance does not meet the industry standard of acceptable performance. The margin of nuclear safety is measurably reduced. Strong and immediate management action to correct deficiencies is required. Special attention, assistance, and follow-up are required.

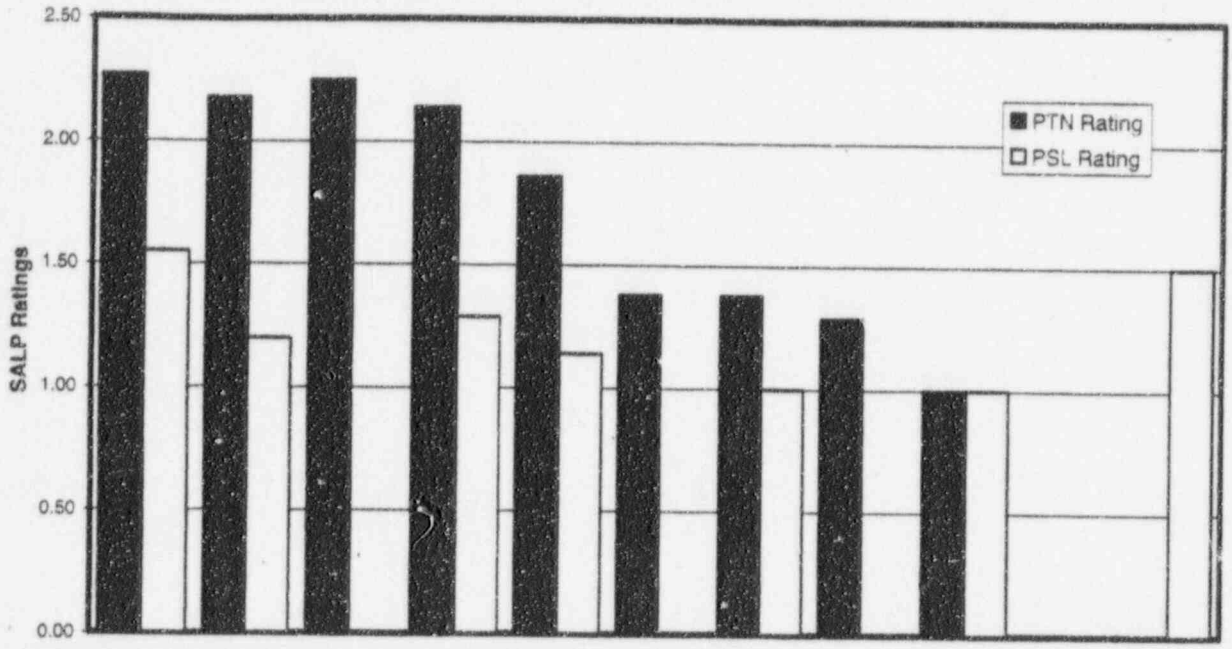
NOTE: If a plant is found to be operating without an adequate margin of nuclear safety, INPO will request that the plant be shutdown, or not started up.

PERFORMANCE SUMMARY

Turkey Point and St. Lucie received an INPO category rating of "1" in 1995. The next evaluations for Turkey Point and St. Lucie are expected in the last quarter of 1996 and first quarter of 1997, respectively.

Data Providers: (PTN) Gary Hollinger 246-6078 and (PSL) Ed Weinkam 467-7162

NRC SALP CATEGORY RATINGS



	1986	1987	1988	1989	1990	1991	1992	1993	1994	1995	1996
PTN Rating	2.27	2.18	2.25	2.14	1.86	1.38	1.38	1.29	1.00	-	-
Period Ending	4/30/86	5/31/87	6/30/88	7/31/89	7/31/90	9/28/91	-	1/30/93	8/27/94	-	-
PSL Rating	1.55	1.20	-	1.29	1.14	-	1.00	-	1.00	-	1.50
Period Ending	4/30/86	10/31/87	-	4/30/89	10/31/90	-	5/2/92	-	1/1/94	-	1/6/96

SALP PROGRAM DESCRIPTION

It is the policy of the NRC to use the Systematic Assessment of Licensee Performance (SALP) process to articulate the agency's observations and insights on licensee safety performance. The SALP report communicates those observations and insights.

Evaluation Frequency: The NRC will normally review and evaluate each power reactor licensee that possesses an operating license at least every 18 months. When the NRC determines that the performance warrants a more frequent evaluation, the normal SALP frequency may be increased. The assessment period may be extended to a maximum of 24 months when a plant receives a Category 1 rating in all four functional areas.

Functional Areas: Performance is generally evaluated in four (4) functional areas:

1. Plant Operations. This functional area consists chiefly of the control and execution of activities directly related to operating a plant. It includes activities such as plant startup, power operations, plant shutdown, and system lineups. It also includes initial and requalification training of licensed operators.
2. Maintenance. This functional area includes all activities associated with either diagnostic, predictive, preventive, or corrective maintenance of plant structures, systems, and components, or maintenance of the physical condition of the plant.
3. Engineering. This functional area addresses the adequacy of technical and engineering support for all plant activities. It includes all licensee activities associated with design control; the design, installation, and testing of plant modifications; engineering and technical support for operations, outages, maintenance, testing, surveillance, and procurement activities; configuration management; design-basis information and its retrieval; and support for licensing activities.
4. Plant Support. This functional area covers all activities related to plant support functions, including radiological controls, emergency preparedness, security, chemistry, and fire protection. Housekeeping controls are also included in this area.

Performance Category Ratings: Licensee performance in each functional area is assessed by assigning a category rating as discussed below:

Category 1. Licensee attention and involvement have been properly focused on safety and resulted in a superior level of safety performance.

Category 2. Licensee attention and involvement are normally well focused and resulted in a good level of safety performance.

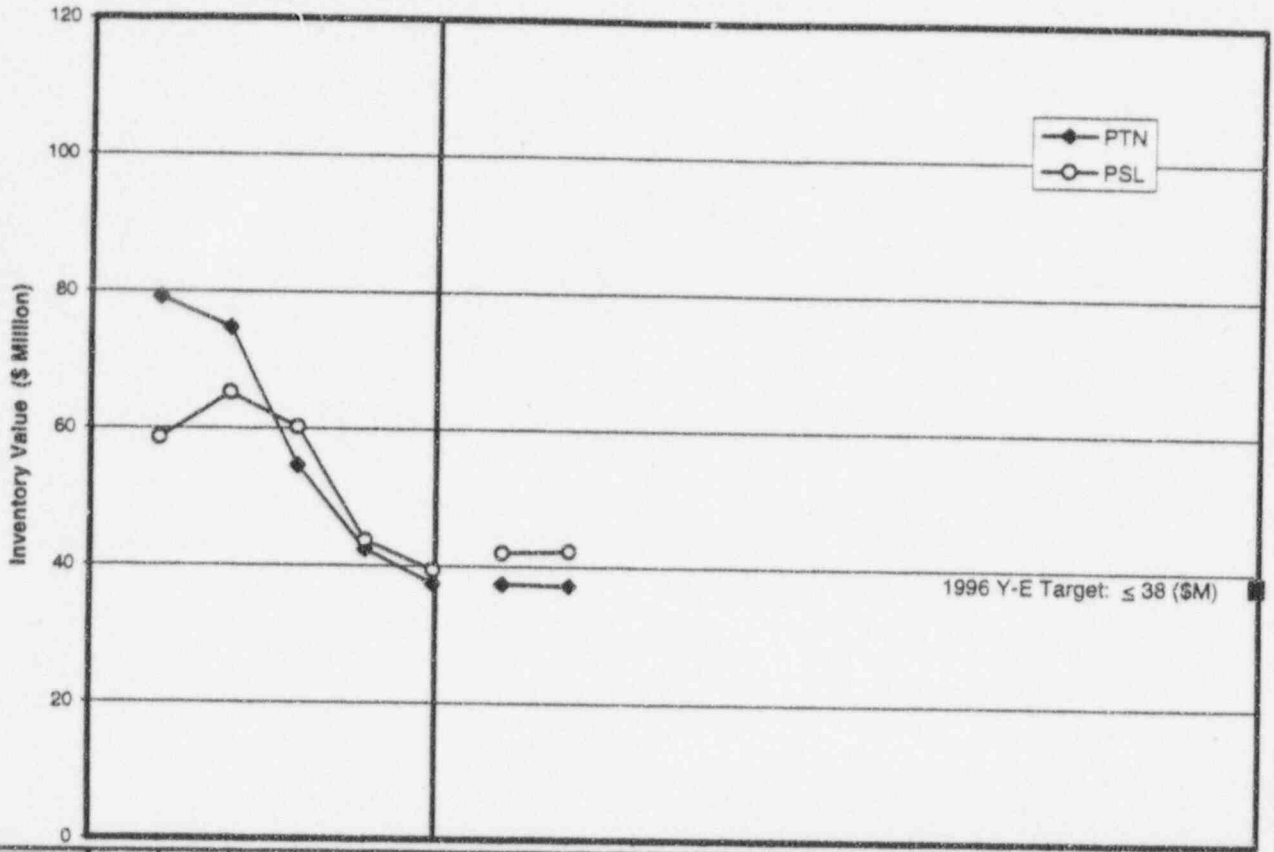
Category 3. Licensee attention and involvement have resulted in an acceptable level of performance. However, performance may exhibit one or more of the following characteristics: ineffective programs and significant issues, lack of corrective action thoroughness, and deficiencies in root cause analysis. Because the margin to unacceptable performance in important aspects is small, increased NRC and licensee attention is required.

PERFORMANCE SUMMARY

	Turkey Point		St. Lucie	
	Prior	Most Recent	Prior	Most Recent
Functional Area:				
Plant Operations	1	1	1	2
Maintenance	2 Improving	1	1	2
Engineering	2 Improving	1	1	1
Plant Support	n/a	1	1	1
Emergency Preparedness	1	n/a	n/a	n/a
Radiological Controls	1	n/a	n/a	n/a
Security	1	n/a	n/a	n/a
Self Assessment / Quality Verification	1	n/a	n/a	n/a
Overall:	1.29	1.00	1.00	1.50

Data Providers: (PTN) Gary Hollinger 246-6078 and (PSL) Ed Weinkam 467-7162

INVENTORY VALUE ACCOUNT 154.300



Unit	1991	1992	1993	1994	1995	1/96	2/96	3/96	4/96	5/96	6/96	7/96	8/96	9/96	10/96	11/96	12/96
PTN	79.1	74.7	54.6	42.4	37.4	37.4	37.2										
PSL	58.6	65.2	60.2	43.7	39.4	42.0	42.3										

DEFINITION

This indicator reflects the valve of Account 154.300. This account reflects materials needed to keep operational the physical equipment and facilities of the plant (e.g., spare parts, consumables, commodities, tools). The information is pulled from SAR Report #G0009R72-501. The PassPort system utilizes SAR for system data reporting.

STATISTICAL SUMMARY

		Start	End	%Change	YE Target
PTN	Monthly	37.4	37.2	-0.5%	
	Y-T-D	37.4	37.2	-0.5%	≤38M
PSL	Monthly	42.0	42.3	0.7%	
	Y-T-D	39.4	42.3	7.4%	≤38M

INDUSTRY PERFORMANCE

IBG (Year 1994)	*Total
Average	$43.6 Million
Top Quartile Entry	$34.7 Million
Top Quartile Average	$28.9 Million

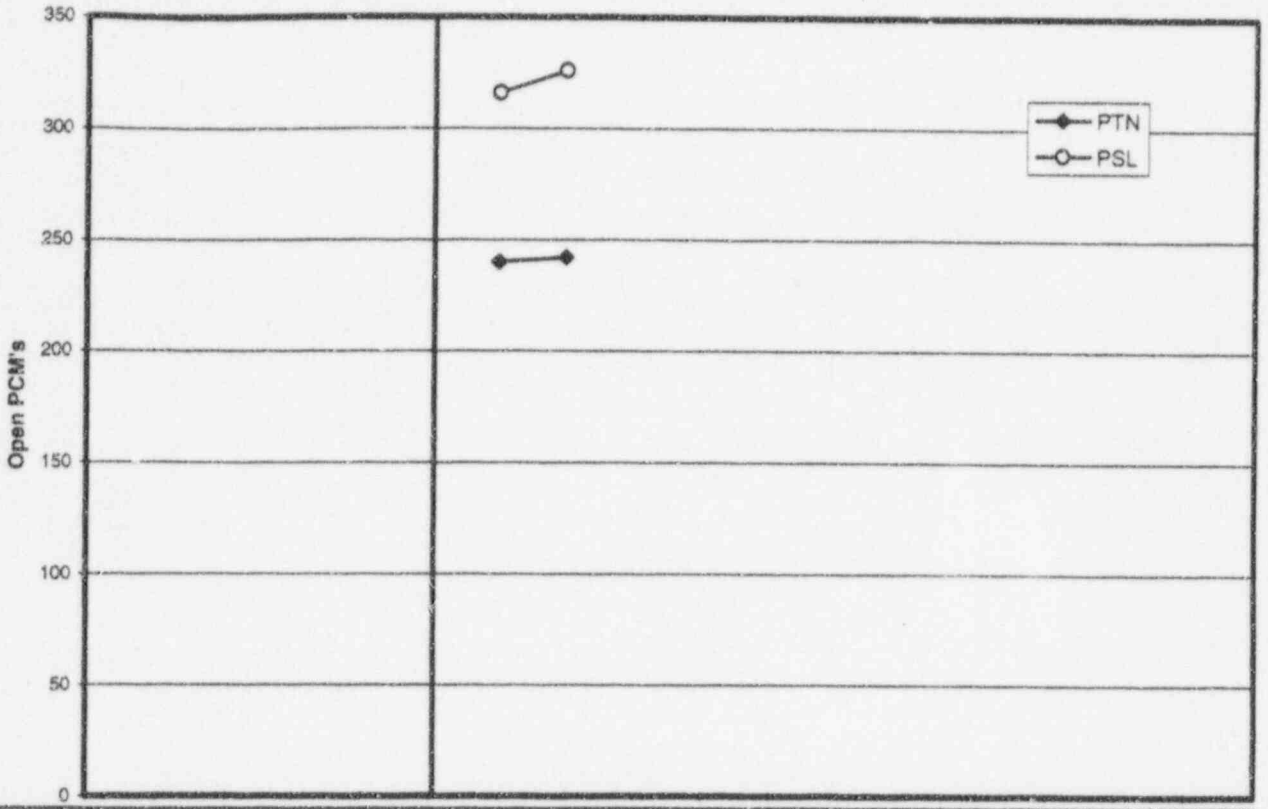
*Value does not include Capital as defined by PRUC

PERFORMANCE SUMMARY

- Turkey Point Regular Inventory decreased .2M in February.
- St. Lucie's Regular Inventory increased by .3M for the month.

Data Providers:
PSL Tom Kreinberg 465-4183
PTN Dick Rose 246-6692

OPEN PLANT CHANGES / MODIFICATIONS (PC/Ms)



Unit	1991	1992	1993	1994	1995	1/96	2/96	3/96	4/96	5/96	6/96	7/96	8/96	9/96	10/96	11/96	12/96
PTN						240	242										
PSL						316	326										

DEFINITION

This indicator tracks the total number of PC/M's in the engineering department. The purpose is to provide management a snapshot of the number of PC/M's. Includes all jobs that have been approved by plant management, and are in design, operability review, or implementation phases. The PC/M is considered open until System Acceptance Turnover Sheet (SATS) and drawing update is complete.

STATISTICAL SUMMARY

	Design	Review	Working	Update	Total
PTN					242
PSL	58	47	126	108	326

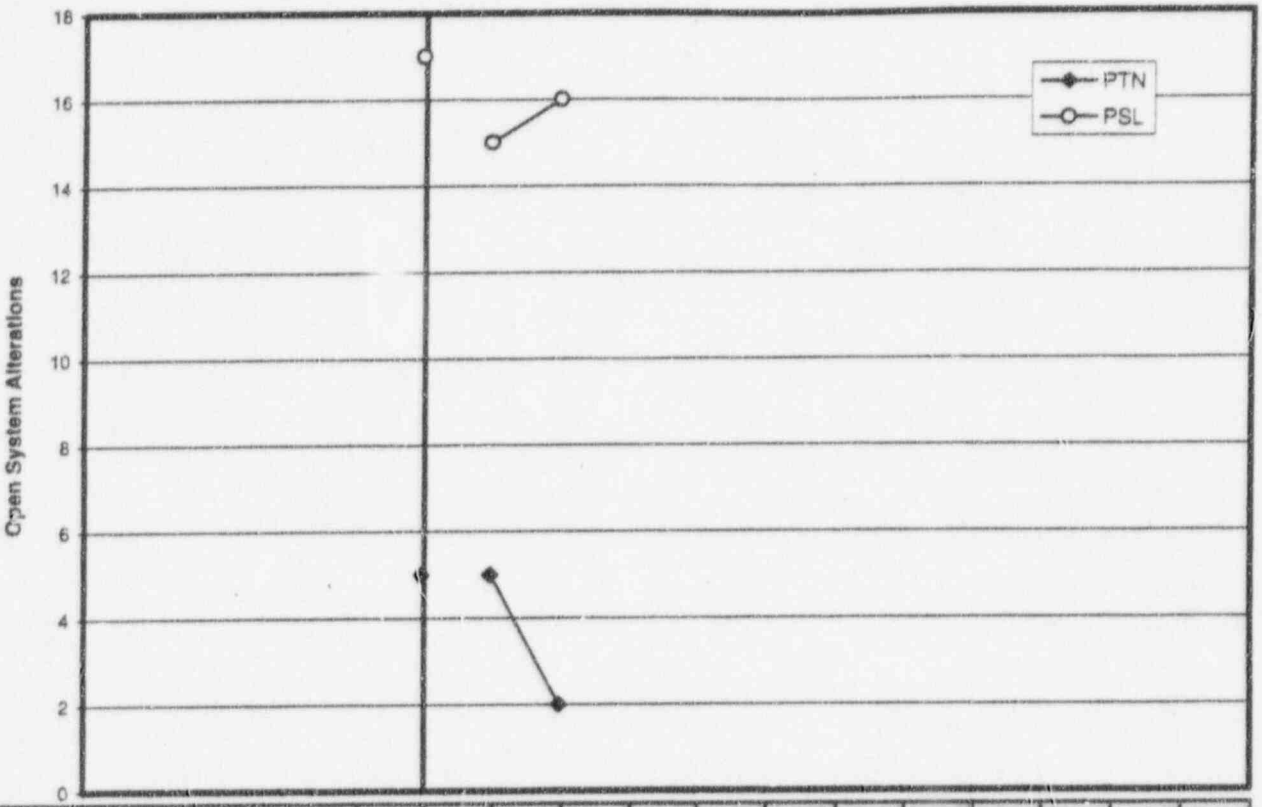
INDUSTRY PERFORMANCE

PERFORMANCE SUMMARY

St. Lucie has excluded the Steam Generator Replacement Project PC/M's which accounts for 20-30 PC/M's. St. Lucie is working on a Generic PC/M to convert the open ARP's and As Fail PC/M's; thus a future reduction is anticipated.

Data Providers:
PTN Jim Reed 246-6903
PSL Kris Mohindroo 467-7482

OPEN TEMPORARY SYSTEM ALTERATIONS



Unit	1991	1992	1993	1994	1995	1/96	2/96	3/96	4/96	5/96	6/96	7/96	8/96	9/96	10/96	11/96	12/96
PTN					5	5	2										
PSL					17	15	16										

DEFINITION

A temporary system alteration is a modification made to plant equipment, components, or systems that does not conform with approved drawings or other design documents; a modification that is necessary for continued safe plant operation; a modification that will remove a nuisance or distraction to the Plant Operators; a modification necessary to enable the plant to start up in a safe manner.

STATISTICAL SUMMARY

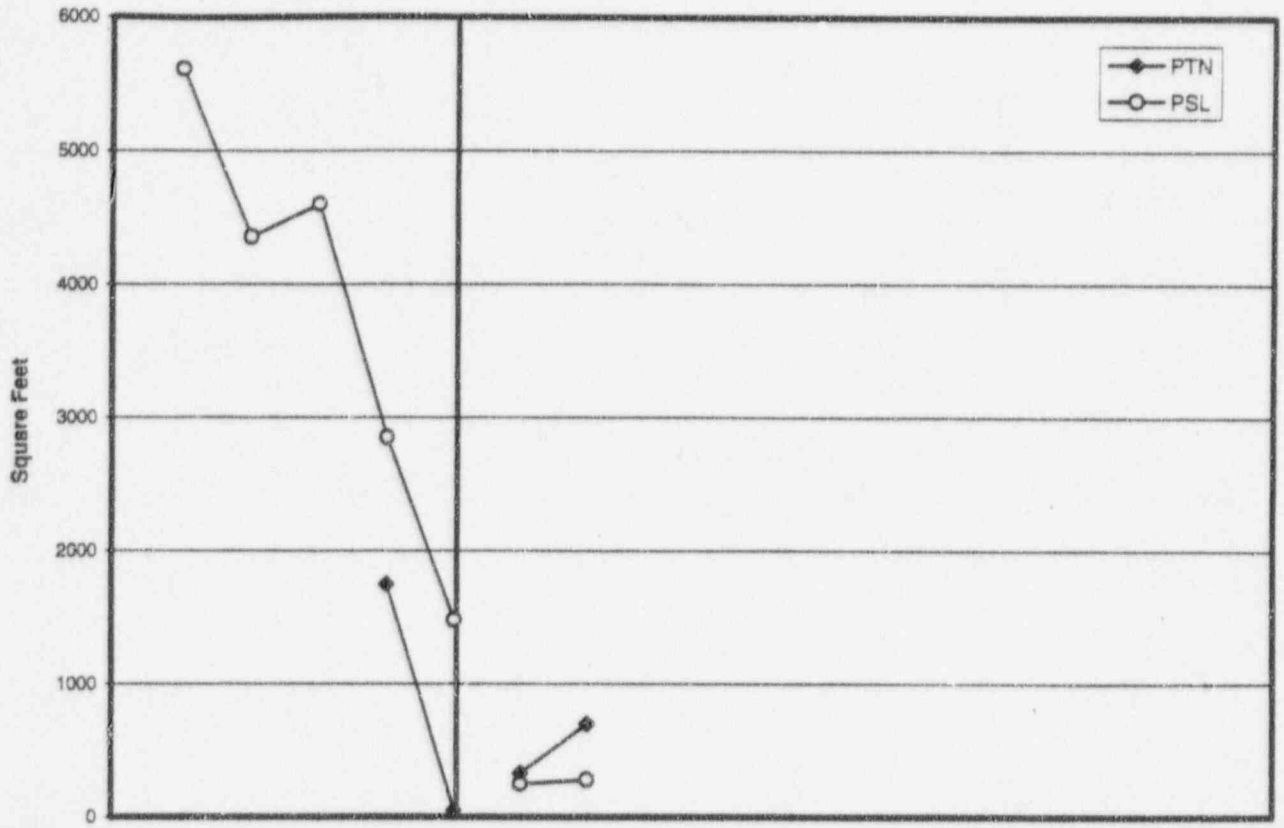
	Feb	Year End Goal
PTN	5	5
PSL	16	10

INDUSTRY PERFORMANCE

PERFORMANCE SUMMARY

Data Provider:
PSL Kris Mohindroo 467-7482
PTN Julio Balaguero 246-6971

CONTAMINATED FLOOR SPACE



Unit	1991	1992	1993	1994	1995	1/96	2/96	3/96	4/96	5/96	6/96	7/96	8/96	9/96	10/96	11/96	12/96
PTN				1750	50	330	700										
PSL	5610	4351	4600	2648	1480	250	280										

DEFINITION

This indicator, designed to measure contaminated floor space with removable activity ≥1000 dpm/100cm sq. beta/gamma or ≥ 20 dpm/100cm sq. alpha, is counted against the base. Areas that can be specifically exempted from the base include: reactor containment building, chemical volume control system demineralizer room and long term process areas such as the decontamination facility. Contaminated components such as charging pumps, evaporators, etc. are not included as part of "recoverable" floor space (i.e. not considered floor area you can walk or step on).

PTN: Total Base(117,746 sq. ft.) Exempted Area (6,110 sq. ft.)
PSL: Total Base(112,422 sq. ft.) Exempted Area (7,722 sq. ft.)

STATISTICAL SUMMARY

	Feb	Target
PTN	700	50
PSL	280	50

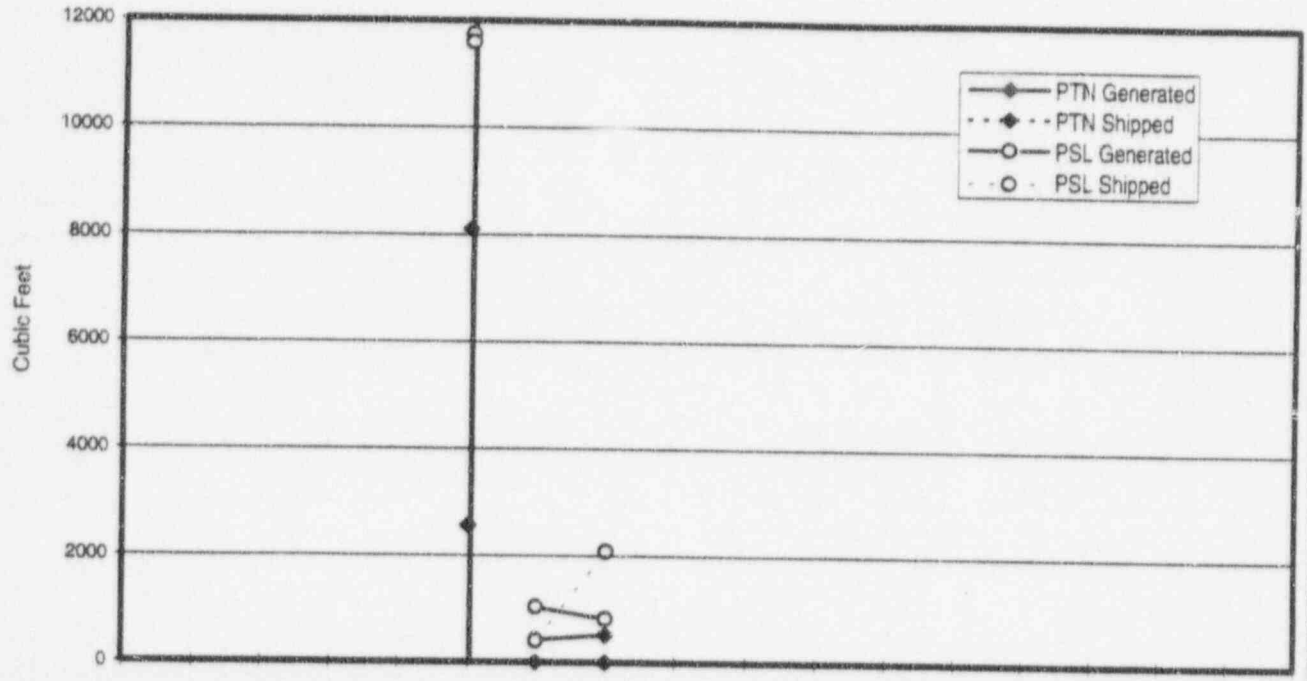
INDUSTRY PERFORMANCE

PERFORMANCE SUMMARY

Data Providers: (PTN) John Lindsay 246-6548 and (PSL) Hank Buchannon 467-7300

DRY ACTIVE WASTE: GENERATED, SHIPPED OFFSITE

(Year-to-Date)



Unit	1991	1992	1993	1994	1995	1/96	2/96	3/96	4/96	5/96	6/96	7/96	8/96	9/96	10/96	11/96	12/96
PTN Generated					8100	425	515										
PTN Shipped					2560	0	0										
PSL Generated					11740	1040	820										
PSL Shipped					11600	404	2080										

DEFINITION

Generated - Is an estimate based on the number of "yellow bags" initially generated prior to surveying for free release or shipment as radwaste.

Calculation: Number of yellow bags x 5 cubic feet = Estimated Monthly Generated Waste figure.

Shipped offsite - The amount of dry active radioactive waste that FPL ships to either Scientific Ecology Group Inc. (SEG) or American Ecology Recycle Center (AERC) for processing.

STATISTICAL SUMMARY

PSL made 6 shipment to SEG in 1995.

PTN made 3 shipments to SEG in 1995.

INDUSTRY PERFORMANCE

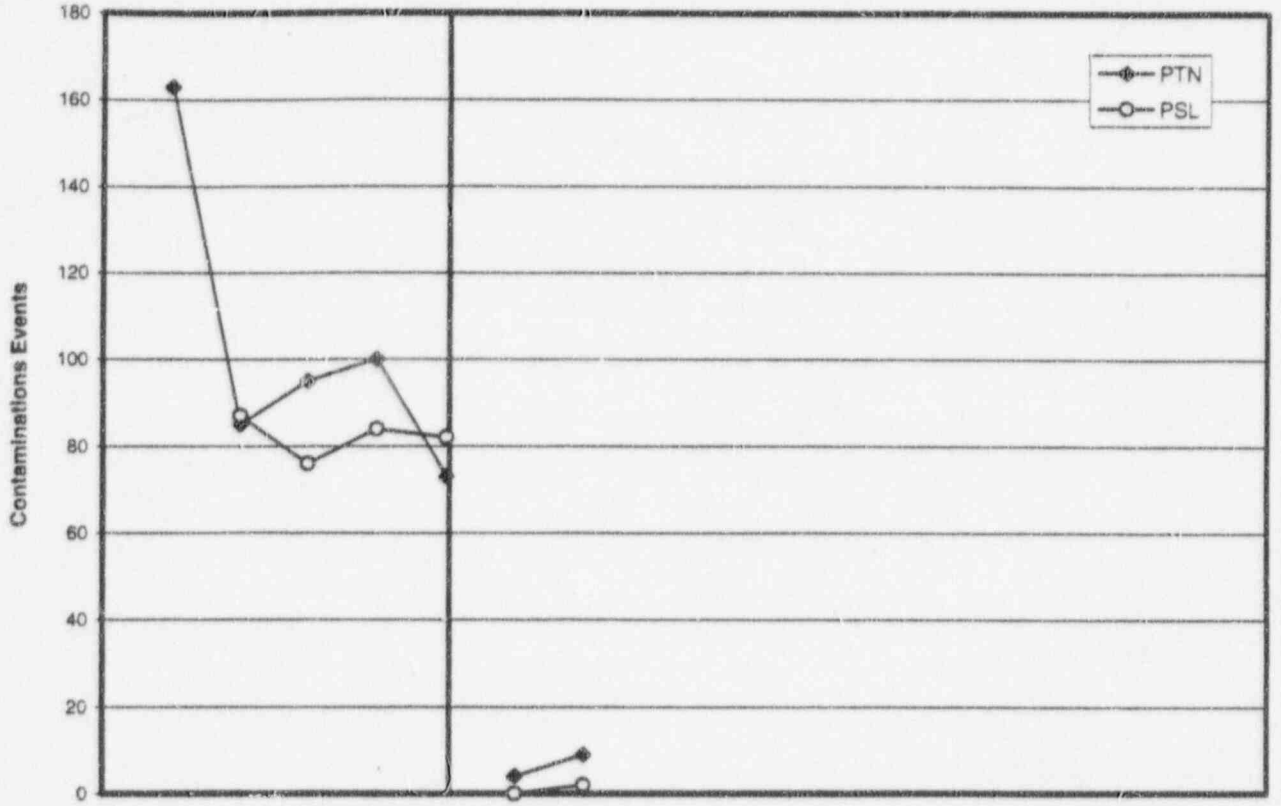
PERFORMANCE SUMMARY

- PTN projects to ship 2 Sealand Containers to SEG in 1996.
- PSL projects to ship 4 Sealand Containers to SEG in 1996.

Data Providers: (PTN) John Lindsay 246-6548 and (PSL) Hank Buchannon 467-7300

PERSONNEL CONTAMINATION EVENTS

(Year-to-Date)



Unit	1991	1992	1993	1994	1995	1/96	2/96	3/96	4/96	5/96	6/96	7/96	8/96	9/96	10/96	11/96	12/96
PTN	163	85	95	100	73	4	9										
PSL		87	76	84	82	0	2										

DEFINITION

This indicator is designed to monitor personnel contamination. A personnel contamination exists when 5000 dpm per 100cm2 on skin or personal clothing as detected by Personal Contamination Monitor and ≥100 counts per minute (net) using the Frisker is observed.

STATISTICAL SUMMARY

	Feb	Y-T-D
PTN	5	9
PSL	2	2

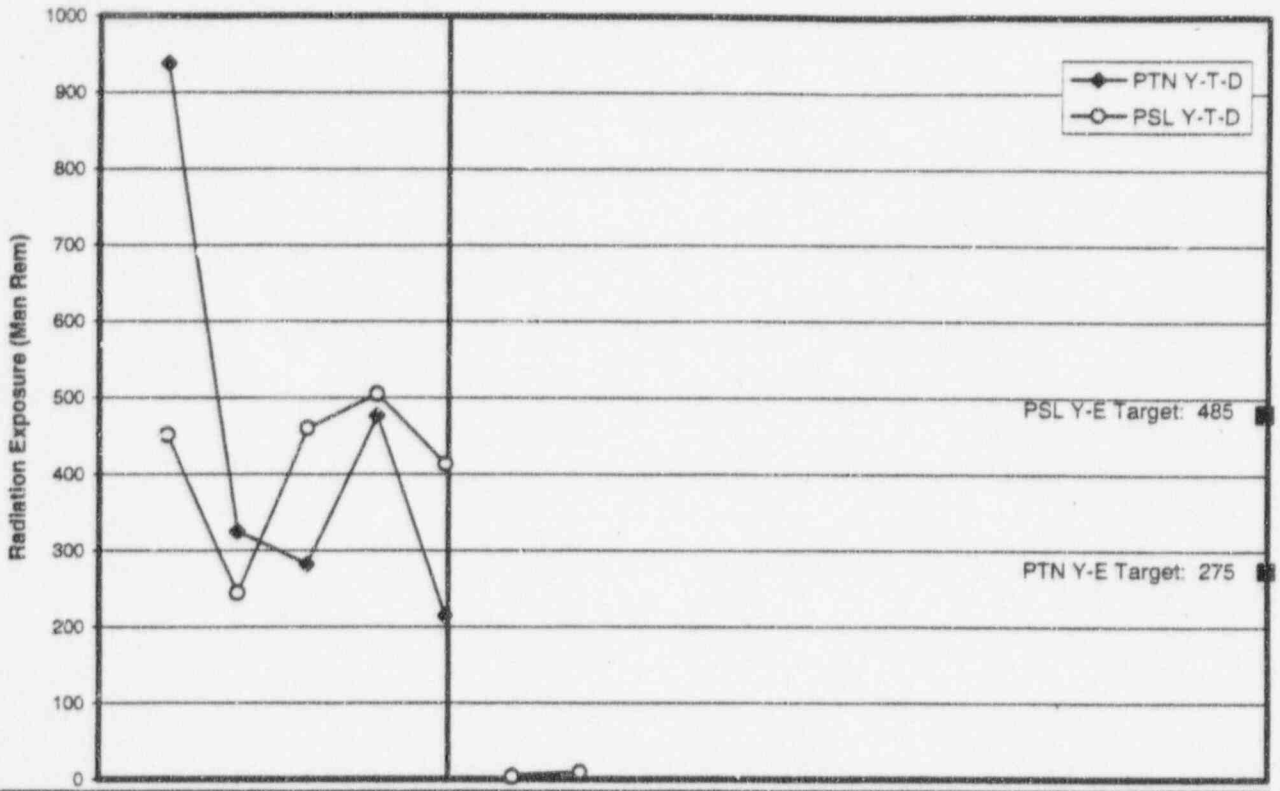
INDUSTRY PERFORMANCE

PERFORMANCE SUMMARY

Data Providers: (PTN) John Lindsay 246-6548 and (PSL) Hank Buchannon 467-7300

RADIATION EXPOSURE

(Year-to-Date)



Unit	1991	1992	1993	1994	1995	1/96	2/96	3/96	4/96	5/96	6/96	7/96	8/96	9/96	10/96	11/96	12/96
PTN Y-T-D	938.0	324.9	281.8	476.2	214.6	5.3	9.5										
PSL Y-T-D	451.3	244.5	459.9	504.7	412.8	3.0	9.0										

DEFINITION

Collective Radiation Exposure is the total effective dose equivalent received by all on-site personnel (including contractors and visitors), it includes external deep dose as measured by the thermoluminescent dosimeters(TLD's) plus internal dose. It is reported in man-rem for the station. Current month readings may be taken from the direct reading dosimeters (DRD's).

STATISTICAL SUMMARY

	Feb	YTD	YTD Target	12-Mo. Ending
PTN	4.2	9.5	10.0	218.8
PSL	6.0	9.0	25.0	397.6

Year-End Targets:
PTN: 275.0
PSL : 485.0

INDUSTRY PERFORMANCE

WANO (PWR's)	Man-Rem (two units)
1995 Median (7/92 - 6/95)	270
1995 Goal (12-Mo. ending 6/95)	370

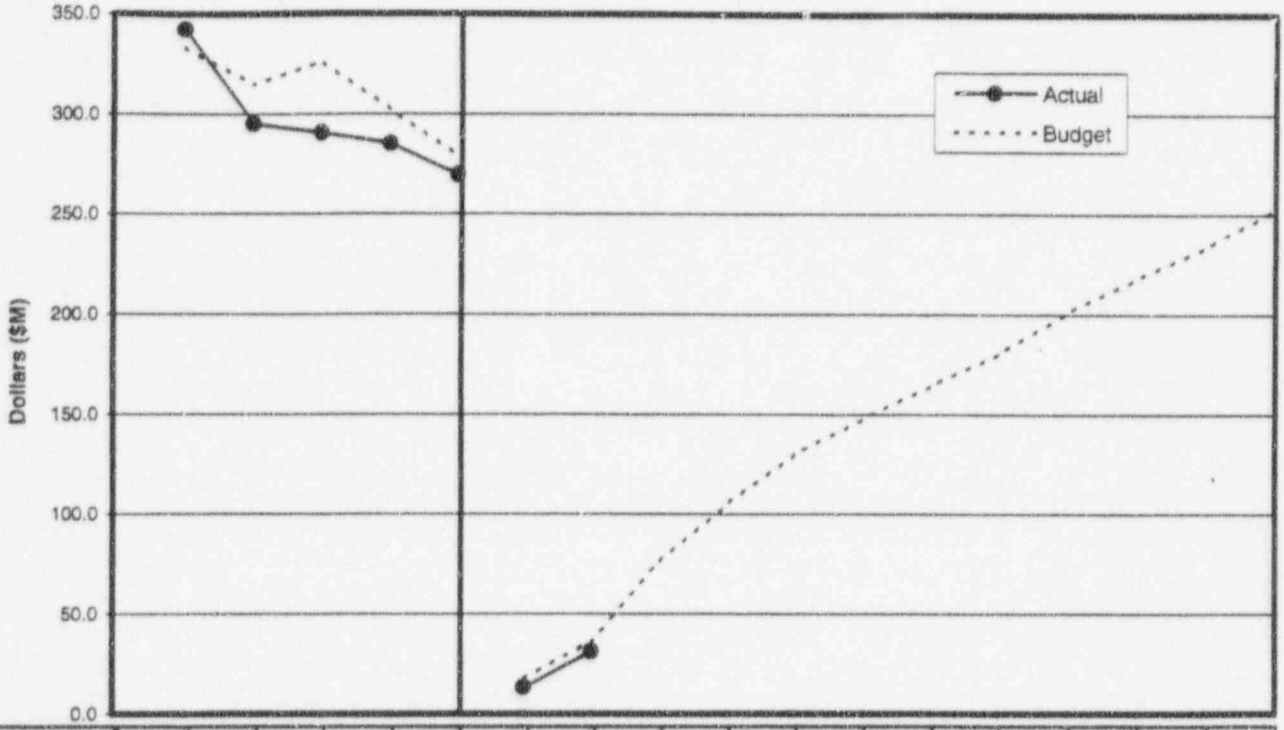
PERFORMANCE SUMMARY

- Turkey Point's 12-Mo. ending of 218.8 was below industry median in Februa y.
- St. Lucie's Man-Rem 12-mo. ending totaled 397.6 which is higher than the 12-mo. ending industry median.

Data Providers: (PTN) John Lindsay 246-6548 and (PSL) Hank Buchannon 467-7300

O&M BUDGET - DIVISION

(Year-to-Date)



	1991	1992	1993	1994	1995	1/96	2/96	3/96	4/96	5/96	6/96	7/96	8/96	9/96	10/96	11/96	12/96
Actual	342.4	295.2	290.6	285.2	269.5	13.4	31.2										
Budget	333.3	314.6	326.4	302.0	279.2	17.8	36.7	75.8	104.8	129.6	147.4	164.0	180.0	201.2	216.0	233.9	252.8
Variance (%)	2.7	-6.2	-11.0	-5.6	-3.5	-24.9	-14.9										

DEFINITION

Operating and Maintenance Expenditures include Nuclear Division operation and maintenance expenses associated with direct employees, contractors and consultants, equipment, tools, design, engineering and other items/activities required to sustain the electrical generation of the plants and to provide required support. Fuel costs, corporate administrative and general expenses, and charges from other departments outside the Nuclear Division are excluded.

$$\frac{\text{Y-T-D Actual Expenses - Y-T-D Budgeted Expenses}}{\text{Y-T-D Budgeted Expenses}} \times 100\% = \text{O\&M Variance \%}$$

STATISTICAL SUMMARY

	Y-T-D Actual ($M)	Y-T-D Variance ($M)	%
Jan '96	31.2 ($M)	-5.5 ($M)	-14.9%
Jan '95	27.8 ($M)	-7.7 ($M)	-21.8%

1996 Y-E Budget: 252.8 ($M)

INDUSTRY PERFORMANCE

	O&M Budget
IBG Actual 1994 Avg.	$297.0M
IBG Top Quartile Entry	$270.0M
IBG Top Quartile Avg.	$213.2M
IBG Projected 1995 Avg.	$284.0M

Projection (per dual unit site) derived by trending 1986-94 Actual data and 1995 Budgeted data for IBG Group.

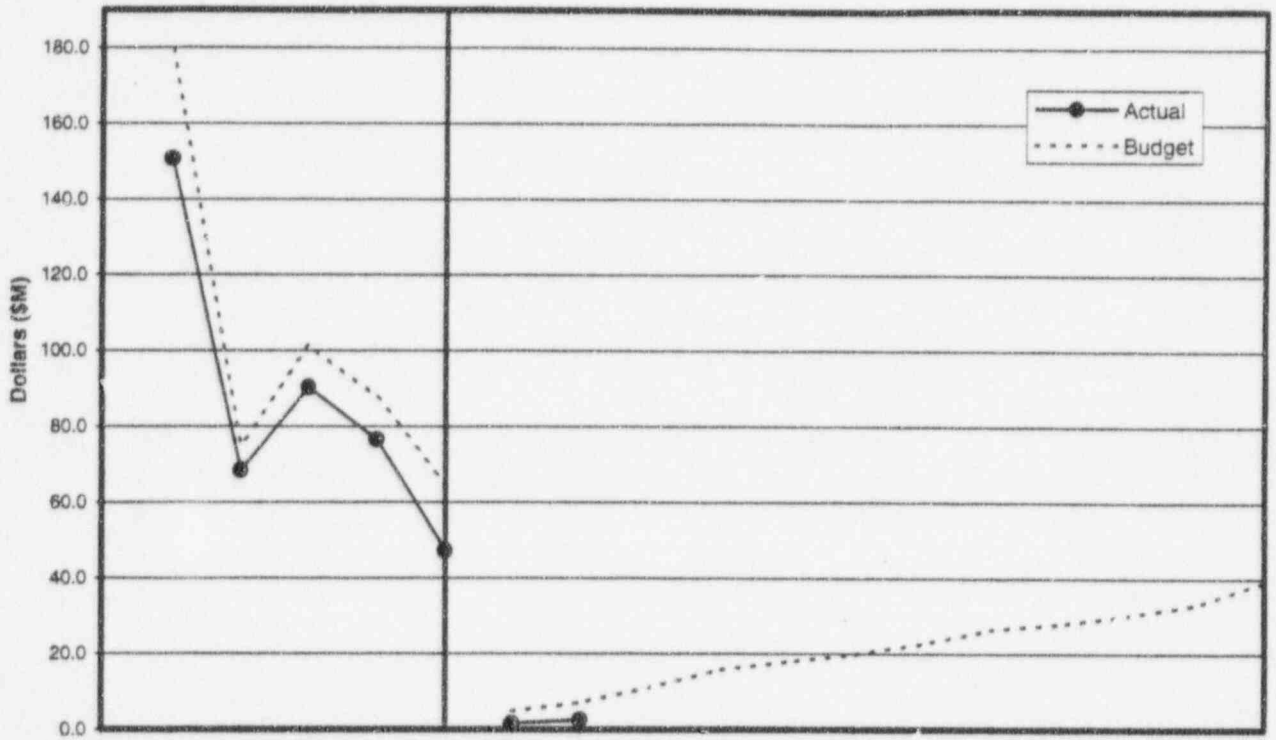
PERFORMANCE SUMMARY

O&M Expenditures in February 1996 were $31.2 million which represented a budget underrun of $5.5 million (or 14.9%). The variance was the result of: underrun primarily due to St. Lucie non-outage project underruns; and underruns in materials and contracted services at Turkey Point due outage cash flow revisions.

Data provider: T. O. Nasby 694-4188

CAPITAL BUDGET - DIVISION

(Year-to-Date)



	1991	1992	1993	1994	1995	1/96	2/96	3/96	4/96	5/96	6/96	7/96	8/96	9/96	10/96	11/96	12/96
Actual	150.7	68.4	90.3	76.5	47.2	1.5	2.5										
Budget	179.6	75.1	101.4	87.7	64.7	4.8	7.1	11.1	15.7	17.9	19.8	22.6	26.3	27.8	30.1	33.1	39.2
Variance (%)	-16.1	-9.0	-11.0	-12.7	-27.0	-67.6	-65.0										

DEFINITION

Capital Expenditures are those directly incurred/budgeted by the Nuclear Division for the construction of new utility plant additions and improvements made to increase efficiency, reliability or safety. Capital fuel costs are excluded.

$$\frac{\text{Y-T-D Actual Expenses - Y-T-D Budgeted Expenses}}{\text{Y-T-D Budgeted Expenses}} \times 100\% = \text{Capital Variance \%}$$

STATISTICAL SUMMARY

	Y-T-D Actual ($M)	Y-T-D Variance ($M)	%
Jan '96	2.5 ($M)	-4.6 ($M)	-65.0%
Jan '95	3.6 ($M)	-3.7 ($M)	-50.4%

1996 Y-E Budget: 39.2 ($M)

INDUSTRY PERFORMANCE

	Capital Budget
IBG Actual 1994 Avg.	$63.1M
IBG Top Quartile Entry	$28.3M
IBG Top Quartile Avg.	$21.1M
IBG Projected 1995 Avg.	$58.5M

Projection (per dual unit site) derived by trending 1986-94 Actual data and 1995 Budgeted data for IBG Group.

PERFORMANCE SUMMARY

Capital Expenditures in February 1996 were $2.5 million which represented a budget underrun of $4.6 million (or 65.0%). The variance was primarily due to: underrun in the Steam Generator Replacement Project (SGRP) due to a change in the outage start date; and during year cash flow revisions for various St. Lucie plant projects.

Data provider: T. O. Nasby 694-4188

INTERNAL DISTRIBUTION:

R.	J.	Acosta	JNA/JB
T.	V.	Abbatiello	PTN/PLT
K.	E.	Beatty	JNO/JB
L.	W.	Bladow	PSL/PLT
W.	H.	Bohlke	PSL/PLT
G.	J.	Boissy	JNO/JB
J.	B.	Brady	CC/JB
J.	L.	Broadhead	JEX/JB
C.	L.	Burton	PSL/PLT
R.	L.	Castro	JDC/JB
J.		Clay	PSL/PLT
K.	R.	Craig	JPN/JB
J.	L.	Danek	JNS/JB
D.	J.	Denver	PSL/JB
R.	E.	Dawson	PSL/PLT
M.	S.	Dryden	JNL/JB
P.	L.	Fincher	PSL/PLT
J.	C.	Hampp	ETS/JB
J.	R.	Hartzog	PTN/PLT
R.	G.	Heisterman	PTN/PLT
G.		Hollinger	PTN/PLT
R.	J.	Hovey	PTN/PLT
D.	E.	Jernigan	PTN/PLT
H.	H.	Johnson	PTN/PLT
V.	A.	Kaminskas	PTN/PLT
T.		Kreinberg	PSL/PLT
H.	N.	Paduano	JPN/JB
J.	G.	Pizzutelli	JNE/JB
T.	F.	Plunkett	JEX/JB
T.	E.	Roberts	JPN/JB
R.	E.	Rose	PTN/PLT
D.	A.	Sager	JNA/JB
J.		Scarola	PSL/PLT
C.	H.	Shotwell	JHR/JB
R.		Sipos	PSL/PLT
D.	L.	Smith	JPN/JB
R.	A.	Symes	JNA/JB
C.		Villard	JPN/JB
W.	E.	Walker	PSL/PLT
E.		Weinkam	PSL/PLT
D.	H.	West	PSL/PLT
J.	A.	West	PSL/PLT
R.		West	FTN/PLT

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Kerry D. Landis, Region II
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PMAI Form Version: 0

Snyder

PMAI Corrective Action Form

PMAI Site: PSL

Source Number: PM96-05-023 Document: MAINT RULE

Originator Dept: SCE

Due Date: 06/15/96

Assigned Department: ENG

ENG/DENVER
Implementor Name

0
Unit Outage Mode SNO NCR OWA

Title: MAINT RULE

Description: DEVELOP A PROGRAM FOR MONITORING AND TRENDING OF PSL STUCTURES IN ACCORDANCE WITH NUREG 1526 AND VEI DRAFT GUIDELINES. EXAMPLES AND REFERENCES ARE ATTACHED.

Due Date Extension/Transfer Responsibility Requests

Extend To Date	Request by /Date	Appvd By: Implementing Dept. Supervisor /Date	Originating Dept. /Date	Implementing Dept. Manager /Date
7/10/96	E. Hollowell 6-3-96	MH Snyder SCE 6/4/96	MH Snyder SCE 6/4/96	for CH Wood SCE 6/4/96

Reasons: _____

XFER Resp. To	Request by /Date	Appvd By: Implementing Dept. Supervisor /Date	Implementing Dept. Manager /Date	Recipient Dept. /Date
SCE/M. Snyder	A-Falta 5-28-96	5-28-96	K.K. Mohindra 5/29/96	5/28/96

Reasons: _____

Completion Section

"SCOPING DOCUMENT FOR THE IMPLEMENTATION OF THE MAINTENANCE RULE FOR MONITORING THE EFFECTIVENESS OF STRUCTURES"

Completion Date: _____ Close-Out Documents: ADM 17.08 R/4, ADM 08-02 R/12, AP0005 750 R/4, Φ-DI-99-02 R/1, SCE-L6 R/0

Comments:

THE CLOSEOUT DOCUMENTS PROVIDE THE INSTRUCTIONS/GUIDELINES/AND REQUIREMENTS FOR THE PROGRAM FOR IMPLEMENTATION OF "THE MAINTENANCE RULE" FOR STRUCTURES ACCORDANCE WITH 10 CFR 50.65.

ATTACHMENT 2 - SCOPING DOCUMENT FOR THE IMPLEMENTATION OF THE MAINTENANCE RULE FOR MONITORING THE EFFECTIVENESS OF STRUCTURES

ATTACHMENT 3 - ADM 17.08 R/4 ATTACHMENT 4 - SCEG-D3 R/0

Implementor: Edward Hollowell (Signature) / EDWARD HOLLOWELL (Print) Date: _____

Reviewed By: Implementing Supv. (Signature) / M Snyder SCE (Print) Date: 7/10/96

Approved By: Implementing Dept. Mgr. (Signature) / CH Wood / SCE (Print) Date: 7/10/96

Reviewed By: Originating Dept. (Signature) / M Snyder SCE (Print) Date: 7/10/96

BBB/68

PMAI 96-05-023
ATTACHMENT 1
36 PAGES
(INCLUDING THIS SHEET)

System 73 ~~**Pipes, Supports, and Structures**~~ DRAFT R/N N a1/a2 a2

Function

1. This system captures all the safety-related structures as listed in the FSAR that were not scoped as part of other Maintenance Rule systems. (See Table 3.2-1)
2. This system also captures the safety related piping as listed in the FSAR. The valves are scoped under the original system.
3. Safety related supports and snubbers are scoped in this system.

Key Components

Unit 1 Structures

Containment Shield Bldg.
Reactor Auxiliary Building
Diesel Generator Building
Fuel Handling Building
Reactor Building Interior Structures

Unit 2 Structures

Containment Shield Bldg
Reactor Auxiliary Building
Diesel Generator Building
Fuel Handling Building
Reactor Building Interior Structures
Component Cooling Area Structure
Diesel Oil Storage Tank Building
Condensate Storage Tank Building
Steel Missile Barriers
Spent Fuel Pool and Liner
Spent and New Fuel Storage Racks
Main Steam Trestle
Biological Shielding within RAB and FHB
Roofs of Safety Related Structures
Spent Fuel Cask

Duplicate for U1

Containment Vessel

Containment Vessel

~~Systems with Safety~~ Related Piping (Both Units)

Reactor Coolant
Shutdown Cooling
Containment Spray
Containment Cooling
(Intake) Cooling Water
Auxiliary Feedwater
Shield Building Ventilation
[Iodine Removal]
Safety Injection
Chemical Volume and Control
Waste Management
Component Cooling Water
Main Steam and Feedwater
Emergency Power
Ventilation Systems

~~The snubbers~~ are listed in the Seismic Snubber List, 8770-B-122 [2998-B-122].

DRAFT

Programs in Place

All SSCs designated Quality Group A, B or C are covered by ASME Boiler and Pressure Vessel Code requirements for testing and inspections. Class I, II, and III piping is hydrostatically tested three times every ten years. Pipes > 4" have their supports inspected and their welds examined (NDE).

All SSCs designated Quality Group A, B or C or seismic Category I are designed, constructed and operated under the provisions of the Quality Assurance program requirements of Chapter 17 of the FSAR consistent with their effect on safety.

QI 10-PR/PSL-6, Control, Inspection and Monitoring of Mechanical and Hydraulic Shock Arrestors (Snubbers) and OSP-73.01, Guidelines for the Implementation of the Snubber Inspection and Test Program, ensure testing and examinations are performed in accordance with plant Technical Specifications and ASME Section XI.

Relative Risk Significance

~~This system is not risk significant.~~

risk significant.

The Shield Building and Containment Vessel are ~~scoped under system 68 - Containment Structure.~~ The piping for the Containment Isolation System and the Sampling System containment penetrations are tested as part of Appendix J and are also included in this system.

The Combustible Gas Control is scoped under 27 - Hydrogen Sampling/Recombiner.

Shield Building Ventilation is not subject to ASME testing. The integrity of the system is tested during safeguards and a failure would be counted in this system.

Emergency Power, Reactor Coolant Gas Vent, Fuel Pooling Cooling and Radiation Monitoring are all scoped under their main systems.

The Intake Structure is scoped under system 21B - Circulating Water.

Performance Criteria

Repetitive Functional Failures < 1 per two operating cycles.

The definition of a failure is established in accordance with ASME Boiler and Pressure Vessel Code and ASME Section XI requirements.

Failures of components not included in ASME, eg, seismic components are tracked and trended through the STAR program.

NUREG–1526

Lessons Learned from Early Implementation of The Maintenance Rule at Nine Nuclear Power Plants

U.S. Nuclear Regulatory Commission

Office of Nuclear Reactor Regulation

C. D. Petrone, R. P. Correia, S.C. Black

was being monitored at the system level rather than the train level, which could allow unreliable components to go undetected.

Trending of zero failures. At many of the sites visited, licensees had established zero MPFFs or 100-percent reliability as a goal or performance criterion for many of the SSCs under the scope of the rule. The rule intends that licensees be afforded maximum flexibility in establishing goals and performance criteria. However, the rule also intends that where failures are likely to cause loss of an intended function, monitoring should be predictive, giving early warning of degradation. The team was concerned that it would be difficult to use trending to help predict or anticipate failures when failure data is the only information being monitored.

Conclusions

Coordination of trending and goals. Most licensees had established trending programs. Trending was not required for all SSCs under the rule although it should be considered. The trending being performed by most licensees was not well coordinated and integrated with the goals and performance criteria established under the rule.

Monitoring of standby systems or systems with redundant trains. Certain non–risk-significant systems used in standby service were being monitored at the plant level rather than at the system or train level as required.

Trending of zero failures. Reliance on the use of zero MPFFs or 100-percent reliability as a goal or performance criterion may preclude predictive trending.

Recommendations

Coordination of trending and goals. Coordinate and integrate goals and performance criteria with equipment trending wherever possible.

Monitoring of standby systems or systems with redundant trains. Ensure that one train does not mask the poor performance of a redundant or standby train. Accomplish this task by monitoring at the following levels.

1. Monitor single train risk-significant systems at the system level.
2. Monitor multiple train risk-significant systems at the train level.
3. Monitor single train non–risk-significant systems used in standby at the system level.
4. Monitor multiple train non–risk-significant systems that are used in standby service at the train level.
5. Monitor normally operating, non–risk-significant systems at the plant level.

Trending of zero failures. Where reliance on the use of zero MPFFs or 100-percent reliability as a goal or performance criterion may preclude predictive trending, consider establishing additional goals and performance criteria that can be trended.

2.4.6 Monitoring and Trending of Structures

The rule requires that the performance or condition of structures be monitored in a manner sufficient to give reasonable assurance that those structures are capable of fulfilling their intended function. The statements of consideration for the rule states "[w]here failures are likely to cause loss of an intended function, monitoring should be predictive in nature, providing early warning of degradation." NUMARC 93-01, paragraph 9.4.2.4 lists examples of structural monitoring activities including nondestructive

examination, visual inspection, vibration analysis, and measurement of deflection.

Findings

The team reviewed the monitoring and trending of structures at the nine pilot sites and found that most licensees considered monitoring of structures under the rule to be a low priority. Some licensees had not established goals or performance criteria for monitoring most structures at their sites. Many licensees considered most structures to be inherently reliable. Some licensees believed that as long as a structure such as a building did not fall down and damage the equipment inside, the structure itself need not be monitored. At some sites, the onsite personnel were not aware of existing preventive maintenance and monitoring activities that were being performed on these structures by offsite structural or civil engineering groups.

One licensee took the position that their structures had performed acceptably for the past 20 years, not causing a loss of function of the systems contained in or supported by the these structures, and were not expected to begin a more rapid degradation from aging in the future. Therefore, believing these structures "very reliable," they found it unnecessary to establish goals or performance criteria to monitor them. However, they also stated that inspection and maintenance is necessary to ensure degradation of these structures does not cause a loss of function. The structures are monitored during the normal operator rounds, management walkaround inspections, and inspection by other plant departments in the course of normal work activities. Deficiency cards and maintenance work orders are generated when conditions adverse to quality are found. The team believes that the existence of these longstanding monitoring activities contradicts the licensee's position that no monitoring is needed. The team believes that the licensee should establish performance criteria and goals under the rule which take credit for the existing monitoring activities and build upon them.

One licensee had established performance criteria for most structures under the scope of the rule. However, the performance criteria for many of these structures was that the structure would not degrade to the point where it caused a loss of function of systems contained in the structure or supported by it. For example, the roof of a building that was leaking would meet the performance criteria until the water leaking into the building caused the system inside the building to fail. However, such performance criteria are not acceptable because they are not predictive and do not give early warning of degradation. The team believes that a more appropriate performance criterion would have been "no water leaks."

Another licensee had determined that all structures within scope of the rule, except the primary containment, were inherently reliable and therefore did not require goal setting under paragraph (a)(1) of the rule or monitoring against performance criteria under paragraph (a)(2) of the rule. The licensee's representative stated that these structures are routinely examined by plant personnel during their walkdown inspections of the plant. They believe that this monitoring activity is sufficient to verify that preventive maintenance is adequate. The team believes that although condition-monitoring is an appropriate method of monitoring structures, the lack of specific criteria to monitor against would make it difficult to detect degradation of these structures.

Conclusions

Most licensees considered the monitoring of structures under the rule to be a low priority. Some licensees incorrectly assumed that many

of their structures are inherently reliable. The performance criteria for monitoring the performance or condition of some structures are not predictive and do not give early warning of degradation.

Certain structures such as the primary containment can be monitored by fulfilling established testing requirements such as those in 10 CFR Part 50, Appendix J. However, other structures such as reactor buildings, auxiliary buildings, and cooling towers may be more amenable to condition-monitoring. Some licensees are developing programs for monitoring structures that will include doing plant walkdown inspections and engineering evaluations to establish condition-monitoring criteria. This program should include the establishment of specific criteria for monitoring.

Recommendations

1. Reevaluate the monitoring of structures and determine, using the methods described in NUMARC 93-01, paragraph 9.3.2, to determine whether performance criteria or goals are needed to monitor the performance or condition of individual structures.

2. Review the existing structural monitoring activities and use them, with enhancements as necessary, as a basis for establishing a monitoring program under the rule.

3. Do not use *very reliable* or *inherently reliable* to describe structures that require preventive maintenance or monitoring.

4. Establish performance criteria or goals that are predictive and give early warning of failure.

5. Take credit for existing plant walkdown inspections or other structural inspection activities under the rule.

2.4.7 Functional Failures

The statements of consideration for the rule state that where one or more *maintenance preventible failures* occur on SSCs under paragraph (a)(2) of the rule, the effectiveness of preventive maintenance is no longer demonstrated, and the SSC must then be treated under paragraph (a)(1) of the rule. This term was changed in NUMARC 93-01 to *maintenance preventible functional failures* to emphasize that only a failure in which the item actually failed the function should be counted as a failure that would require goal setting and monitoring under paragraph (a)(1).

Findings

Two licensees focused on *functional failures* rather than just *maintenance preventible functional failures* as described in NUMARC 93-01. These licensees did so because it was easier to process both types of failures (maintenance-related or not) in the same manner because they would not know if the failure was maintenance-preventible until after the root cause evaluation had been performed. The team reviewed this approach and noted that all functional failures would be evaluated and dispositioned in the same manner as maintenance-preventible functional failures.

Conclusion

The use of *functional failures* in stead of *maintenance-preventible functional failures* is acceptable. Approaches other than those described in NUMARC 93-01 are acceptable if the licensee can demonstrate that the alternative gives the same level of assurance that the requirements of the rule will be satisfied.