

FPL NUCLEAR DIVISION

TURKEY POINT NUCLEAR STATION
ST. LUCIE NUCLEAR STATION

MONTHLY INDICATOR REPORT

March 1996

BBB/HH
Issued: April 17, 1996

FPL NUCLEAR DIVISION

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FOREWORD

The Nuclear Division Monthly Indicator Report presents a compilation of performance indicators which provide a quantitative indication of performance. Specific areas of focus include nuclear and personnel safety, plant reliability, and economic performance.

The specific indicators included in this report have been selected by senior management as key indicators of operating performance. Summaries of NRC indicator and WANO indicator performance have been incorporated in this report.

Data contained herein will be refined on the basis of feedback from data providers, of continuing quality control efforts, and of comparisons to other data sources. Each monthly report will reflect the best available data.

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MANAGEMENT SUMMARY

OPERATING PERFORMANCE

Turkey Point Unit 3. Unit 3 achieved an EAF of 95.4% in March. The unit was shutdown March 27th for 34.5 hours as a result of a manual reactor trip due to the loss of the 3B Main Feedwater Pump. For the year, the EAF was 87.4% which is below the 95.0% Y-T-D target.

Turkey Point Unit 4. Unit 4 began powering down for a scheduled refueling outage on March 3rd and remained shutdown through month-end. The EAF for the month was 6.8% and 68.3% Y-T-D, which is higher than the Y-T-D target of 65.8%.

St. Lucie Unit 1. For the month, Unit 1 EAF was 95.8%. Power reduction equating to 30.9 hours was mostly the result of waterbox cleanings. For the year, the EAF was 94.9% which is slightly higher than the Y-T-D target of 94.8%.

St. Lucie Unit 2. Unit 2 essentially operated at full power achieving an EAF of 100.0% for the month. Year-to-date, EAF was 92.0% which is below the target of 95.0%.

Y-T-D Equivalent Availability for the Nuclear Division was 85.6% which is below the 87.6% targeted through the period.

No Unplanned Automatic Trips occurred in March. Year-to-date, one trip was experienced at Turkey Point Unit 3 in February.

A summary of key plant operating statistics is summarized below.

	PTN Unit 3		PTN Unit 4	
	March	Y-T-D	March	Y-T-D
Gross Generation (WMh)	510,722	1,373,032	34,730	1,081,294
Net Generation (MWh)	487,269	1,308,954	32,680	1,032,192
Net Heat Rate (Btu/KWh)	10735.5	10852.5	11806.6	10818.9
Equivalent Availability	95.4%	87.4%	6.8%	68.3%
Capacity Factor	98.3%	90.0%	6.6%	71.0%
Auto Trips	0	1	0	0
Forced Outage Rate	4.6%	9.3%	0.0%	0.0%

	PSL Unit 1		PSL Unit 2	
	March	Y-T-D	March	Y-T-D
Gross Generation (WMh)	632,780	1,824,369	668,680	1,783,700
Net Generation (MWh)	599,306	1,726,652	634,666	1,686,090
Net Heat Rate (Btu/KWh)	10957.8	11026.7	10738.6	10835.2
Equivalent Availability	95.8%	94.9%	100.0%	92.0%
Capacity Factor	96.0%	94.2%	101.7%	92.0%
Auto Trips	0	0	0	0
Forced Outage Rate	0.0%	2.9%	0.0%	2.2%

MANAGEMENT SUMMARY

REGULATORY PERFORMANCE

One NRC Violation was received at Turkey Point in March:

#96-02-04 - Failure to meet requirements of 10CFR50.71(e) in that the FSAR was not appropriately updated to reflect the offload of a full core during a refueling evolution. Exit Meeting Date: 3/29/96.

St. Lucie reported no NRC Violations in March; however, the following were reported for February:

#96-01-01 - Temporary Procedure changes were made which involved changes of intent, without prior FRG review as required by Technical Specifications. Exit Meeting Date: 2/20/96.

#96-03-01* - Operators failed to follow procedures for boron dilution. Exit Meeting Date: 2/8/96.

#96-03-02* - Inadequate design control of RCS boron dilution procedure. Exit Meeting Date: 2/8/96.

#96-03-03* - Inadequate 50.59 safety evaluation of change to boron dilution procedure. Exit Meeting Date: 2/8/96.

* Note: On April 1, St. Lucie was informed that these violations represent a Severity Level III penalty for Unit 1 only. A Civil Penalty of \$50,000 was imposed.

Year-to-date, Turkey Point has reported one NRC Violation, and St. Lucie has reported four.

COST PERFORMANCE

O&M expenditures through March 1996 were \$67.4 million which represented a budget underrun of \$8.4 million (or 11.1%). This variance was primarily due to: St. Lucie Unit 1 outage schedule change; St. Lucie non-outage project underruns; and overtime payroll underrun associated with Turkey Point Unit 4 outage.

February O&M budget performance variances are stratified as follows:

Turkey Point Site Specific	\$2.9 million (or 7.4%) below budget
St. Lucie Site Specific	\$5.8 million (or 20.8%) below budget
Other Nuclear Division	\$0.3 million (or 4.2%) above budget

Capital expenditures for March Y-T-D were \$2.9 million. This represented a budget underrun of \$8.2 million (or 73.7%). The favorable variance was primarily due to: an underrun in the Steam Generator Replacement Project (SGRP) reflecting a revision to the SGRP outage start date; St. Lucie Unit 2 Reactor Head Seal Ring Replacement Project determined to be O&M; and plant cancelled projects.

For the month, Capital budget performance variances are detailed as follows:

Turkey Point Site Specific	\$1.0 million (or 94.7%) below budget
St. Lucie Site Specific	\$6.3 million (or 67.7%) below budget
Other Nuclear Division	\$0.9 million (or 124.1%) below budget

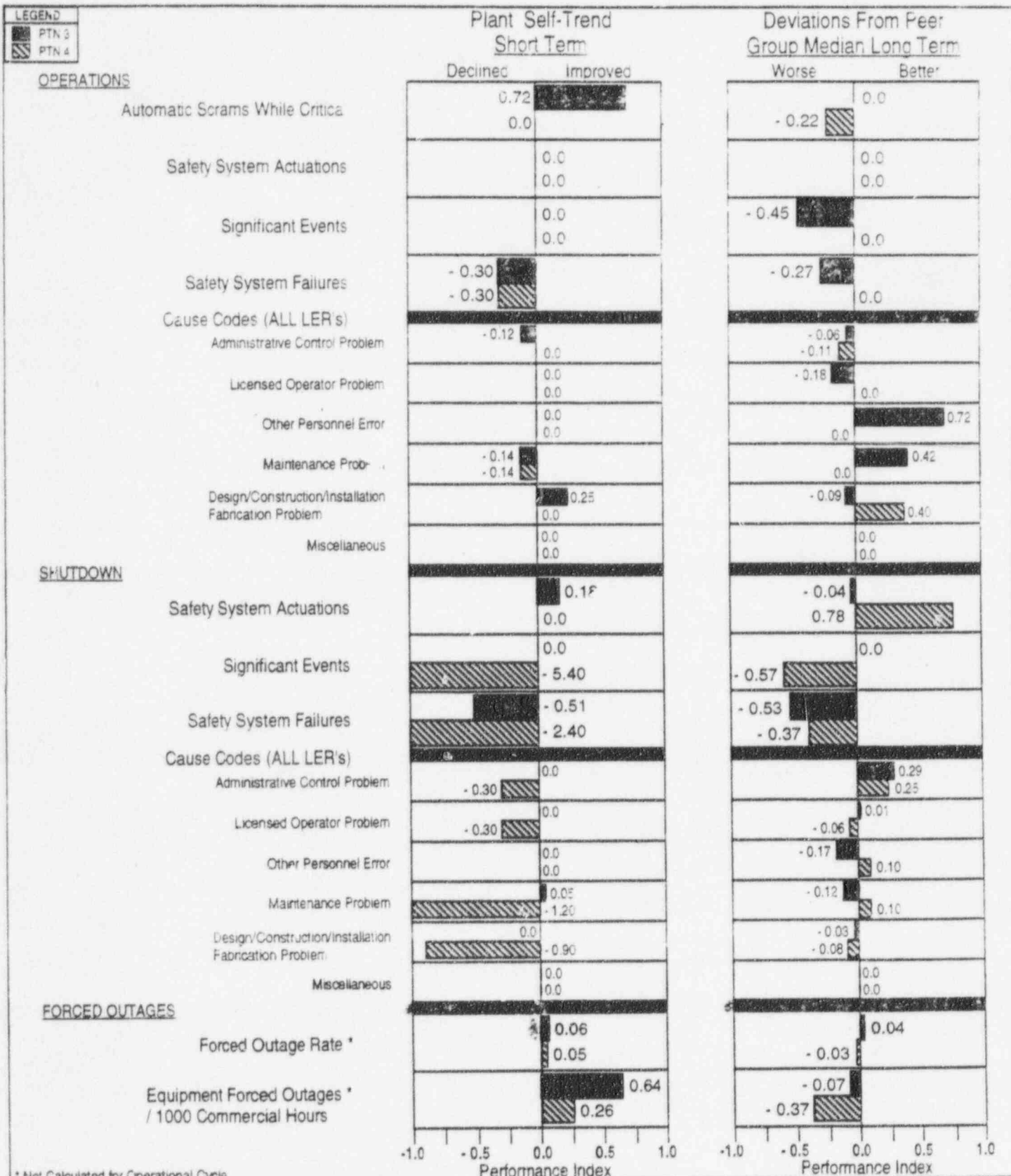
NUCLEAR DIVISION BUSINESS PLAN INDICATOR OVERVIEW

(DATA THROUGH MARCH 31, 1996)

Indicator		Plant	1996 Y-E Target	1996 Y-T-D Target	1996 Y-T-D Actuals	Comments
Collective Radiation Exposure (Man-Rem)		PTN PSL	≤ 275.0 ≤ 485.0	≤ 225.0 ≤ 33.0	153.9 13.7	Turkey Point and St. Lucie were below Y-T-D Man-Rem targets.
NRC SALP Ratings		PTN PSL	1.00 1.50		1.00 1.50	The SALP for St. Lucie ending 1/6/96 was 1.50; for PTN, the period ends Aug. 31, 1996.
NRC Violations		PTN PSL	≤ 7 ≤ 7		1 4	PTN reported one NOV in March; PSL had no NOV's in March; four were reported for February.
Unplanned Automatic Trips		PTN3 PTN4 PSL1 PSL2	≤ 3		1 0 0 0	No unplanned automatic trips were reported in March.
Budget Performance (\$ Millions)		Capital O&M Div. Total	39.2 253.0 292.2	11.1 75.8 86.9	2.9 67.4 70.3	For the year, O&M and Capital actuals were below Year-End targets; underruns were mainly due to schedule revisions and project underruns.
Equivalent Availability Factor (%)		PTN3 PTN4 PSL1 PSL2	95.0 82.0 78.5 95.0	95.0 65.8 94.8 95.0	87.4 68.3 94.9 92.0	PSL1 slightly exceeded the Y-T-D EAF target in March. Division EAF Y-T-D was 85.6%, which is below the 87.6% Y-T-D target.
M&S Inventory Levels (\$ Millions)		PTN PSL	≤ 38 ≤ 38		36.0 43.5	PSL Inventory increased \$1.2M in March primarily due to the scheduled Unit 1 outage of 4/29/96.
Cost (¢) Per KWh	Production Cost (O&M and Fuel)	PTN PSL	1.75 1.55	2.37 1.49	2.31 1.36	Production Costs (¢/KWh) were below the Y-T-D target in March.
	Total Cost (O&M, Fuel, and Capital Carrying Costs)	Div. Total	4.56	4.76	4.70	In March, Division Total Cost (¢/KWh) was below the Y-T-D target, but higher than the Y-E target.
Nuclear Division Staffing Levels		FPL LT Contr Total	2045.0 463.0 2508.0		1959.0 459.0 2418.0	Total Nuclear Division Staffing Levels year-to-date was below the Y-E target.
Refueling Outage Duration (Days)		PTN3 PTN4 PSL1 PSL2	NA 45 63 NA		0 29 0 5	PTN 4 began powering down for a scheduled refueling on 3/3. Future refueling outage schedules are: PTN3 03/08/97 PSL1 04/29/96 PTN4 09/15/97 PSL2 04/15/97
Lost Time Injuries per 200,000 Hours Worked (12 Month Running)		PTN PSL	0.30 0.30		0.11 0.32	No Lost Time Injuries or Restricted Duty Cases were reported in March.

NRC INDICATOR PERFORMANCE OVERVIEW

for Turkey Point (Data through Quarter Ending September 30, 1995)



* Not Calculated for Operational Cycle

NOTES:

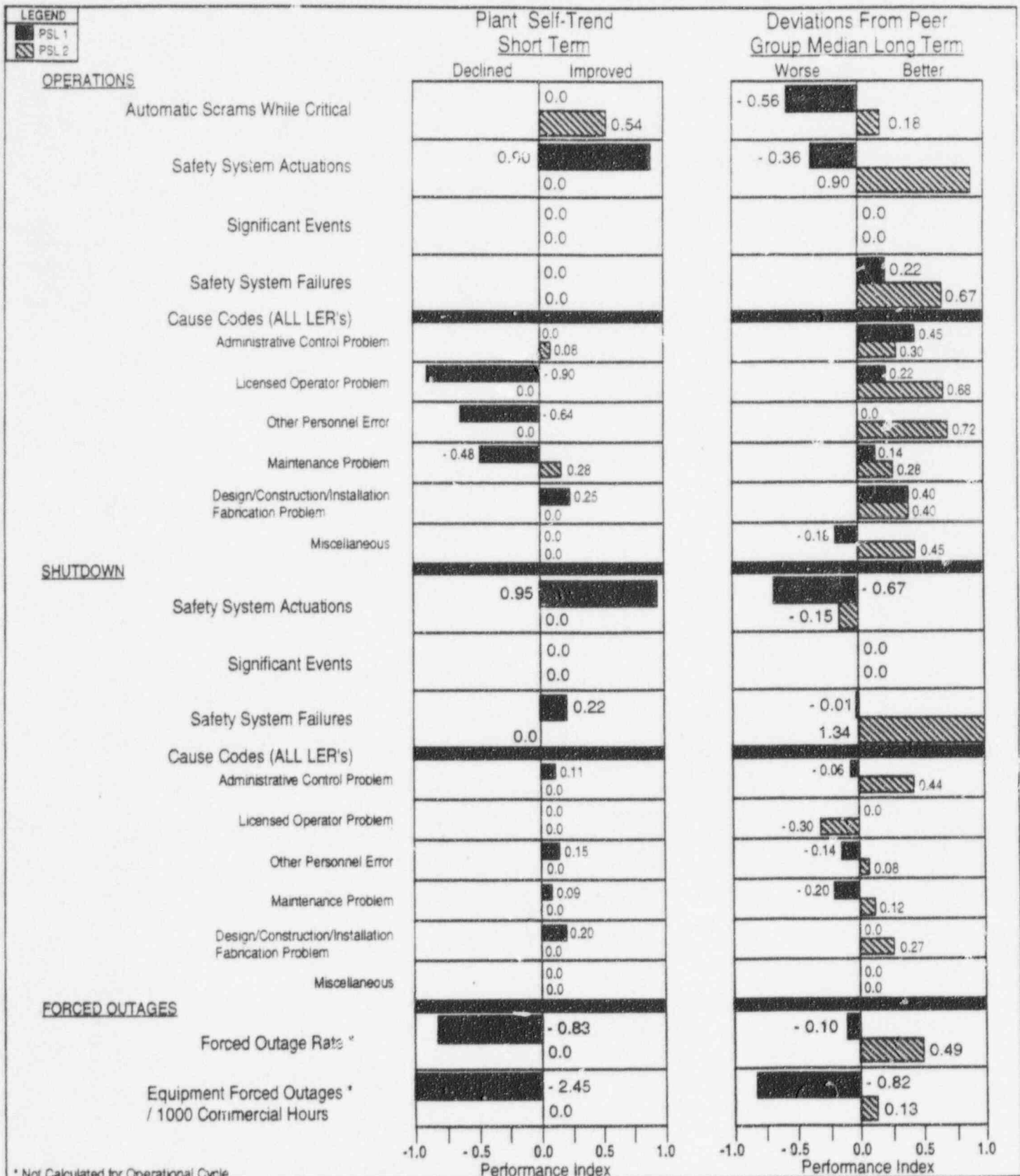
Plant Self-Trend Short Term: Based on the slope of a linear regression line plotted over each plant's data. Time intervals used in the trends are 4 Quarters for "Operations" and Forced Outages" indicators and 6 quarters for Shutdown" indicators.

Deviations from Peer Group Median Long Term: Comparisons are made of each plant to the performance of its peers over a 12 Quarter time interval.

Peer Groups: PTN 3&4 - Older Westinghouse 3-Loop.

NRC INDICATOR PERFORMANCE OVERVIEW

for St. Lucie (Data through Quarter Ending September 30, 1995)



* Not Calculated for Operational Cycle

NOTES:

Plant Self-Trend Short Term: Based on the slope of a linear regression line plotted over each plant's data. Time intervals used in the trends are 4 Quarters for "Operations" and Forced Outages" indicators and 6 quarters for Shutdown" indicators.

Deviations from Peer Group Median Long Term: Comparisons are made of each plant to the performance of its peers over a 12 Quarter time interval.

Peer Group: PSL 1&2 - Combustion Engineering with core protection calculator plants.

WANO OVERALL INDICATOR PERFORMANCE OVERVIEW

(March 31, 1996)

Performance Indicators		Unit or Station Values				Industry Median Values *	
		PTN 3	PTN 4	PSL 1	PSL 2	3-Year Distribution (1993-1995)	2000 Goals
Unit Capability Factor (Unit %, 3-Yr. Distribution Ending March '96)		88.7%	84.6%	77.9%	80.5%	81.2%	87.0%
Unplanned Capability Loss Factor (Unit %, 3-Year Distr. Ending March '96)		3.1%	4.3%	11.0%	6.6%	5.2%	3.0%
Unplanned Automatic Scrams Per 7000 Hours Critical (Per Unit, 3-Year Distribution Ending March '96)		0.6	0.9	1.6	0.6	0.9	1.0
Safety System Performance:	High Pressure Safety Injection System (Per Unit, 3-Year Distribution Ending March '96)	0.008	0.009	0.026	0.009	0.003 - 0.008	not available
	Auxiliary Feedwater System (Per Unit, 3-Year Distribution Ending March '96)	0.015	0.012	0.010	0.013	0.004 - 0.009	not available
	Emergency AC Power System (Per Unit, 3-Year Distribution Ending March '96)	0.006	0.003	0.019	0.013	0.007 - 0.012	not available
Thermal Performance (Ratio of Design to Actual Gross Heat Rate, 1-Yr. Distribution Ending March '96)		99.9%	100.1%	98.9%	98.2%	99.5% **	99.5%
Collective Radiation Exposure (Man-Rem per unit per year, 3-year running avg. ending March '96)		184	184	220	220	153	120
Volume of Low-level Solid Radioactive Waste (Cubic meters per unit per year, 3-year avg. ending March '96)		33.1	33.1	54.6	54.6	47.0	45.0
Chemistry Index (12-Mo. average through March '96)		1.11	1.03	1.18	1.19	1.18 **	1.10
Industrial Safety Lost-time Accident Rate (Station rate per 200,000 man-hours worked ending March '96)		0.11		0.32		0.55 **	0.40
Fuel Reliability (Unit microcuries/g, month ending March '96)		1.00E-6	Outage	3.81E-5	1.69E-5	Fuel Defect Reference 5.0E-04***	

NOTE: Shaded area denotes FPL performance is unfavorable to actual industry median.

Source of Industry Data:

- * 1995 Year-End Report for Performance Indicators for the U.S. Nuclear Utility Industry (1993-1995 Distribution).
- ** 1995 Year-End Report for Performance Indicators for the U.S. Nuclear Utility Industry (1995 Distribution).
- *** 1995 Year-End Report for Performance Indicators for the U.S. Nuclear Utility Industry (4/95-6/95 Distribution).

WANO OVERALL INDICATOR PERFORMANCE OVERVIEW

Discussion of FPL Performance Unfavorable to Industry Median

(March 31, 1996)

UNIT CAPABILITY FACTOR

(3-Years Ending 3/31/96)

St. Lucie Unit 1. The 3-year running Capability Factor for the unit was 77.9%. Capability loss is attributed to the following: Refueling Outage and extension from 3/29/93 to 6/17/93 (7.4%); Hot Leg Valve MV-3480 leak repairs from 3/29/94 to 4/2/94 (0.4%); Main Transformer trip from 6/6/94 to 6/11/94 (0.4%); Quench Tank leak repairs from 2/27/95 to 3/8/95 (0.9%); 1A2 Reactor Coolant Pump seal repairs from 8/2/95 to 8/9/95 (0.7%); inoperable Power Operated Relief Valves from 8/9/95 to 8/17/95 (0.7%); and, inadvertent Containment Spray actuation and clean-up from 8/17/95 to 9/3/95 (1.3%) 1B2 Diesel Generator failure from 9/1/95 to 9/6/95 (0.5%); 1A Diesel Generator Radiator leakage from 9/6/95 to 9/10/95 (0.4%); and, Code Safety Valve repairs and modifications from 9/11/95 to 10/13/95 (2.4%). Other miscellaneous unplanned outages and derates accounted for the remaining 7.0% Unit Capability Factor loss.

St. Lucie Unit 2. Capability Factor for the three years ending 3/31/96 was 80.5%. Capability loss is attributed to the following: dropped CEA's on 5/21/93 (0.5%); Condenser Tube leak repairs from 8/9/93 to 8/11/93 (0.2%); refueling outage from 2/13/94 to 4/22/94 (4.6%); shutdown for auto reactor trip investigation on 4/23/94 (0.3%); and, a refueling outage from 10/9/95 to 1/5/96 (7.9%). Other unplanned outages and power reductions accounted for the remaining 6.0% Unit Capability Factor loss.

UNPLANNED CAPABILITY LOSS FACTOR

(3-Years Ending 3/31/96)

St. Lucie Unit 1. The Unplanned Capability Loss Factor for the three years ending 3/31/96 was 11.0% compared to an industry median of 5.2%. Unplanned outages and power reductions contributing to this performance included: Refueling Outage extension from 6/1/93 to 6/17/93 (1.6%); Waterbox cleaning due to jelly fish intrusion from 9/18/93 to 9/29/93 (0.7%); Hot Leg Valve MV-3480 leak repairs from 3/29/94 to 4/2/94 (0.4%); Main Transformer trip from 6/6/94 to 6/11/94 (0.4%); Quench Tank leak repairs from 2/27/95 to 3/8/95 (0.9%); 1A2 Reactor Coolant Pump seal repairs from 8/2/95 to 8/9/95 (0.7%); inoperable Power Operated Relief Valves from 8/9/95 to 8/17/95 (0.7%); inadvertent Containment Spray actuation and clean-up from 8/17/95 to 9/1/95 (1.3%) 1B2 Diesel Generator failure from 9/1/95 to 9/6/95 (0.5%); 1A Diesel Generator Radiator leakage from 9/6/95 to 9/10/95 (0.4%); and, Code Safety Valve repairs and modifications from 9/11/95 to 10/31/95 (2.4%). Other miscellaneous unplanned outages and derates accounted for the remaining 1.0% Capability Loss.

St. Lucie Unit 2. The Unplanned Capability Loss Factor for the three years ending 3/31/96 was 6.6%. Major unplanned occurrences contributing to this performance included: dropped CEA's on 5/21/93 (0.5%); shutdown for auto reactor trip investigation on 4/23/94 (0.3%); and, refueling outage extension from 12/1/95 to 1/5/96 (3.2%). Other unplanned outages and power reductions accounted for the remaining 2.6% in Capability Loss.

UNPLANNED AUTOMATIC SCRAMS PER 7000 HOURS CRITICAL

(3-Years Ending 3/31/96)

St. Lucie Unit 1. Increased rate for Unit 1 was the result of five auto trips occurring on 3/18/94, 4/3/94, 6/6/94, 10/26/94 and 7/8/95.

HIGH PRESSURE SAFETY INJECTION SYSTEM

(3-Years Ending 3/31/96)

Turkey Point Unit 4. Average performance was affected due to the on-line replacement of the HPSI pump motors following discovery of cracked rotor bars.

St. Lucie Unit 1. Average performance for the last three years was affected by on line Motor Operated Valve testing in the 3rd Quarter of 1994 and a breaker failure on 2B HPSI pump in the 1st Quarter of 1995.

St. Lucie Unit 2. In the 1st Quarter of 1995, average performance was affected as a result of Component Cooling Water (CCW) Heat Exchanger cleaning which placed the respective HPSI pump OOS due to lack of dedicated sea cooling.

AUXILIARY FEEDWATER SYSTEM

(3-Years Ending 3/31/96)

Turkey Point. Average performance for both units was affected by the B AFW Turbine failure in the 4th Quarter of 1994 due to malfunction of the mechanical overspeed trip device: in the 3rd Quarter of 1995, performance was affected by: Part 21 repairs on the Trip and Trottle Valves, and Unit 3 outage work.

St. Lucie. Average performance for three-years was affected by failure in the 4th Quarter of 1994 of the 1C AFW Pump Governor. In the 3rd Quarter of 1995, performance was attributed to: failure of the AFW PP 2C Steam Admission Valve MV-08-13 to open, a mechanical trip linkage for AFW PP 2C when the Electrical Overspeed Solenoid was energized, and a discrepancy between field wiring and plant wiring drawing for the AFW PP 2B.

EMERGENCY DIESEL GENERATOR SYSTEM

(3-Years Ending 3/31/96)

St. Lucie Unit 1. Unit 1's average performance for three-years was the result of a high water jacket temperature trip of 1A EDG and failure of the governor on 1B2 during monthly surveillance run which closed off fuel to the 12 cylinder engine in the 2nd Quarter of 1995, 1B diesel 12 cylinder engine valve failure in the 3rd Quarter of 1995, and 1B diesel due to replacement of the cooling water valves in the 4th Quarter 1995.

St. Lucie Unit 2. Unit 2's average performance for three-years was affected by a failure of the 2A EDG Governor in the 4th Quarter of 1994.

COLLECTIVE RADIATION EXPOSURE - MAN-REM

(3-Years Ending 3/31/96)

Turkey Point. The three-year running average Collective Radiation Exposure level for Turkey Point was 184 Man-Rem per unit which was greater than the industry median of 153 Man-Rem. Site performance was influenced by scheduled refueling outages.

St. Lucie. Collective Radiation Exposure three-year running average level for St. Lucie was 220 Man-Rem per unit which was greater than the industry median of 153 Man-Rem. Site performance was influenced by unplanned and scheduled outages.

VOLUME OF SOLID RADIOACTIVE WASTE

(Annual Avg. for 3-Years Ending 3/31/96)

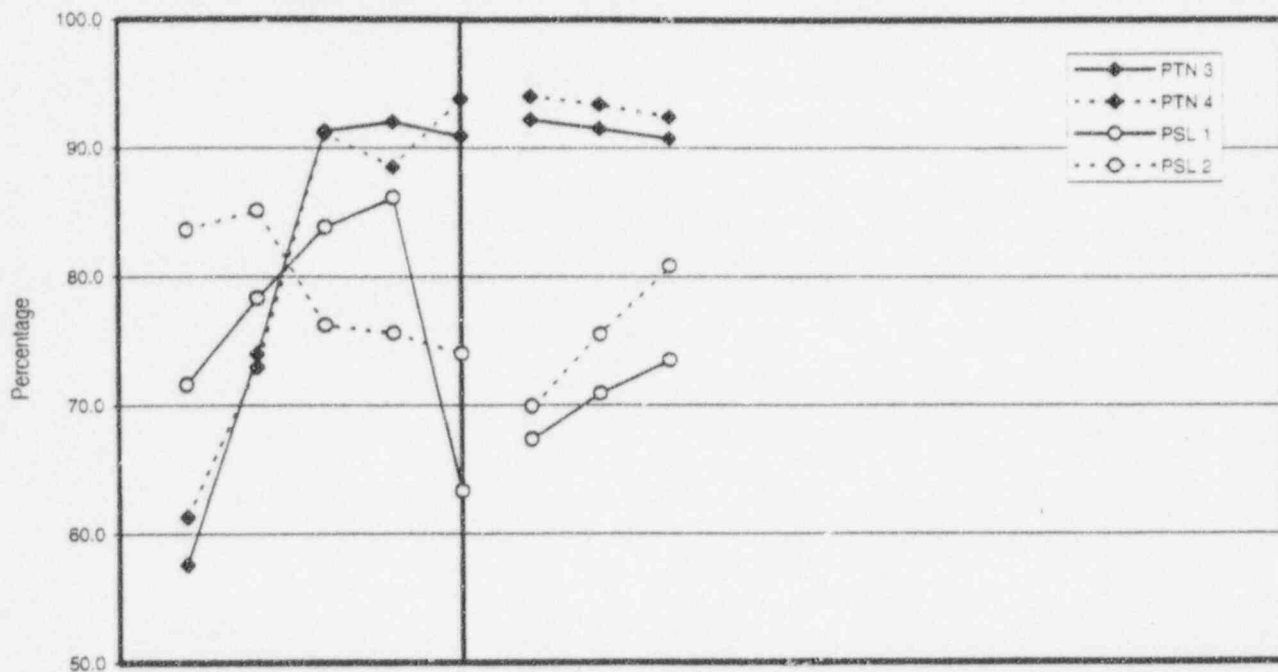
St. Lucie. Volume of Solid Radioactive Waste for the 3-year annual average was higher than industry median due to 1994 waste volume: the 1995 value of 41.9 cubic meters was below the median value.

CHEMISTRY INDEX

(12 Mo's Ending 3/31/96)

St. Lucie Unit 2. Unit 2 exceeded target as a result of elevated condensate dissolved oxygen.

WANO WEIGHTED OVERALL PERFORMANCE



Unit	1991	1992	1993	1994	1995	1/96	2/96	3/96	4/96	5/96	6/96	7/96	8/96	9/96	10/96	11/96	12/96
PTN 3	57.6	74.0	91.3	92.0	90.9	92.2	91.5	90.7									
PTN 4	61.3	73.0	91.1	88.5	93.8	94.0	93.4	92.4									
PSL 1	71.6	78.3	83.8	86.1	63.3	67.3	70.9	73.5									
PSL 2	83.6	85.1	76.2	75.6	74.0	69.9	75.5	80.8									

DEFINITION

The WANO Overall Performance Index is a composite indicator utilized to trend nuclear station performance.

The index is a weighted combination of the following 10 individual performance indicators:

- | | | |
|---|---------------------------------------|--------------------------------------|
| 1. Unit Capability Factor (15%) | 5. Emergency AC Power (9%) | 9. Thermal Performance (6%) |
| 2. Unplanned Capability Loss Factor (12%) | 6. Unplanned Auto-Scrams (8%) | 10. Chemistry Indicator (6%) |
| 3. High Pressure Safety Injection (9%) | 7. Collective Radiation Exposure (8%) | 11. Low-level Radwaste Volume (5%) |
| 4. Auxiliary Feedwater System (9%) | 8. PWR Fuel Reliability (7%) | 12. Industrial Safety Accidents (5%) |

STATISTICAL SUMMARY

INDUSTRY PERFORMANCE

WANO:

Through 4th Quarter 1995 - Industry Median: 82.0%

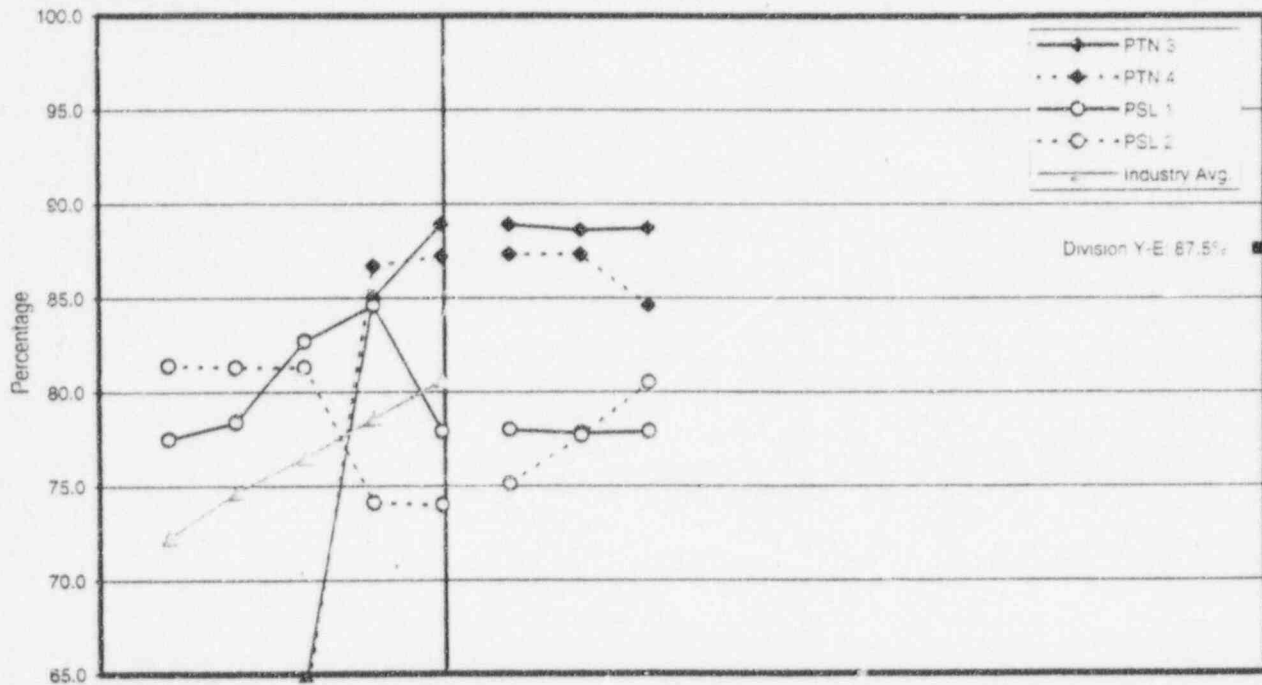
PERFORMANCE SUMMARY

Turkey Point Units 3 and 4 overall performance exceeds WANO's industry median of 82.0%.

Data Provider: Sharon Bilger 694-4255

UNIT CAPABILITY FACTOR

(3-Year Running Average)



Division Y-E: 87.5%

Unit	1991	1992	1993	1994	1995	1/96	2/96	3/96	4/96	5/96	6/96	7/96	8/96	9/96	10/96	11/96	12/96
PTN 3	47.9	52.7	64.6	85.0	88.9	88.9	88.6	88.7									
PTN 4	42.2	61.8	63.7	86.7	87.2	87.3	87.3	84.6									
PSL 1	77.5	78.4	82.7	84.6	77.9	78.0	77.8	77.9									
PSL 2	81.4	81.3	81.3	74.1	74.0	75.1	77.7	80.5									
Industry Avg	72.3	74.6	76.5	78.6	80.6												

DEFINITION

Unit Capability Factor is the ratio of the available energy generation over a given time period to the reference energy generation over the same time period, expressed as a percentage with both energy generation terms determined relative to reference ambient conditions. Available energy generation is the energy that could have been produced under reference ambient conditions considering only limitations within control of plant management, i.e., plant equipment and personnel performance, and work control. Reference energy generation is the energy that could be produced if the unit were operated continuously at full power under reference ambient conditions throughout the period. Reference ambient conditions are environmental conditions representative of the annual mean (or typical) ambient conditions for the unit.

STATISTICAL SUMMARY

	Mar	Y-T-D	3-yr Running	Y-E Target
PTN 3	95.4%	87.4%	88.7%	≥ 95.0%
PTN 4	6.8%	68.3%	84.6%	≥ 82.0%
PSL 1	95.8%	94.9%	77.9%	≥ 78.0%
PSL 2	100.0%	92.0%	80.5%	≥ 95.0%

INDUSTRY PERFORMANCE

WANO	
1993 - 1995 Median	81.2%
1995 Median	82.6%
1995 Average	79.4%
1995 Goal	80.0%

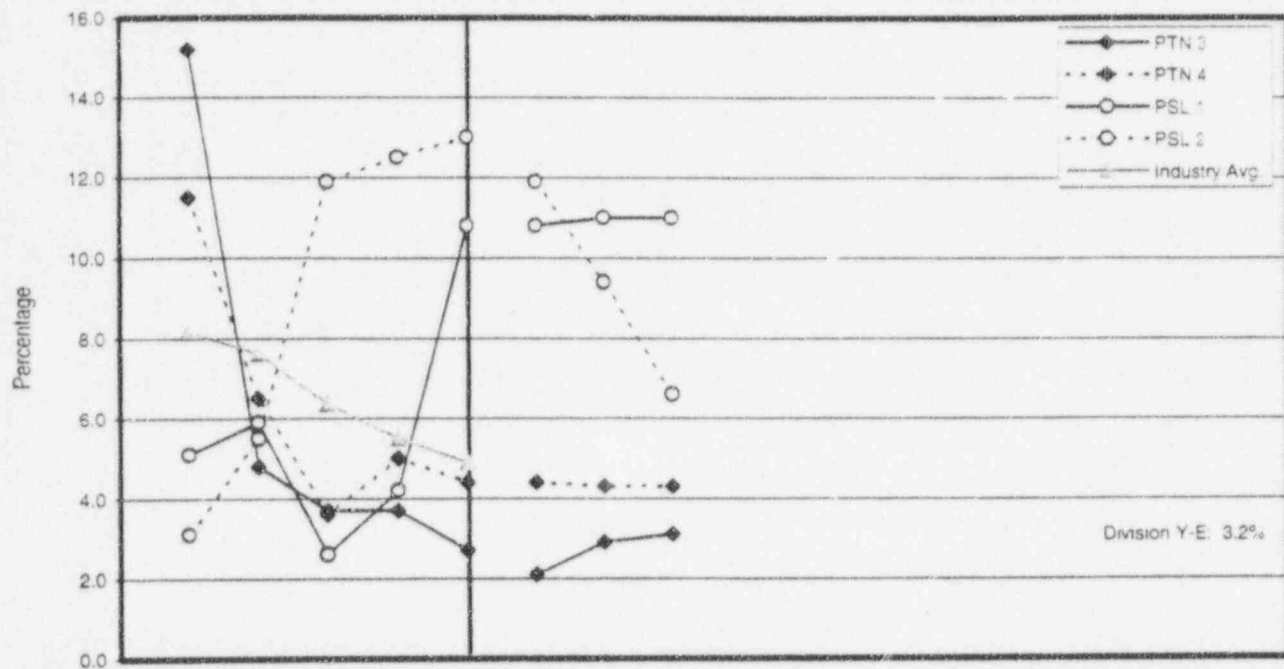
PERFORMANCE SUMMARY

Turkey Point Units 3 and 4 Capability Factor (3-yr. running average) performance through March 1996 was higher than WANO's 3-yr. running industry median.

Data Provider: Sharon Bilger 694-4255

UNPLANNED CAPABILITY LOSS FACTOR

(3-year Running Average)



Division Y-E: 3.2%

Unit	1991	1992	1993	1994	1995	1/96	2/96	3/96	4/96	5/96	6/96	7/96	8/96	9/96	10/96	11/96	12/96
PTN 3	15.2	4.8	3.7	3.7	2.7	2.1	2.9	3.1									
PTN 4	11.5	6.5	3.6	5.0	4.4	4.4	4.3	4.3									
PSL 1	5.1	5.9	2.6	4.2	10.8	10.8	11.0	11.0									
PSL 2	3.1	5.5	11.9	12.5	13.0	11.9		6.6									
Industry Avg.	8.2	7.6	6.4	5.5	4.9												

DEFINITION

Unplanned Capability Loss Factor is defined as the ratio of the unplanned energy losses during a given period of time to the reference energy generation, expressed as a percentage. Unplanned energy loss is energy that was not produced during the period because of unplanned shutdowns, outage extensions, or unplanned load reductions due to causes under plant management control. Causes of energy losses are considered to be unplanned if they are not scheduled at least four weeks in advance. Reference energy generation is the energy that could be produced if the unit were operated continuously at full power under reference ambient conditions throughout the period. Reference ambient conditions are environmental conditions representative of the annual mean (or typical) ambient conditions for the unit.

STATISTICAL SUMMARY

3-yr Running Avg

PTN 3	3.1%
PTN 4	4.3%
PSL 1	11.0%
PSL 2	6.6%

1996 Target (all units): ≤ 3.2%

INDUSTRY PERFORMANCE

WANO

1993 - 1995 Median	5.2%
1995 Median	5.0%
1995 Goal	4.5%

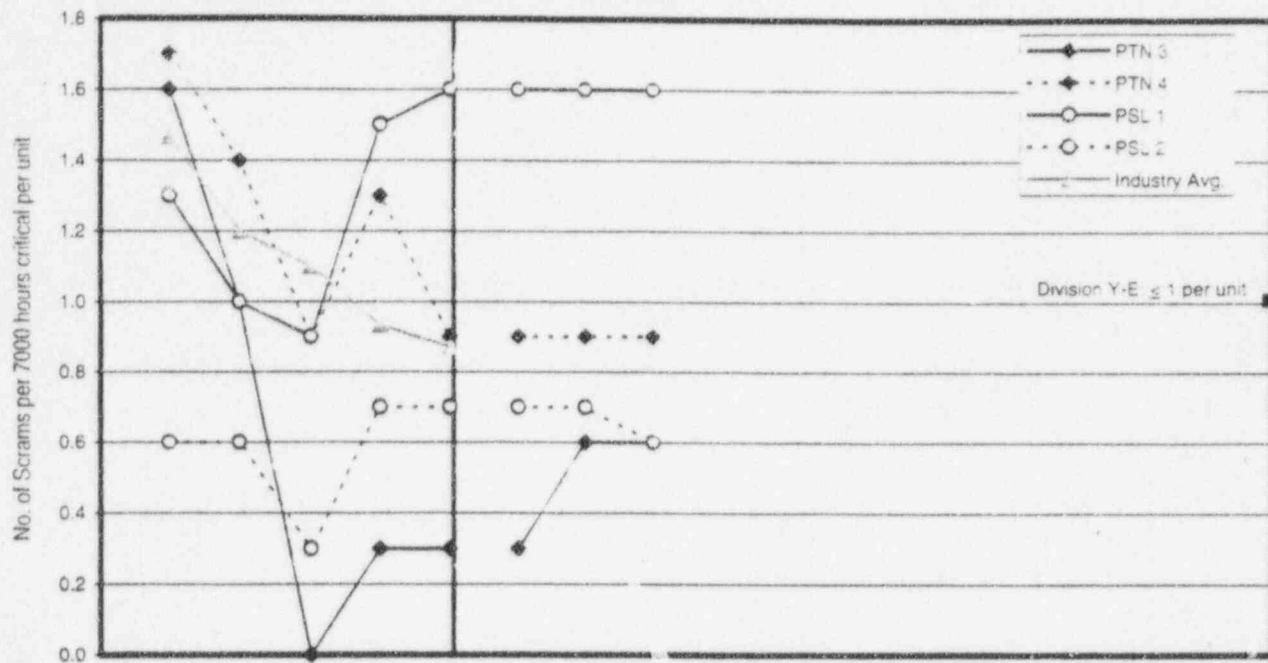
PERFORMANCE SUMMARY

St. Lucie Units 1 and 2's Unplanned Capability Loss Factor performance for 3-yr running through March 1996 was higher than the WANO's 3-year running industry median.

Data Source: John Pizzutelli 694-4245

UNPLANNED AUTO TRIPS PER 7000 HOURS CRITICAL

(3-year Running Average)



Unit	1991	1992	1993	1994	1995	1/96	2/96	3/96	4/96	5/96	6/96	7/96	8/96	9/96	10/96	11/96	12/96
PTN 3	1.6	1.0	0.0	0.3	0.3	0.3	0.6	0.6									
PTN 4	1.7	1.4	0.9	1.3	0.9	0.9	0.9	0.9									
PSL 1	1.3	1.0	0.9	1.5	1.6	1.6	1.6	1.6									
PSL 2	0.6	0.6	0.3	0.7	0.7	0.7	0.7	0.6									
Industry Avg.	1.5	1.2	1.1	0.9	0.9												

DEFINITION

Unplanned Automatic Scrams per 7000 hours Critical is defined as the number of unplanned automatic scrams that occur per 7000 hours of critical operation. Unplanned means that the scram was not an anticipated part of a planned test. Scram means the automatic shutdown of the reactor by a rapid insertion of negative reactivity (by control rods, liquid injection shutdown system, etc.) caused by actuation of the reactor protection system. The scram signal may have resulted from exceeding a setpoint or may have been spurious. Automatic means that the initial signal that caused actuation of the reactor protection system logic was provided from one of the sensors monitoring plant parameters and conditions rather than from the manual scram switches or, in certain cases described in INPO 94-009, from manual turbine trip switches provided in the main control room. Critical means that during the steady-state condition prior to the scram, the effective reactor multiplication factor was essentially equal to one.

STATISTICAL SUMMARY

	Through March	
	12-mo.	36-mo.
Division Totals:		
1996	0.5	0.9
1995	1.8	1.0

Division Target: ≤ 3.0 auto trips

INDUSTRY PERFORMANCE

WANO

1993 - 1995 Median	0.9
1995 Median	0.9
1995 Goal	1.0

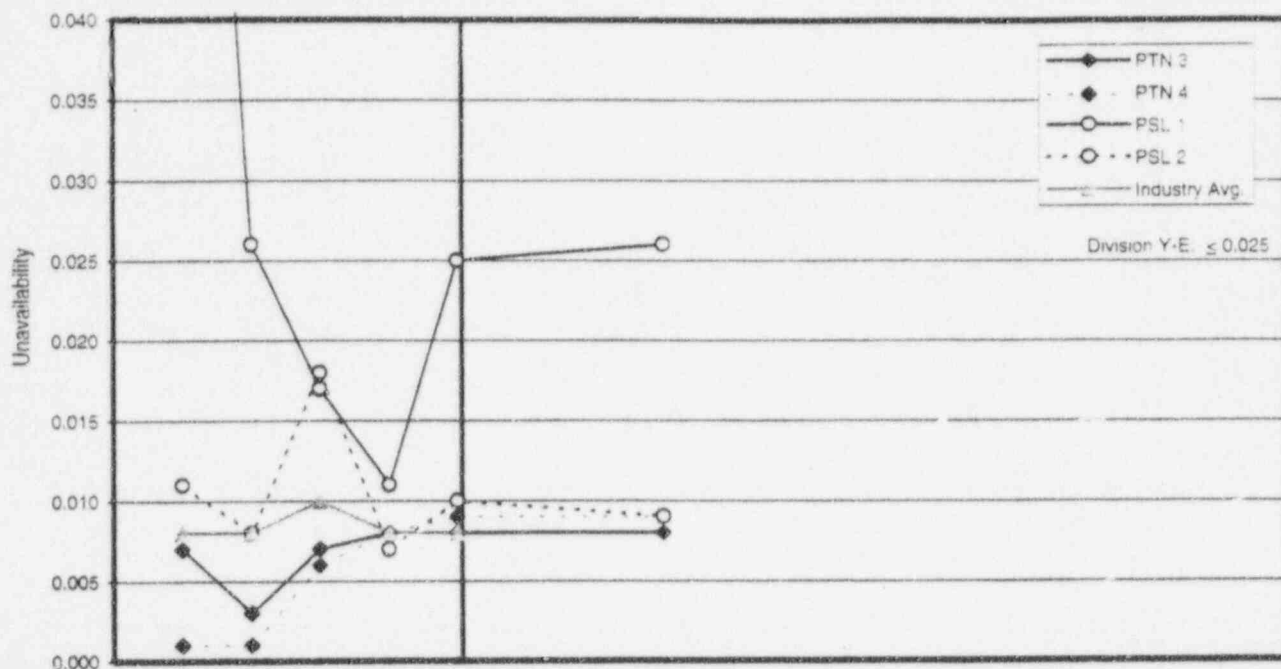
PERFORMANCE SUMMARY

- No unplanned automatic trips were reported in March.
- Turkey Point Unit 3 experienced an automatic trip on February 9th. The "B" Steam Generator Feed Pump was stopped to monitor its discharge check valve closing stroke which did not strike closed as expected. The resulting feed flow transient caused the "C" Steam Generator level to increase resulting in a turbine trip which tripped the reactor. Unit 3's 3-year running average of 0.6 was below the industry median.
- St. Lucie Unit 1's 3-year running average of 1.6 exceeded WANO's 3-year running average of 1.0 as a result of four trips occurring in 1994 and one in 1995.

Data Providers: (PTN) Jim Knorr 246-6757 and (PSL) Kelly Korth 467-7054

HIGH PRESSURE SAFETY INJECTION SYSTEM PERFORMANCE

(3-yr Running Average)



Unit	1991	1992	1993	1994	1995	3/96	6/96	9/96	12/96
PTN 3	0.007	0.005	0.007	0.008	0.008	0.008			
PTN 4	0.001	0.001	0.006	0.008	0.009	0.009			
PSL 1	0.091	0.026	0.017	0.011	0.025	0.026			
PSL 2	0.011	0.008	0.018	0.007	0.010	0.009			
Industry Avg.	0.008	0.008	0.010	0.008	0.008				

DEFINITION

This Safety System Performance indicator monitors the readiness of the Safety Injection (SI) System at Turkey Point and the High Pressure Safety Injection (HPSI) System at St. Lucie to respond to off-normal events or accidents. The indicator is determined from the unavailabilities, due to all causes, of the components in the system during a time period, divided by the number of trains in the system. The definition is further explained: component unavailability is the ratio of the hours the component was unavailable (unavailable hours) to the hours the system was required to be available for service. Data is reported on a quarterly basis.

$$\text{Unavailability} = \frac{(\text{Known Unavailable Hours}) + (\text{Estimated Unavailable Hours})}{(\text{Hours System Required}) \times (\text{Number of Trains})}$$

STATISTICAL SUMMARY

	1st Qtr 1996	3-yr Avg Ending 3/96
PTN 3	0.006	0.008
PTN 4	0.005	0.009
PSL 1	0.015	0.026
PSL 2	0.000	0.009

Targets:	PTN	PSL
1995 Y-E Target	≤ 0.016	≤ 0.023
1996 Y-E Target	≤ 0.025	≤ 0.025

INDUSTRY PERFORMANCE

WANO

1993 - 1995 Median (PWR)
1995 Goal

Unavailability

0.003 - 0.008
0.020

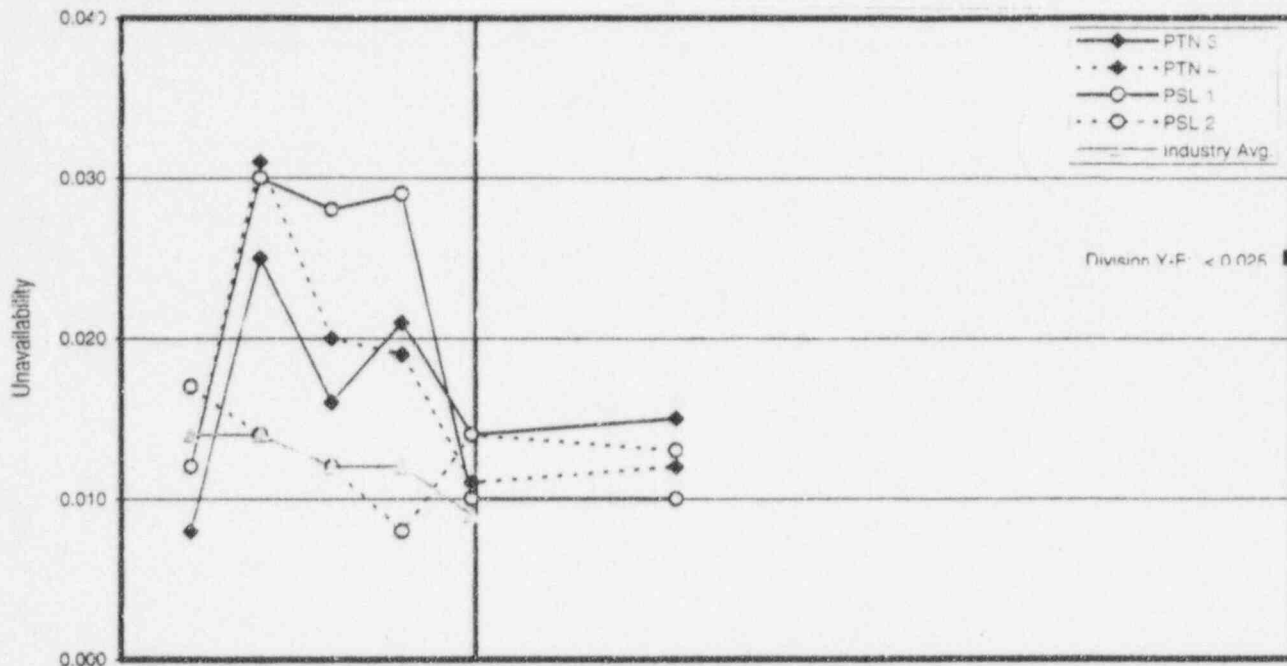
PERFORMANCE SUMMARY

- Turkey Point Unit 3's 3-year running average for Safety Injection System performance through the 1st Quarter 1996 was below the year-end target and industry median. Unit 4's performance for 3-years was below the year-end target and slightly higher than industry average.
- St. Lucie Units 1 and 2's High Pressure Injection System average performance for 3-years running was higher than the industry median through the 1st Quarter 1993; Unit 2's performance was below the year-end target.

Data Providers: (PTN) Ed Lyons 246-6967/Carlos Melchor 246-6964
(PSL) Chuck Wood 467-7034/Bob Young 467-7063

SAFETY SYSTEM PERFORMANCE - AUXILIARY FEEDWATER SYSTEM

(3-year Running Average)



Unit	1991	1992	1993	1994	1995	3/96	6/96	9/96	12/96
PTN 3	0.008	0.025	0.016	0.021	0.014	0.015			
PTN 4	0.012	0.031	0.020	0.019	0.011	0.012			
PSL 1	0.012	0.030	0.028	0.029	0.010	0.010			
PSL 2	0.017	0.014	0.012	0.008	0.014	0.013			
Industry Avg.	0.014	0.014	0.012	0.012	0.009				

DEFINITION

This Safety System Performance indicator monitors the readiness of the Auxiliary Feedwater (AFW) System to respond to off-normal events or accidents. The indicator is determined from the unavailabilities, due to all causes, of the components in the system during a time period, divided by the number of trains in the system. This definition is further explained: component unavailability is the ratio of hours the component was unavailable (unavailable hours) to the hours the system was required to be available for service.

$$\text{AFW Unavailability} = \frac{(\text{Known Unavailable Hours}) + (\text{Estimated Unavailable Hours})}{(\text{Hours System Required}) \times (\text{Number of Trains})}$$

STATISTICAL SUMMARY

	1st Qtr 1996	3-yr Avg Ending 3/96
PTN 3	0.008	0.015
PTN 4	0.007	0.012
PSL 1	0.005	0.010
PSL 2	0.008	0.013
Targets:		
1995 Y-E Target	≤ 0.020	≤ 0.021
1996 Y-E Target	≤ 0.025	≤ 0.025

INDUSTRY PERFORMANCE

WANO	Unavailability
1993 - 1995 Median (PWR)	0.004 - 0.009
1995 Goal	0.025

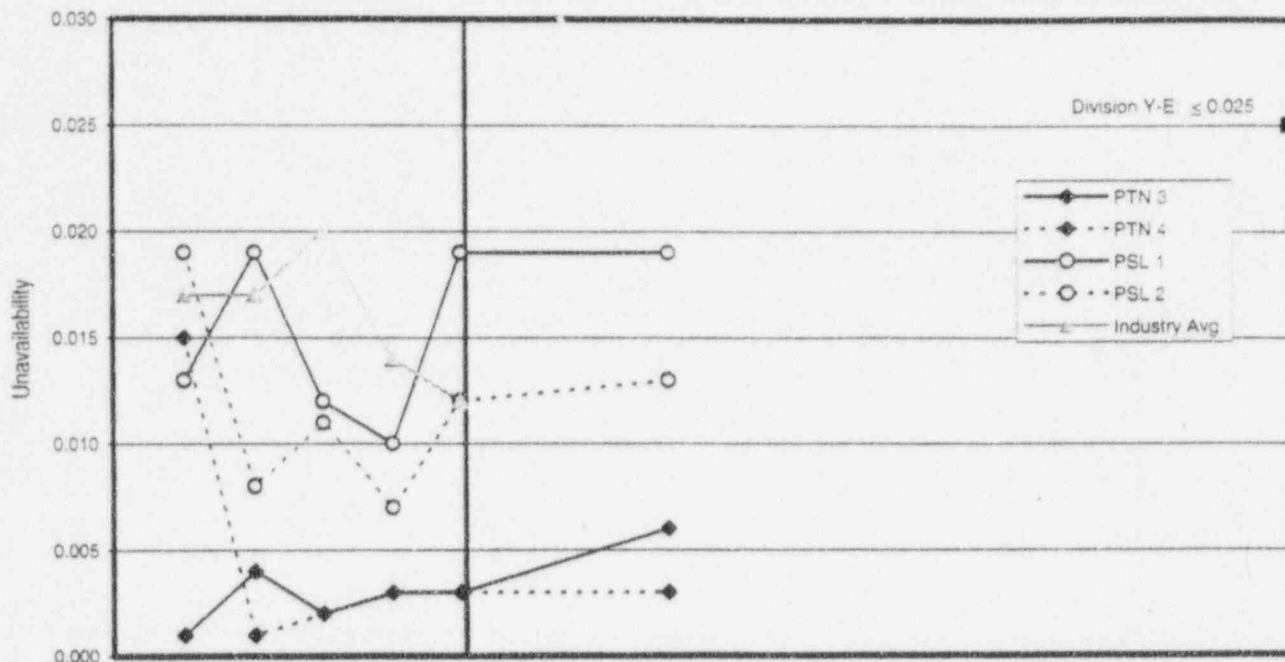
PERFORMANCE SUMMARY

- Turkey Point Units 3 & 4 Auxiliary Feedwater System performance for 3-years running through the 1st Quarter 1996 was lower than the year-end target and higher than the industry median.
- St. Lucie Units 1 & 2 Auxiliary Feedwater System performance for 3-years running through the 1st Quarter 1996 was below the year-end target and higher than the industry median.

Data Providers: (PTN) Jose Donis 246-6008/Woody Raasch 246-6527
(PSL) Chuck Wood 467-7034/Mark Wolaver 467-7083

SAFETY SYSTEM PERFORMANCE - EMERGENCY AC POWER SYSTEM

(3-year Running Average)



Unit	1991	1992	1993	1994	1995	3/96	6/96	9/96	12/96
PTN 3	0.001	0.004	0.002	0.003	0.003	0.006			
PTN 4	0.015	0.001	0.002	0.003	0.003	0.003			
PSL 1	0.013	0.019	0.012	0.010	0.019	0.019			
PSL 2	0.019	0.008	0.011	0.007	0.012	0.013			
Industry Avg.	0.017	0.017	0.020	0.014	0.012				

DEFINITION

Emergency AC Power System is defined as the sum of the emergency diesel generator unavailabilities divided by the number of emergency generators at a station. Data is collected at the train level. The emergency generator includes subsystems such as air start, lube oil, fuel oil, cooling water, etc. However, for this safety system performance indicator, unavailable hours are counted only when the emergency generator is unavailable to start or load-run. For example, if a component fails in one train of a redundant support system the emergency generator is still operable, and no unavailable hours are counted.

STATISTICAL SUMMARY

	1st Qtr 1996	3-yr Avg Ending 3/96
PTN 3A&B	0.029	0.006
PTN 4A&B	0.000	0.003
PSL 1A&B	0.006	0.019
PSL 2A&B	0.005	0.013
Targets:		
1995 Y-E Target	≤ 0.015	≤ 0.016
1996 Y-E Target	≤ 0.025	≤ 0.025

INDUSTRY PERFORMANCE

WANO	Unavailability
1993 - 1995 Median (PWR)	0.007 - 0.012
1995 Goal	0.025

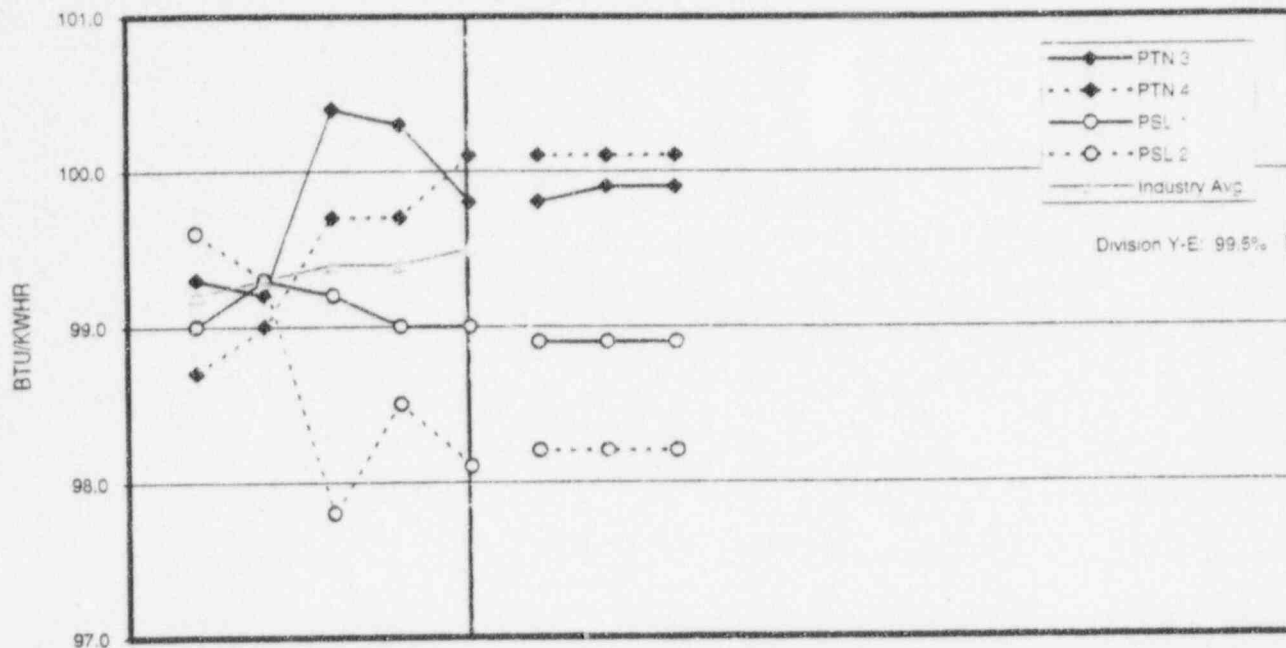
PERFORMANCE SUMMARY

- Turkey Point Units 3 & 4 Emergency AC Power performance for 3-years running ending 1st Quarter 1996 was below the year-end target and industry median.
- St. Lucie Units 1 & 2 Emergency AC Power performance for 3-years running through the 1st Quarter 1996 was below the year-end target and higher than the industry median for 3-years running.

Data Providers: (PTN) Jose Donis 246-6008/Jim Freyre 246-6539
(PSL) Chuck Wood 467-7034/Roger Kulavich 467-7080

THERMAL PERFORMANCE

(12-Month Running Average)



Unit	1991	1992	1993	1994	1995	1/96	2/96	3/96	4/96	5/96	6/96	7/96	8/96	9/96	10/96	11/96	12/96
PTN 3	99.3	99.2	100.4	100.3	99.8	99.8	99.9	99.9									
PTN 4	98.7	99.0	99.7	99.7	100.1	100.1	100.1	100.1									
PSL 1	99.0	99.3	99.2	99.0	99.0	98.9	98.9	98.9									
PSL 2	99.6	99.3	97.8	98.5	98.1	98.2	98.2	98.2									
Industry Avg	99.2	99.3	99.4	99.4	99.5												

DEFINITION

Thermal Performance is the ratio of the design gross heat rate (corrected) to the adjusted actual gross heat rate. Gross heat rate is defined as the ratio of total thermal energy produced by the reactor core to the total gross electrical energy produced by the generator during a given time period. Design gross heat rate (corrected) is the minimum theoretical heat rate that can be attained at design operating conditions for 100 percent power, expressed in British thermal units (BTUs) per kilowatt-hour(electric). Adjusted actual gross heat rate is the gross heat rate attained in the normal equipment lineup during one 24-hour period each month, expressed in BTUs per kilowatt-hour(electric) - power level should be greater than 80 percent.

STATISTICAL SUMMARY

	Mar	12-Mo. Average
PTN 3	100.0%	99.9%
PTN 4	Outage	100.1%
PSL 1	99.0%	98.9%
PSL 2	99.3%	98.2%

1996 Target (all units): 99.5%

INDUSTRY PERFORMANCE

WANO

1995 Median	99.5%
1995 Goal	99.5%

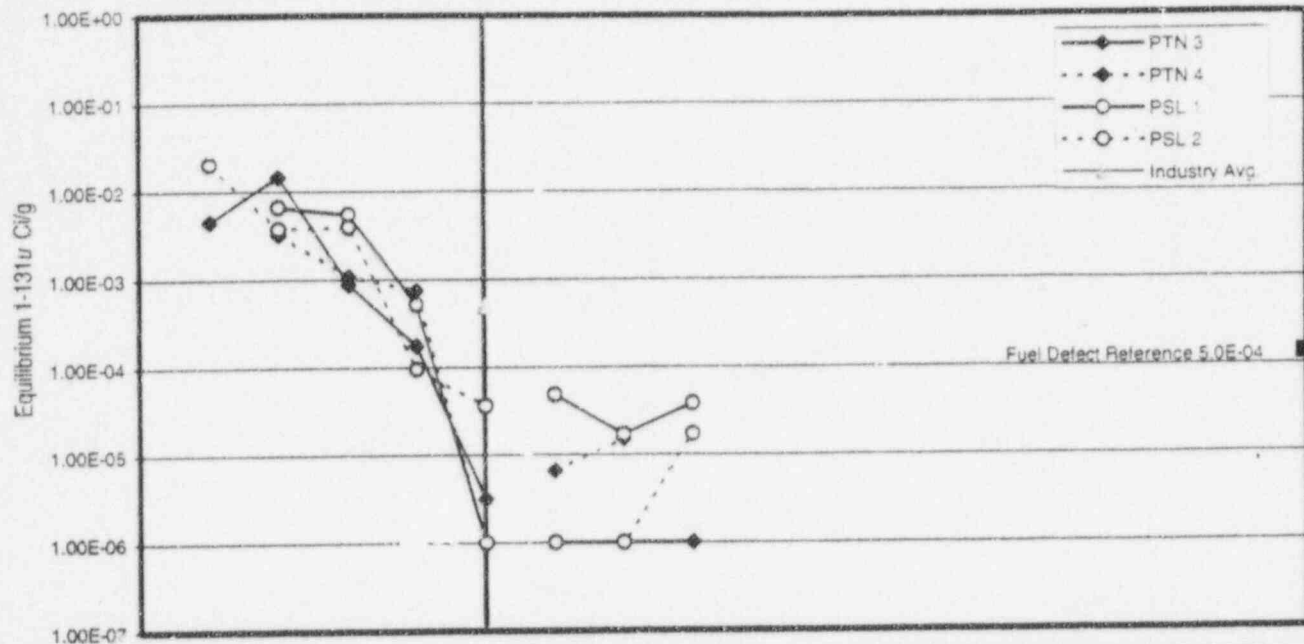
PERFORMANCE SUMMARY

- Turkey Point Unit 3 Thermal Performance for March 1996 was higher than the year-end target. For 12-mo. running average, Units 3 and 4 performance was higher than WANO's 12-mo. running industry average.
- St. Lucie Units 1 & 2 Thermal Performance for the month was below the year-end target. Performance for 12-mo. running was below WANO's 12-mo. running average.

Data Providers: (PTN) Jennifer Nicholson 246-6827 and (PSL) Marty Smit 467-7257

FUEL RELIABILITY

(Fuel Cycle/Monthly)



Unit	Previous	Cycles	1/96	2/96	3/96	4/96	5/96	6/96	7/96	8/96	9/96	10/96	11/96	12/96
PTN 3	4.48E-3	1.44E-2	8.70E-4	1.75E-4	3.16E-6	1.00E-6	1.00E-6	1.00E-6						
PTN 4		3.24E-3	1.08E-3	7.43E-4	1.00E-6	6.56E-6	1.51E-5							
PSL 1		6.54E-3	5.41E-3	5.17E-4	1.00E-6	4.82E-5	1.69E-5	3.81E-5						
PSL 2	2.01E-2	3.71E-3	3.92E-3	9.50E-5	2.62E-5	1.00E-6	1.00E-6	1.69E-5						
Industry Avg.				5.00E-4										

DEFINITION

This indicator is defined as the steady-state primary coolant iodine-131 activity (microcuries/gram) corrected for the tramp contribution and power level, and normalized to a common purification rate and average linear heat generation rate. The indicator value is calculated based on the average of the three monthly values for the most recent quarter of steady-state operation above 85 percent power. Steady state is defined as continuous operation for at least three days at a power level that does not vary more than ± 5 percent.

Note: If a calculated monthly value for a unit is less than 1.0E-6 microcuries per gram, the value is replaced by 1.0E-6 microcuries per gram.

STATISTICAL SUMMARY

	Mar	Cycle No.
PTN 3	1.00E-06	15
PTN 4	Outage	
PSL 1	3.81E-05	13
PSL 2	1.69E-05	9

Division Target for each unit: $\leq 4.50E-03$

INDUSTRY PERFORMANCE

WANO

1995 Fuel Defect Reference Threshold (PWR)	5.00 E-04
1994 Median	7.36 E-05
1994 Best Quartile	3.00 E-06

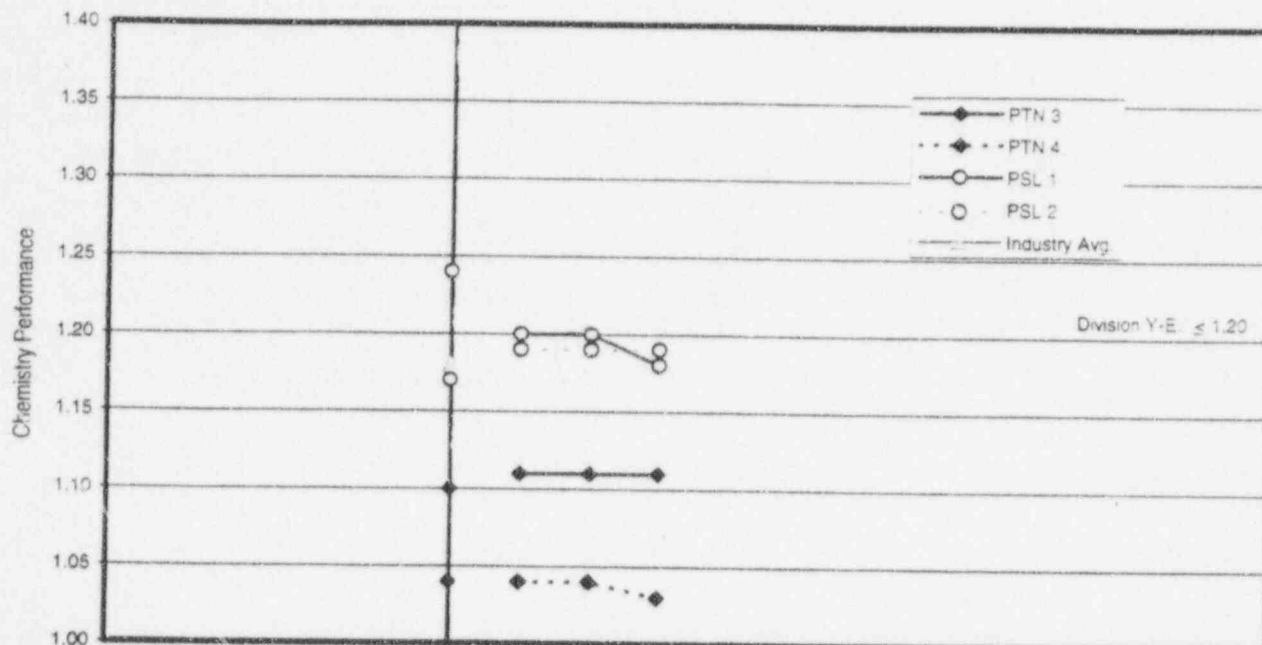
PERFORMANCE SUMMARY

- Unit 3's Fuel Reliability continues to indicate zero fuel defects in Cycle 15. A scheduled refueling outage began on March 4th.
- For Unit 1, Reactor Coolant System radioisotopic data and spiking iodine following shutdowns indicate the presence of one third-cycle failed fuel rod in the current Cycle 13.
- Unit 2 Fuel Reliability continues to indicate zero fuel defects in Cycle 9.

Data Provider: Modesto Jimenez 694-3323

CHEMISTRY INDEX

(12-Months Ending Weighted Average)



Unit	1991	1992	1993	1994	1995	1/96	2/96	3/96	4/96	5/96	6/96	7/96	8/96	9/96	10/96	11/96	12/96
PTN 3					1.10	1.11	1.11	1.11									
PTN 4					1.04	1.04	1.04	1.03									
PSL 1					1.24	1.20	1.20	1.18									
PSL 2					1.17	1.19	1.19	1.19									
Industry Avg.					1.18												

DEFINITION

The Chemistry Index compares the concentration of selected impurities to the limiting values for those impurities. Each impurity value is divided by the limiting value for the impurity, and the sum of these ratios is normalized to 1.0. The limiting values are the "achievable values" defined by international industry-accepted values.

STATISTICAL SUMMARY

	Mar	12-Mo. Ending Weighted Avg
PTN 3	1.00	1.11
PTN 4	1.11	1.03
PSL 1	1.10	1.18
PSL 2	1.11	1.19

1996 Target for each unit: ≤ 1.20

INDUSTRY PERFORMANCE

WANO

(PWR's with Recirculating Steam Generators without Molar Ratio Control)

1995 Median	1.18
1995 Lowest Chemistry Index Value Attainable	1.00

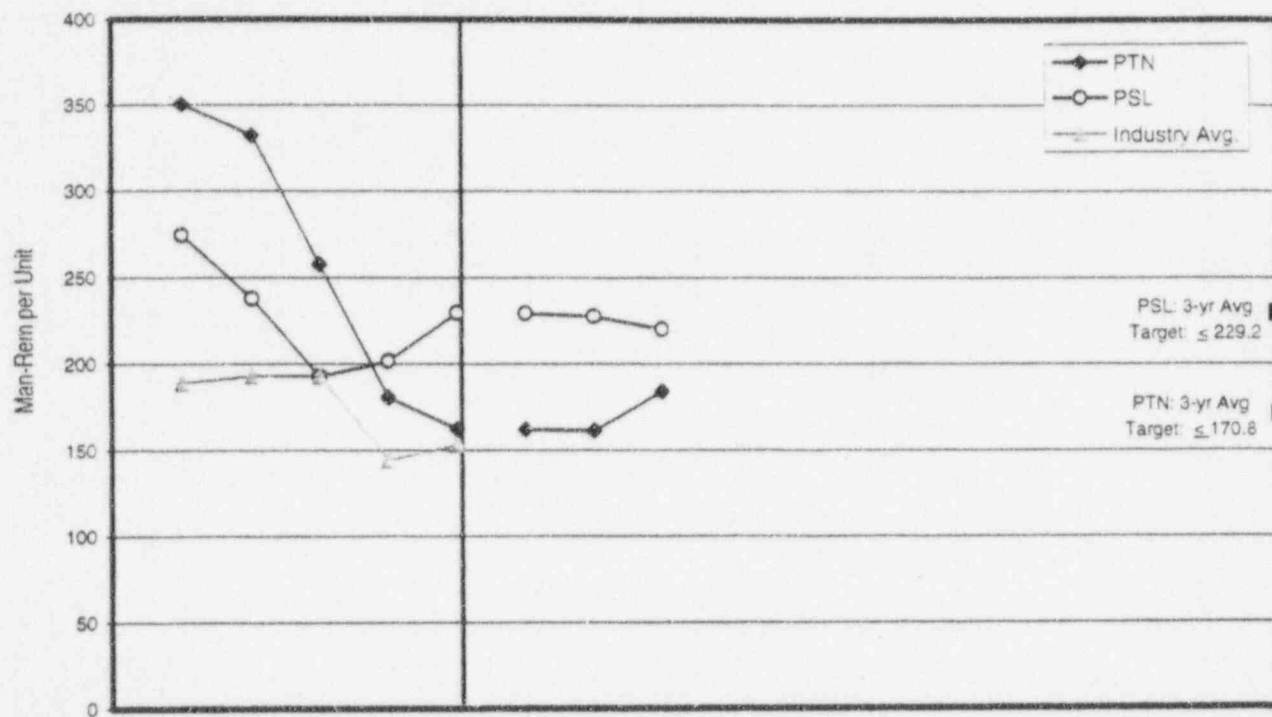
PERFORMANCE SUMMARY

- Turkey Point Units 3 & 4 and St. Lucie Unit 1's Chemistry Index performance for 12-mo. running was below the year-end target of ≤ 1.20 and the industry median.
- St. Lucie Unit 2's performance was below the year-end target and slightly higher than the industry median.

Data Providers: (PTN) R. Steinke 246-6118/Randy Woodard 246-6810
(PSL) R. Frechette 465-3213/Dave Faulkner 465-3393

RADIATION EXPOSURE

(3-year Running Average)



Plant	1991	1992	1993	1994	1995	1/96	2/96	3/96	4/96	5/96	6/96	7/96	8/96	9/96	10/96	11/96	12/96
PTN	350	332	257	181	162	162	161	184									
PSL	274	238	193	202	230	229	227	220									
Industry Avg	189	193	193	144	153												

DEFINITION

Collective Radiation Exposure is the total external whole-body dose received by all personnel (including contractors and visitors) coming on site during a time period, as measured by the primary dosimeter, thermoluminescent dosimeter (TLD) or film badge. Exposure measured by direct reading dosimeters should be included only for those periods or situations when more accurate data is not available to the utility from TLD's or film badges. In order to correlate this indicator with the new 10 CFR 20 reporting guidelines, U.S. utilities report deep dose equivalent (DDE) and the total effective dose equivalent (TEDE).

STATISTICAL SUMMARY

3-yr Running per Unit	
PTN	184.0*
PSL	219.6*
Division Targets:	3-yr. Avg
PTN: Y-E thru 1996	≤ 170.8
PSL: Y-E thru 1996	≤ 229.2

* March data is DRD; TLD was not available at time of publication.

INDUSTRY PERFORMANCE

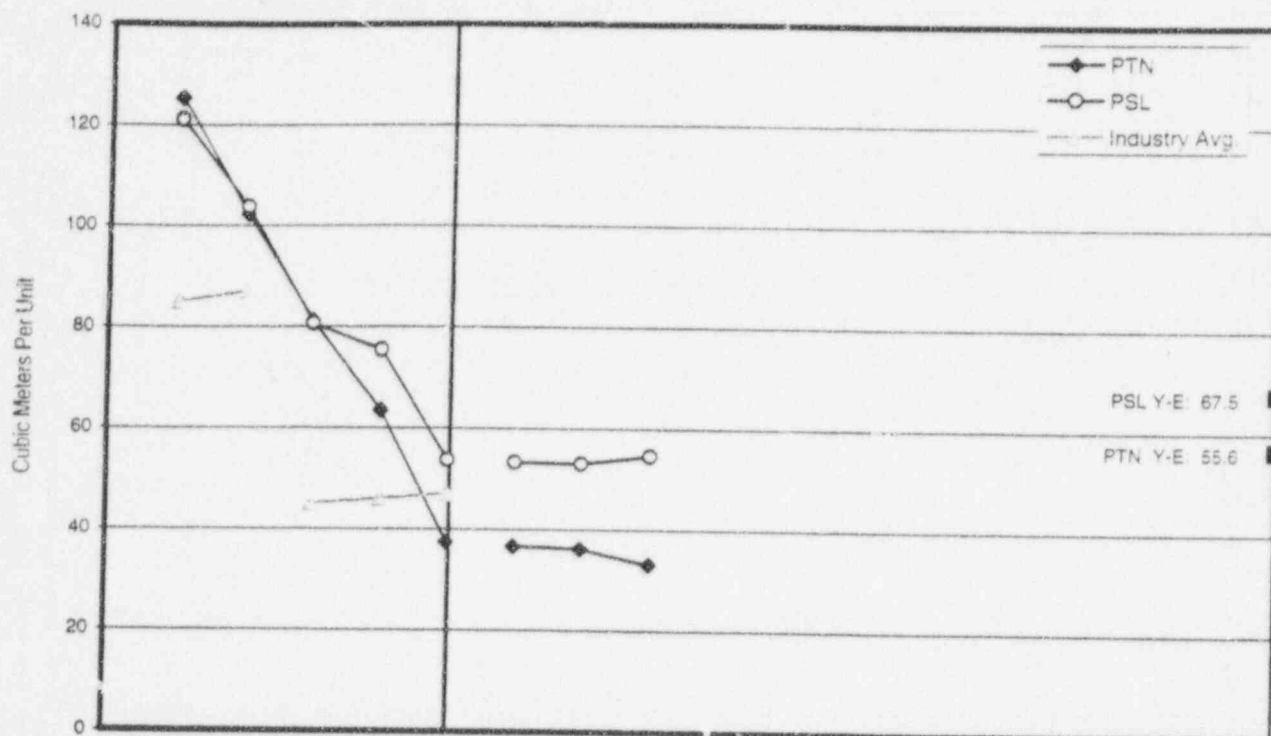
WANO	Man-Rem (per Unit)
(PWR's) Median 3-yr Distribution (1993 - 1995)	163
1995 Median	153
1995 Goal	185

PERFORMANCE SUMMARY

- Turkey Point Collective Radiation Exposure for 3-years running through March 1996 was 184.0; performance was higher than WANO's 3-yr. running median of 163.
- St. Lucie's Collective Radiation Exposure for 3-years running through March 1996 of 219.6 was higher than WANO's 3-yr. running median.

LOW-LEVEL SOLID RADIOACTIVE WASTE

(Cubic Meters per unit per year, 3-year Running Average)



Plant	1991	1992	1993	1994	1995	1/96	2/96	3/96	4/96	5/96	6/96	7/96	8/96	9/96	10/96	11/96	12/96
PTN	125.3	102.3	81.2	63.4	37.4	36.7	36.2	33.1									
PSL	121.1	103.8	80.7	75.4	53.7	53.3	53.0	54.6									
Industry Avg	85.0	87.0	45.0	46.0	47.0												

DEFINITION

This indicator is defined as the volume of low-level solid radioactive waste that has been processed and is in final form (for example, compacted or solidified) ready* for disposal during a given period. It is calculated using the amount of waste in final form, including the container, actually shipped for disposal from both on-site and off-site facilities, plus the change in inventory of final-form waste in storage at both on-site and off-site facilities.

* in transit to or at disposal site for disposal

STATISTICAL SUMMARY

3-yr Running thru Mar (per unit)

PTN 33.1
PSL 54.6

Division Targets:

PTN 1996 Y-E 55.6 cu. mtr. per unit
PSL 1996 Y-E 67.5 cu. mtr. per unit

INDUSTRY PERFORMANCE

WANO

Monthly Cu. Ft.
(2 Unit Site)

3-yr Distribution Median (1993 - 1995)
1995 Median
1995 Goal

47.0
30.0
110.0

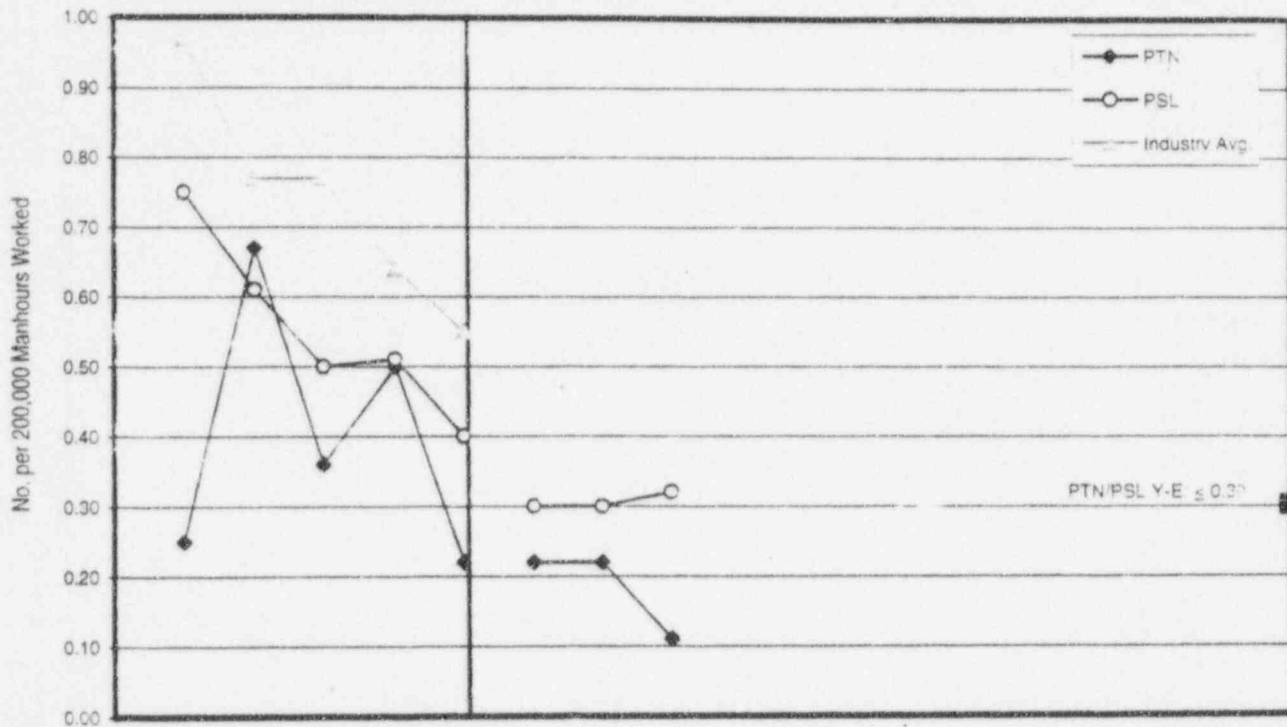
PERFORMANCE SUMMARY

- Turkey Point's Solid Waste Disposal performance for 3-yrs. running was below the year-end target and WANO's 3-yr. industry median.
- St. Lucie's performance was below the year-end target but higher than WANO's 3-yr. industry median.

Data Providers: (PTN) Bob Schuber 246-7227 and (PSL) Bruce Somers 467-7305

INDUSTRIAL SAFETY ACCIDENT RATE

(12-Month Running Average)



Plant	1991	1992	1993	1994	1995	1/96	2/96	3/96	4/96	5/96	6/96	7/96	8/96	9/96	10/96	11/96	12/96
PTN	0.25	0.67	0.36	0.50	0.22	0.22	0.22	0.11									
PSL	0.75	0.61	0.50	0.51	0.40	0.30	0.30	0.32									
Industry Avg.	0.97	0.77	0.77	0.64	0.55												

DEFINITION

Industrial Safety Accident Rate is defined as the number of accidents per 200,000 man-hours worked for all utility personnel permanently assigned to the station that result in any of the following: (1) one or more days of restricted work (excluding the day of the accident); (2) one or more days away from work (excluding the day of the accident); and, (3) fatalities. Contractor personnel are not included for this indicator.

$$\text{Industrial Safety Accident Rate} = \frac{(\text{number of restricted-time} + \text{lost-time accidents} + \text{fatalities}) \times 200,000}{(\text{number of station man-hours worked})}$$

STATISTICAL SUMMARY

12-Mo.-Ending (Mar)

PTN	0.11
PSL	0.32
Division Targets:	
PTN 1996 Year-End	0.30
PSL 1996 Year-End	0.30

INDUSTRY PERFORMANCE

WANO

1995 Median	0.55
1995 Goal	0.50

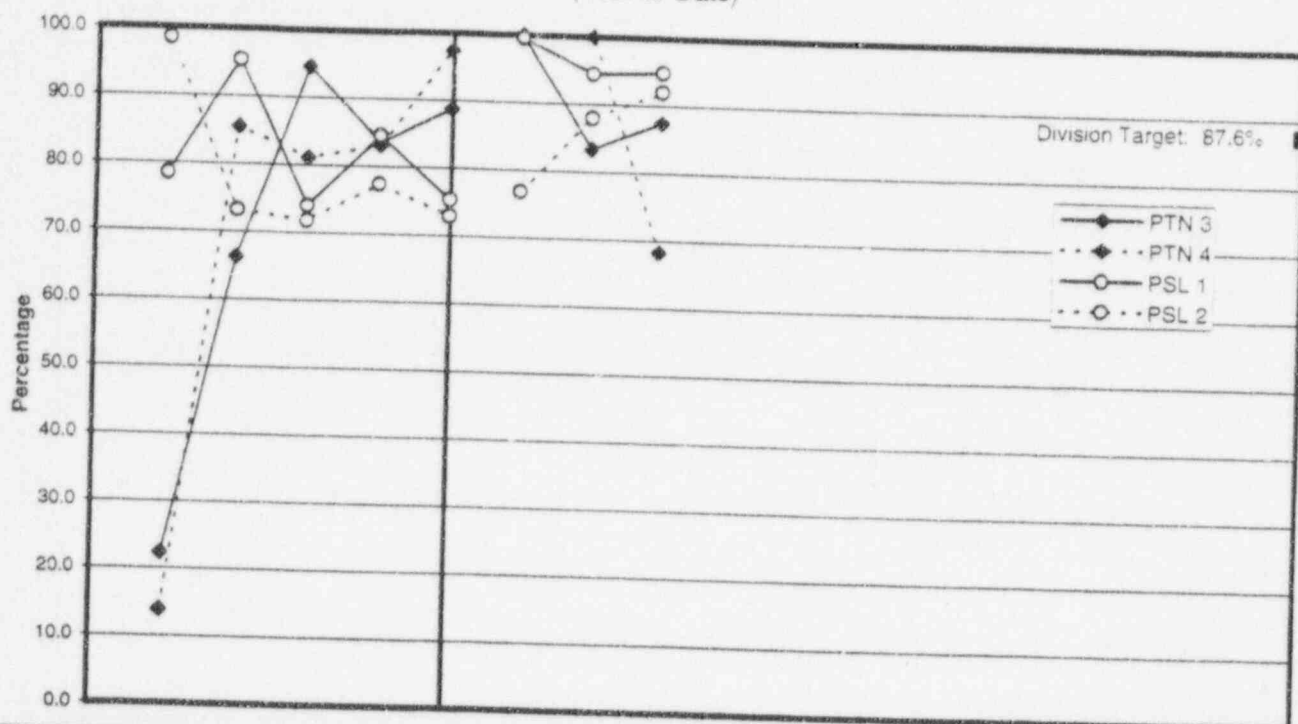
PERFORMANCE SUMMARY

- Turkey Point's Industrial Safety Accident Rate for 12-mo. running through March 1996 was below the year-end target and industry median.
- St. Lucie's rate of 0.32 for 12-mo. running through March 1996 was higher than the year-end target but below the industry median.

Data Provider: W. Korte 694-4235

EQUIVALENT AVAILABILITY FACTOR

(Year-to-Date)



Unit	1991	1992	1993	1994	1995	1/96	2/96	3/96	4/96	5/96	6/96	7/96	8/96	9/96	10/96	11/96	12/96
PTN 3	22.3	66.3	94.6	83.5	88.7	99.9	83.3	87.4									
PTN 4	13.9	85.5	81.1	83.1	97.4	100.0	100.0	68.3									
PSL 1	78.5	95.4	74.0	84.5	75.3	99.6	94.5	94.9									
PSL 2	98.5	73.2	71.8	77.4	72.9	76.8	87.9	92.0									

DEFINITION

Equivalent Availability Factor (EAF) is the ratio of the actual energy production capability to the energy that would be produced operating at full power for the same period expressed as a percent. Equivalent Availability provides an indication of the effectiveness of plant programs and practices in maximizing electrical generation.

$$\text{EAF}\% = \frac{\text{Available Hours} - (\text{Equivalent Unit Derated Hours})}{\text{Period Hours}} \times 100\%$$

STATISTICAL SUMMARY

	Mar	Y-T-D	Y-T-D Target	3-yr Run. Avg
PTN 3	95.4%	87.4%	95.0%	88.7%
PTN 4	6.8%	68.3%	65.8%	84.6%
PSL 1	95.8%	94.9%	94.8%	77.9%
PSL 2	100.0%	92.0%	95.0%	80.5%
Division	74.5%	85.6%	87.6%	82.9%

1996 Division Y-E Target: 87.6%
1996 Year-End Forecast: 87.1%

INDUSTRY PERFORMANCE

NERC/GADS	
1994 (PWR's)	77.8%
1994 (All types)	74.1%
1990-1994 (PWR's)	74.4%
1990 - 1994 (All types)	71.2%
WANO	
1995 Median	80.7%
1995 Average	77.4%

PERFORMANCE SUMMARY

Major contributors to March 1996 Equivalent Availability loss for the Division was as follows:

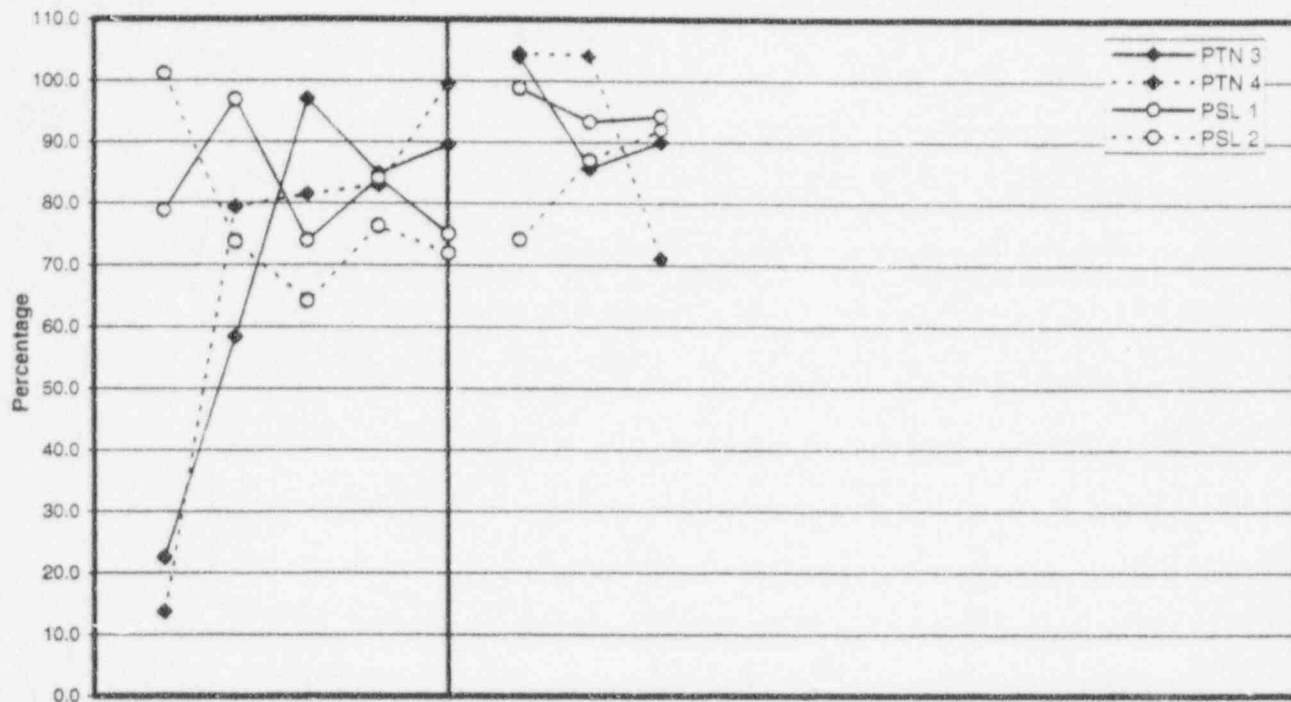
- PTN Unit 3: Manual reactor trip due to the loss of the 3B Main Feedwater Pump (34.5 hours).
- PTN Unit 4: Scheduled refueling outage beginning March 3rd and continuing through the month.
- PSL Unit 1: Waterbox cleanings (30.9 hours).

For the Division, Y-T-D Equivalent Availability was 85.6% which is below the 87.6% targeted through the period.

Data Provider: Sharon Bilger 694-4255

CAPACITY FACTOR (MDC NET)

(Year-to-Date)



Unit	1991	1992	1993	1994	1995	1/96	2/96	3/96	4/96	5/96	6/96	7/96	8/96	9/96	10/96	11/96	12/96
PTN 3	22.5	58.4	97.0	84.8	89.5	103.9	85.7	90.0									
PTN 4	13.7	79.3	81.4	83.0	99.5	104.5	104.2	71.0									
PSL 1	78.8	96.9	73.9	84.1	75.0	98.8	93.3	94.2									
PSL 2	101.1	73.7	64.1	76.3	71.9	74.1	87.0	92.0									

DEFINITION

Capacity Factor (CF) is the index of the actual electrical energy produced by the unit with respect to its potential.

$$\text{Capacity Factor} = \frac{\text{Net Electrical Generation}}{\text{Maximum Dependable Capacity (839 or 666) X Period Hours}} \times 100$$

STATISTICAL SUMMARY

	Mar	Y-T-D	12-mo. ending
PTN 3	98.3%	90.0%	86.4%
PTN 4	6.6%	71.0%	93.3%
PSL 1	96.0%	94.2%	76.1%
PSL 2	101.7%	92.0%	70.9%
Division	75.7%	86.8%	81.7%

INDUSTRY PERFORMANCE

NERC/GADS

1994 Avg PWR's	76.7%
1994 Avg All Types	72.8%
1990-1994 PWR's	72.8%
1990-1994 All Types	69.5%

WANO

1995 Median	81.1%
1995 Average	77.9%

PERFORMANCE SUMMARY

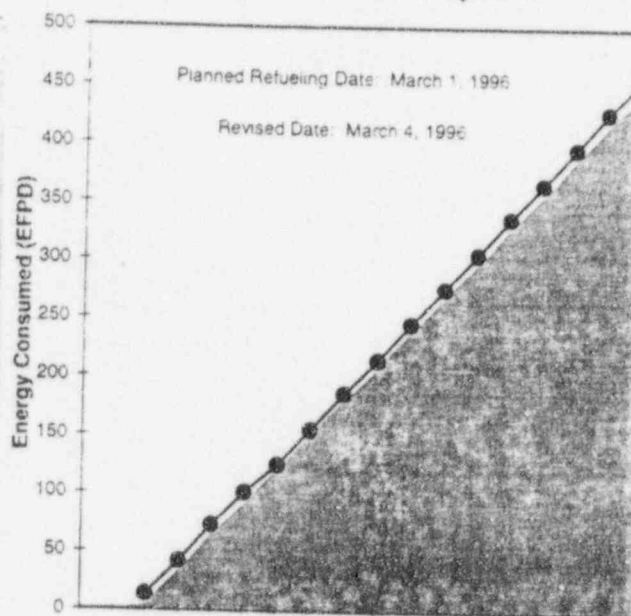
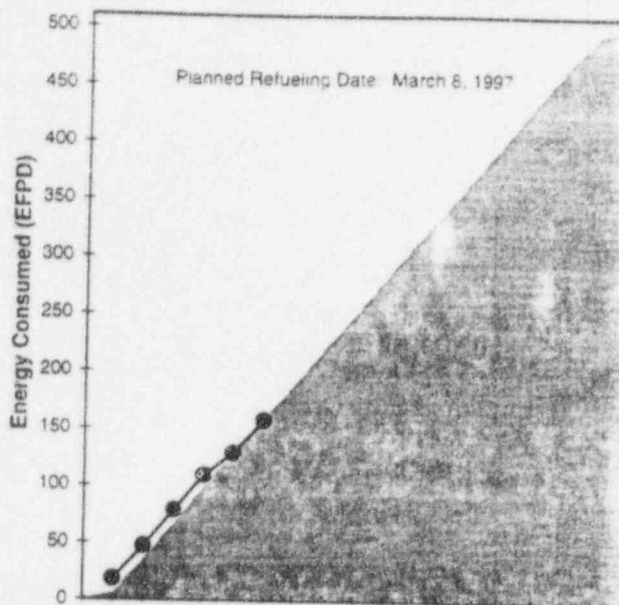
Major contributors to March 1996 Capacity Factor loss for the Division was as follows:

- PTN Unit 3: Manual reactor trip due to the loss of the 3B Main feedwater Pump.
- PTN Unit 4: Scheduled refueling outage beginning March 3rd and continuing through the month.
- PSL Unit 1: Waterbox cleanings

PTN FUEL UTILIZATION FACTOR

Turkey Point Unit #3, Cycle 15

Turkey Point Unit #4, Cycle 15



PTN #3	Oct	Nov	Dec	Jan	Feb	Mar	Apr	May	Jun	Jul	Aug	Sep	Oct	Nov	Dec	Jan	Feb	Mar
Planned	5	35	66	98	127	158	188	219	249	281	312	342	373	403	435	466	494	501
Actual	18	47	78	109	128	157												
Variance (+/-)	13	12	12	11	1	-1												
PTN #4	Oct	Nov	Dec	Jan	Feb	Mar	Apr	May	Jun	Jul	Aug	Sep	Oct	Nov	Dec	Jan	Feb	Mar
Planned		4	34	64	91	121	150	180	209	239	269	298	328	357	387	417	445	446
Actual		13	42	73	101	125	155	186	215	246	276	306	337	366	397	428	457	459
Variance (+/-)		9	8	9	10	4	5	6	6	7	7	8	9	9	10	11	12	13

DEFINITION

Fuel utilization plots the amount of nuclear energy used during the current fuel cycle. The amount of nuclear energy loaded into the core is expressed in effective full power days (EFPD). One EFPD is the equivalent of operating the reactor at maximum thermal rating (2200 at PTN or 2700 at PSL) for a 24 hour period. Planned energy is compared to actual energy used during the cycle. Fuel utilization is directly related to plant performance. The significance of variance EFPD(+/-) is the difference between planned and actual energy consumption. Fuel utilization can be used to project longer or shorter operating fuel cycles.

$$\text{Actual Energy / Calendar Days} = \text{Fuel Utilization Factor}$$

Fuel Cycle Operating Assumptions

PTN 3: In accordance with the April 26, 1995 Approved Operating Schedule (AOS), Unit 3, Cycle 15 is scheduled to run 498 calendar days (October 27, 1995 to March 8, 1997) with design energy to run 501 (EFPD) days. **NOTE:** Due to the 34-day outage, startup began on October 7, 1995, 20 days earlier than planned.

PTN 4: Unit 4, Cycle 15 ended on March 3, 1996.

PERFORMANCE SUMMARY

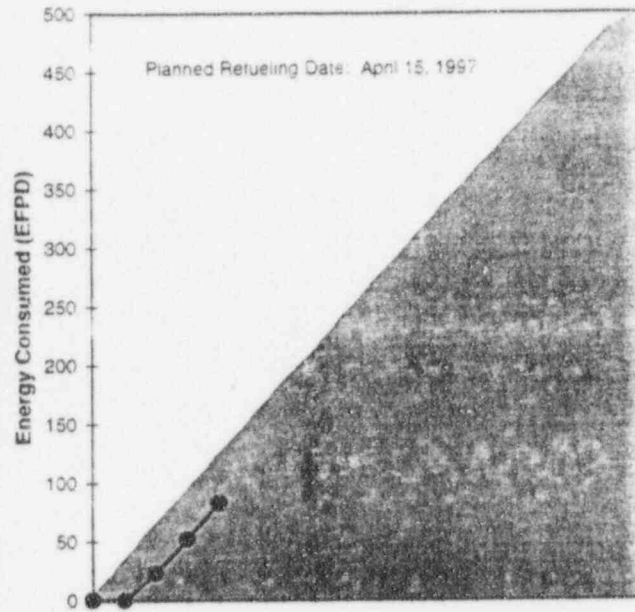
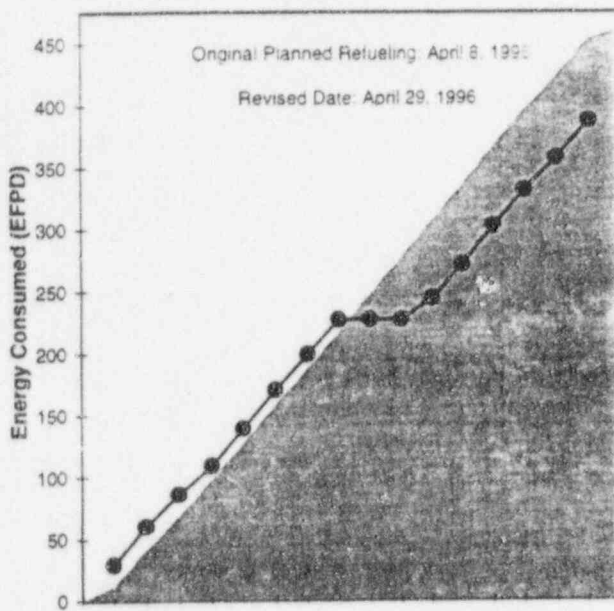
- Unit 3 achieved a Fuel Utilization Factor of 93.6% and operated for 697 effective full power hours in the month of March. For the period October 27, 1995 to March 31, 1996, the Fuel Utilization Factor was 100.0%.
- Unit 4 operated for 51 effective full power hours in the month of March. For the period of November 27, 1994 to March 31, 1996, the Fuel Utilization Factor was 103.0%.

Data provider: Jimmie Perryman 694-3330

PSL FUEL UTILIZATION FACTOR

St. Lucie Unit #1, Cycle 13

St. Lucie Unit #2, Cycle 9



PSL #1	Nov	Dec	Jan	Feb	Mar	Apr	May	Jun	Jul	Aug	Sep	Oct	Nov	Dec	Jan	Feb	Mar	Apr
Planned		11	41	68	98	127	158	187	217	247	276	306	335	366	396	423	453	461
Actual		29	60	86	109	139	170	199	227	227	227	244	271	302	331	357	387	
Variance (+/-)		18	19	18	11	12	12	12	10	-20	-49	-62	-64	-64	-65	-66	-66	
PSL #2	Nov	Dec	Jan	Feb	Mar	Apr	May	Jun	Jul	Aug	Sep	Oct	Nov	Dec	Jan	Feb	Mar	Apr
Planned	5	36	67	95	126	155	186	215	247	276	306	336	366	397	427	455	485	500
Actual	0	0	23	52	83													
Variance (+/-)	-5	-36	-44	-43	-43													

DEFINITION

Fuel utilization plots the amount of nuclear energy used during the current fuel cycle. The amount of nuclear energy loaded into the core is expressed in effective full power days (EFPD). One EFPD is the equivalent of operating the reactor at maximum thermal rating (2200 at PTN or 2700 at PSL) for a 24 hour period. Planned energy is compared to actual energy used during the cycle. Fuel utilization is directly related to plant performance. The significance of variance EFPD(+/-) is the difference between planned and actual energy consumption. Fuel utilization can be used to project longer or shorter operating fuel cycles.

$$\text{Actual Energy / Calendar Days} = \text{Fuel Utilization Factor}$$

Fuel Cycle Operating Assumptions

PSL 1: In accordance with the October 28, 1994 Approved Operating Schedule (AOS), Unit 1, Cycle 13 was scheduled to begin operation December 20, 1994. This provided for a cycle of 475 calendar days with design energy to run 461 effective full power days (EFPD). Unit 1, Cycle 13 actually began operation November 29, 1994 and is currently scheduled to refuel April 29, 1996.
NOTE: Per the revised AOS dated February 8, 1996, the unit will shutdown 21 days later than previously scheduled.

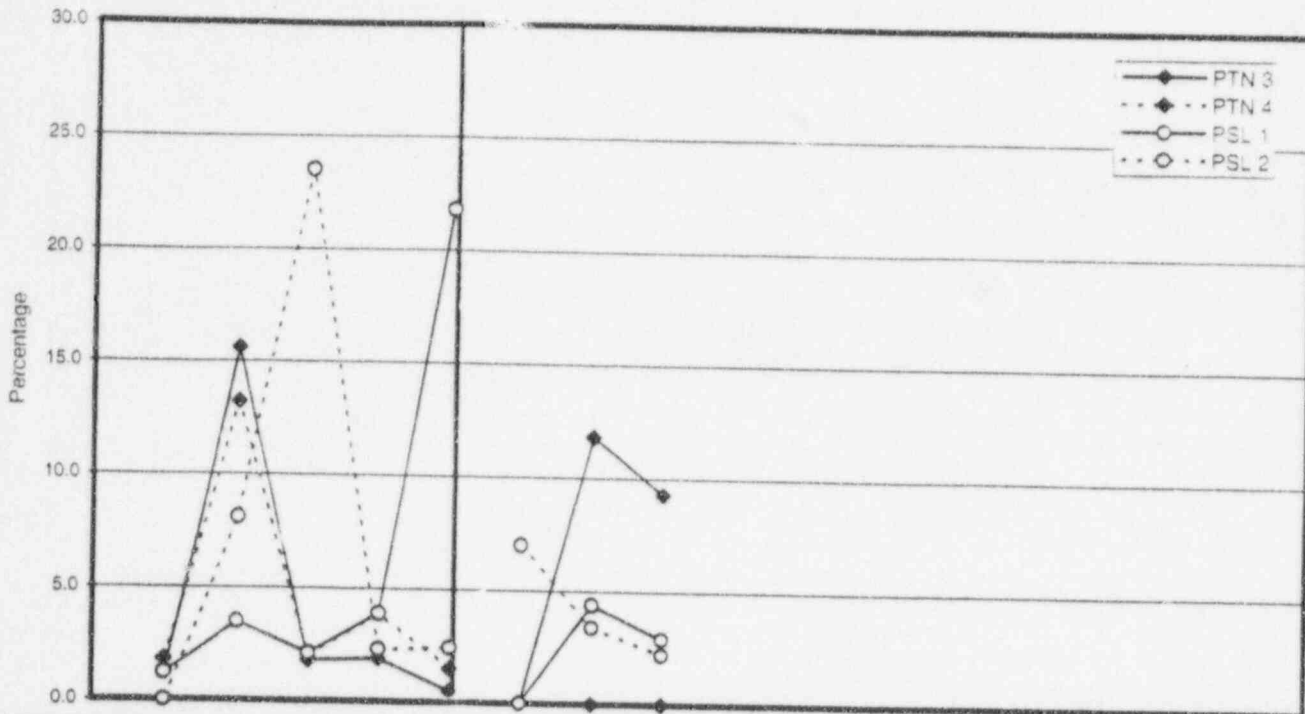
PSL 2: In accordance with the April 26, 1995 AOS, Unit 2, Cycle 9 was scheduled to begin operation November 24, 1995. This provided for a cycle of 508 calendar days with design energy to run 500 EFPD. Unit 2, Cycle 9 actually began operation January 5, 1996 and is currently scheduled to refuel April 15, 1997.

PERFORMANCE SUMMARY

- Unit 1 achieved a Fuel Utilization Factor of 95.8% and operated at 713 effective full power hours in the month of March. For the period of December 20, 1994 through March 31, 1996, the Fuel Utilization Factor was 83.2%.
- Unit 2 achieved a Fuel Utilization Factor of 99.4% and operated at 740 effective full power hours in the month of March. For the period of November 24, 1995 through March 31, 1996, the Fuel Utilization Factor was 64.5%.

FORCED OUTAGE RATE

(Year-to-Date)



Unit	1991	1992	1993	1994	1995	1/96	2/96	3/96	4/96	5/96	6/96	7/96	8/96	9/96	10/96	11/96	12/96
PTN 3	1.2	15.6	1.8	1.9	0.5	0.0	11.8	9.3									
PTN 4	1.8	13.2	2.0	3.9	1.5	0.0	0.0	0.0									
PSL 1	1.2	3.5	2.1	3.9	21.8	0.0	4.4	2.9									
PSL 2	0.0	8.1	23.5	2.3	2.4	0.0	3.4	2.2									

DEFINITION

Forced Outage Rate is the percentage of time that the unit was unavailable due to forced events compared to the time planned for full power operation. A forced outage exists when the unit requires immediate removal from service, i.e. the unit is not synchronized to the grid. This type of outage usually results from immediate mechanical/electrical/hydraulic controls systems and operator-initiated trips in response to unit alarms.

$$\text{Forced Outage Rate} = \frac{\text{Forced Outage Hours}}{\text{Forced Outage Hours} + \text{Service Hours}} \times 100\%$$

STATISTICAL SUMMARY

	Mar	Y-T-D	12 Mo. Ending
PTN 3	4.6%	9.3%	3.1%
PTN 4	0.0%	0.0%	1.5%
PSL 1	0.0%	2.9%	22.0%
PSL 2	0.0%	2.2%	2.0%

INDUSTRY PERFORMANCE

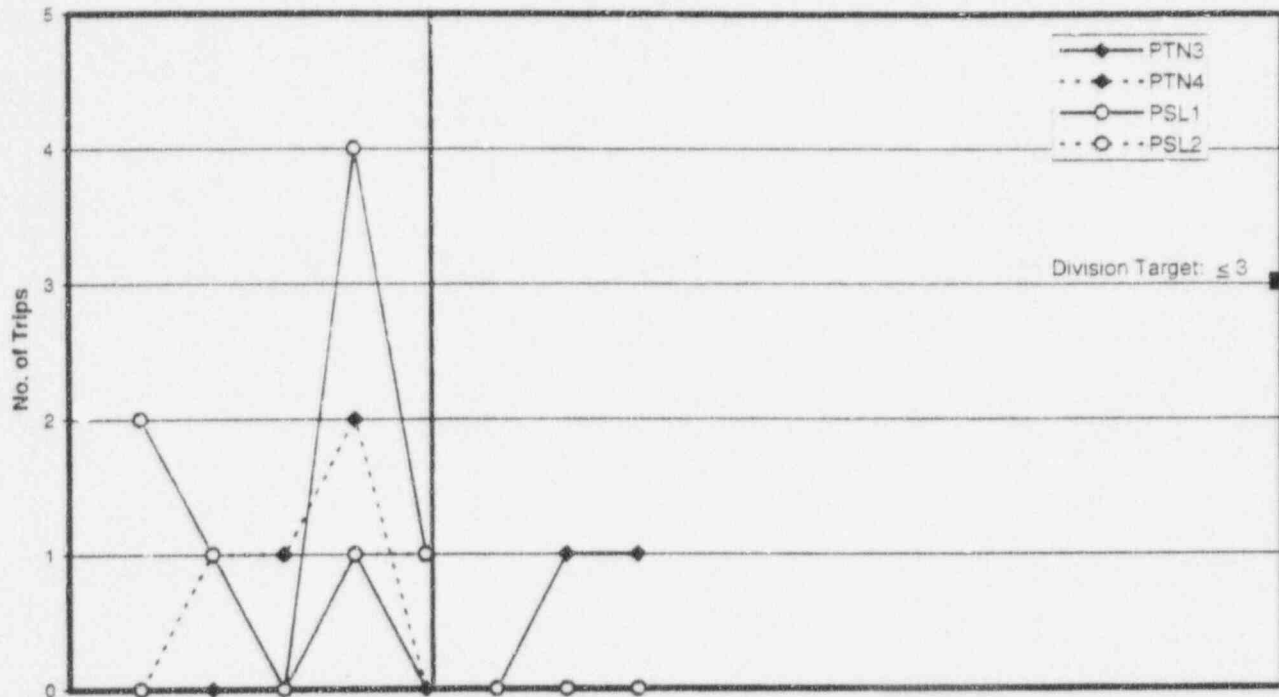
NERC/GADS	
1994 (PWR's)	8.1%
1994 (All types)	10.6%
1990-1994 (PWR's)	7.6%
1990 - 1994 (All types)	10.8%

PERFORMANCE SUMMARY

- Turkey Point Unit 3 Forced Outage Rate of 4.6% was the result of a manual reactor trip on March 27th due to the loss of the 3B Main Feedwater Pump.

UNPLANNED AUTOMATIC TRIPS WHILE CRITICAL

(Year-to-Date)



Unit	1991	1992	1993	1994	1995	1/96	2/96	3/96	4/96	5/96	6/96	7/96	8/96	9/96	10/96	11/96	12/96
PTN3	0	0	0	1	0	0	1	1									
PTN4	0	1	1	2	0	0	0	0									
PSL1	2	1	0	4	1	0	0	0									
PSL2	0	1	0	1	1	0	0	0									

DEFINITION

An Unplanned Automatic Scram is a non-manual actuation of the reactor protection system that results in a scram signal any time the unit is critical. Scrams that are planned as part of special evaluations or tests are not included in this definition. This indicator provides an indication of success in improving plant safety by reducing the number of undesirable and unplanned thermal-hydraulic and reactivity transients requiring reactor scrams.

STATISTICAL SUMMARY

	Mar	Y-T-D	12-Month Ending
PTN 3	0	1	1
PTN 4	0	0	0
PSL 1	0	0	1
PSL 2	0	0	0
Division Total:	0	1	2

1996 Division Target: ≤ 3

INDUSTRY PERFORMANCE

WANO	Trips per unit
Trips per 7000 Critical Hours	
3-yr Distribution Median (7/92-6/95)	1.0
1994 Median	0.8
1995 Goal	1.0
NRC (1995 3rd Qtr Performance Indicator Rpt)	
Quarterly Trips Annualized	1.0
Quarterly Trips per 7000 Critical Hours	1.2

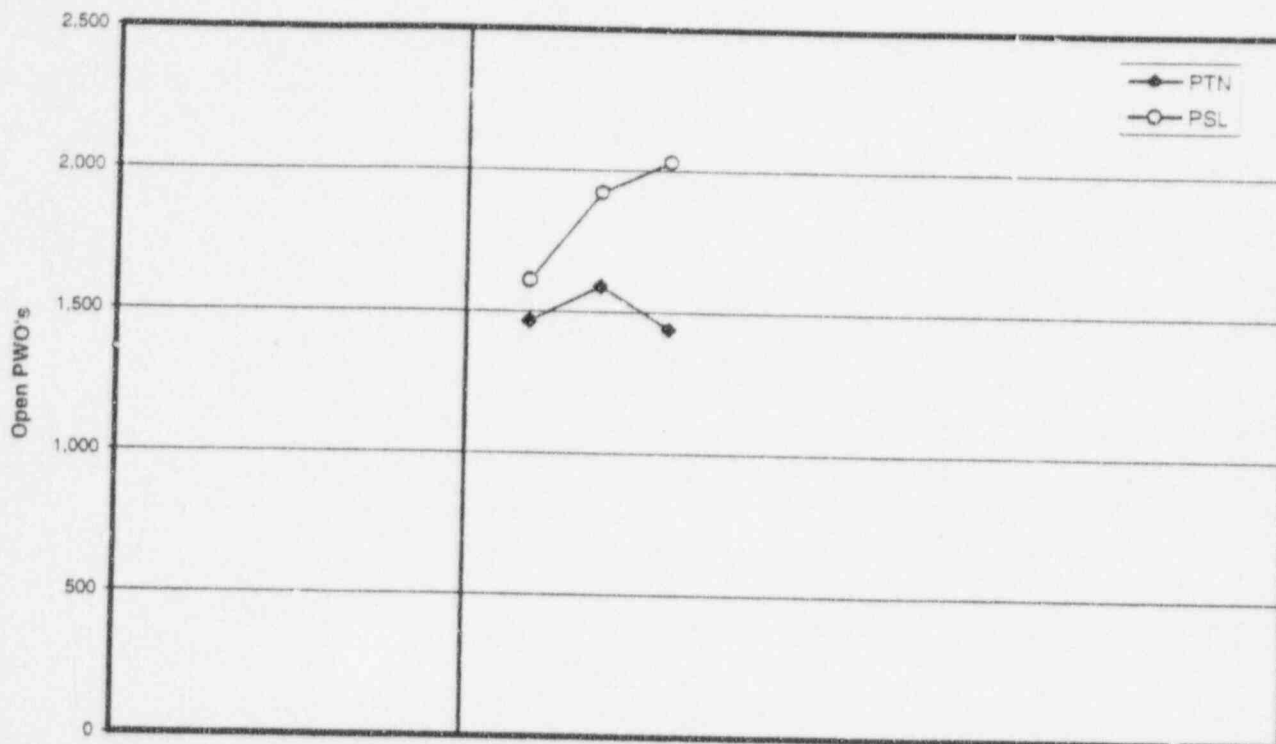
PERFORMANCE SUMMARY

No automatic trips occurred in March 1996. Year-to-date, one automatic trip has been experienced.

- Turkey Point Unit 3 experienced an automatic trip on February 9th; the "B" Steam Generator Feed Pump was stopped to monitor its discharge check valve closing stroke which did not strike closed as expected. The resulting feed flow transient caused the "C" Steam Generator level to increase resulting in a turbine trip which tripped the reactor.

Data Providers: (PTN) J. Knorr 246-6757 (PSL) K. Korth 467-7054

OPEN PWO'S



Unit	1991	1992	1993	1994	1995	1/96	2/96	3/96	4/96	5/96	6/96	7/96	8/96	9/96	10/96	11/96	12/96
PTN	*	*	*	*	*	1,466	1,589	1,441									
PSL	*	*	*	*	*	1,608	1,920	2,030									

DEFINITION

This indicator includes Work Type 1 (Planned Miscellaneous), Work Type 3 (Projects), WorkType 5 (Trouble & Breakdown) and includes all hold codes, status 22 through 48.

* Historic data not meaningful. Prior to 1996, each site was counting Open PWO's differently. St. Lucie was counting only Work Type 5 non-outage corrective and Turkey Point was counting Work Types 1 and 5. Each site now agrees to report per the above definition.

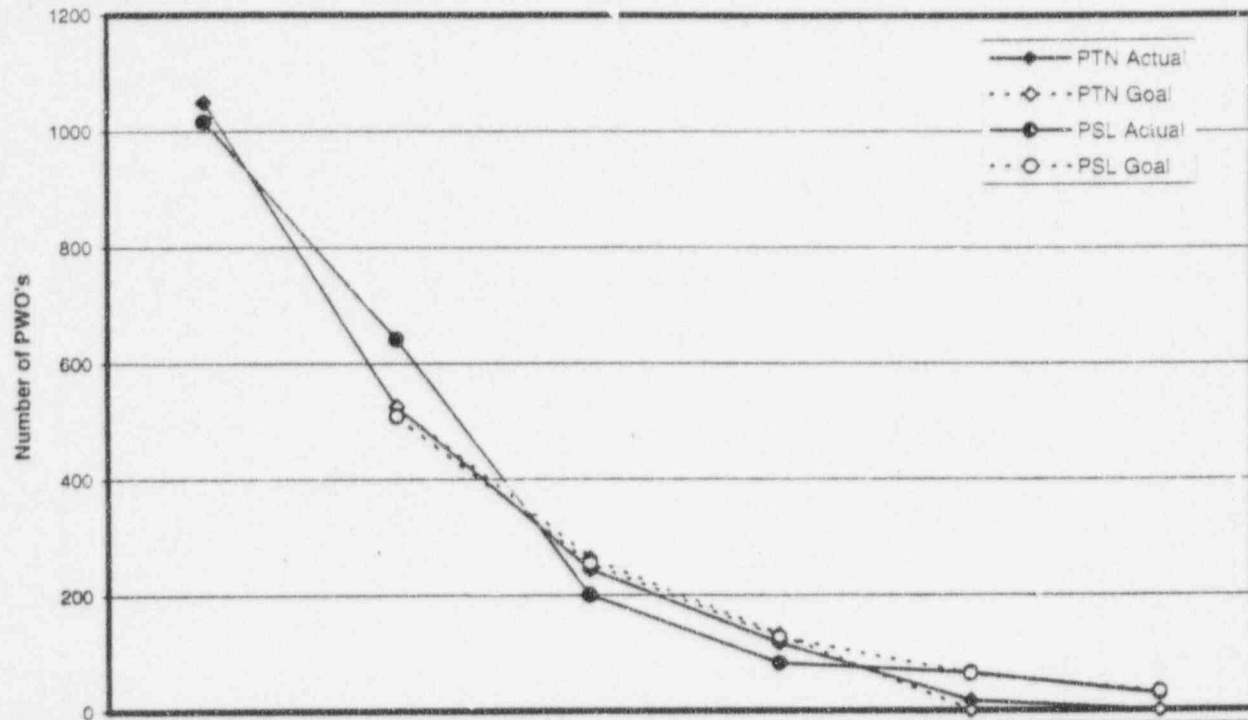
STATISTICAL SUMMARY

INDUSTRY PERFORMANCE

PERFORMANCE SUMMARY

Data Providers: (PTN) Greg Heisterman 246-6796 and (PSL) Joe Marchese 467-7107

PWO AGING CURVE



Unit	TOTALS	3	6	9	12	Over 12
PTN Actual	1050	524	244	117	17	0
PTN Goal		525	263	132	0	0
PSL Actual	1016	641	200	82	65	29
PSL Goal		508	254	127	64	32

DEFINITION

PWO Aging curves includes Work Type 1 (Planned Misc) Work Type 3 (Projects) and Work Type 5 (Trouble & Breakdown), and excludes short notice outages (SNO), HC2-Startup, HC3-Hot Standby, HC4-Hot Shutdown, HC5-Cold Shutdown, and HC6-Refueling. This only includes status 22 through 48.

Goal: To have the backlog every three months.

STATISTICAL SUMMARY

INDUSTRY PERFORMANCE

PERFORMANCE SUMMARY

Data Providers: (PTN) Greg Heisterman 246-6796 and (PSL) Joe Marchese 467-7107

PWO'S GREATER THAN 12 MONTHS (GOLDEN OLDIES)

TURKEY POINT

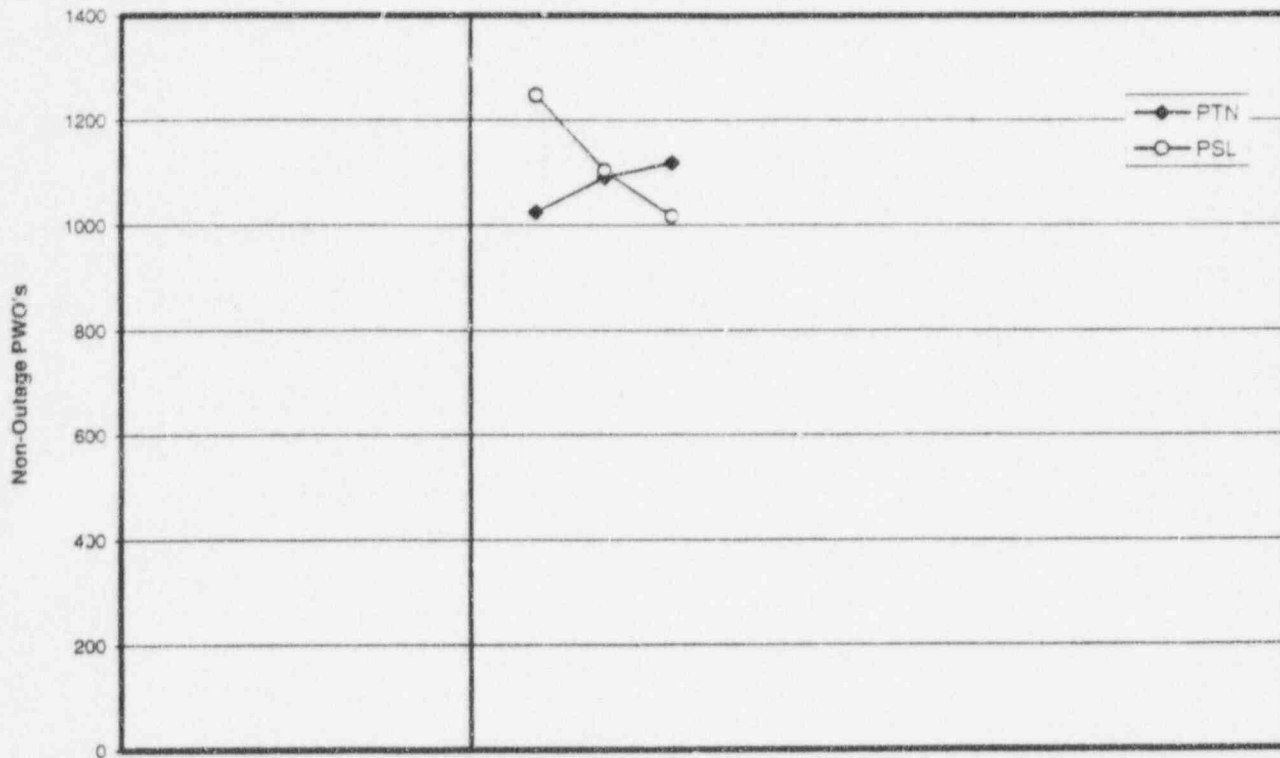
REF	PWO #	PWO TITLE	Orig. Date	Wks on List
1	95000527	IP-4-498, REPLACE WITH ROSEMOUNT I/P	01/05/95	
2	95001652	CV-3-3729, VALVE FAILED OPEN (AWP/ENG HOLDS)	01/13/95	
3	95005744	CV-3-6347B, VALVE LEAKS	01/20/95	
4	95005514	HSC-3-151, BRITTLE/CRACK INSUL ON WIRES (AWP HOLD)	01/27/95	
5	95005513	HSC-3-152, BRITTLE/CRACK INSUL ON WIRES (AWP HOLD)	01/27/95	
6	95006234	4p210, MOTOR SIGHTGLASS LEAKING OIL	02/10/95	
7	95006427	VARIOUS DOORS THROUGHOUT PLANT, INSTALL DRIP FLASHINGS ON DOORS	03/01/95	
8	95006428	D096-1 DOOR, WATER INTRUSION	03/01/95	
9	95007509	HAGAN CONTROLLER, 35V REG MEASURES 30.7	03/10/95	
10	95007510	HAGAN CONTROLLER, 35V REG MEASURES 53.2	03/10/95	

ST. LUCIE

REF	PWO #	PWO TITLE	Orig. Date	Wks on List
1	92052114	SCREEN CORRODED FALLING APART, SAFTEY	10/15/92	
2	93032089	COMPLETE UPGRADE OF COMPUTER SYSTEM	12/1/93	
3	94025159	REPLACE VALVES REMOVE DRIP PAN.	10/4/94	
4	94025159	REPLACE VALVES REMOVE DRIP PAN.	10/17/94	
5	95000079	ASSIST MECHANICAL MAINTENANCE	1/3/95	
6	95001003	INSTALL LEVEL INDICATOR FOR POOL LEVEL	1/11/95	
7	93034478	OVERHAUL SPARE HVS-1A MOTOR.	2/10/95	
8	95004447	REBAR IS RUSTED AND CONCRETE IS CRACKIN	2/14/95	
9	93034478	OVERHAUL SPARE HVS-1A MOTOR.	3/27/95	

PWO Greater than 12 month's, Work Types 1 through 5, excluding outage hold codes (SNO, SNW, HC2, HC3, HC4, HC5, HC6) status 22-48.

NON-OUTAGE PWO'S



Unit	1991	1992	1993	1994	1995	1/96	2/96	3/96	4/96	5/96	6/96	7/96	8/96	9/96	10/96	11/96	12/96
PTN	*	*	*	*	*	1024	1091	1118									
PSL	*	*	*	*	*	1247	1103	1016									

DEFINITION

Non-Outage PWO's includes Work Type 1 (Planned Misc) Work Type 3 (Projects) and Work Type 5 (Trouble & Breakdown), and excludes all outage hold codes. Short notice outages (SNO), HC2-Startup, HC3-Hot Standby, HC4-Hot Shutdown, HC5-Cold Shutdown, HC6-Refueling and includes status 22-48.

* Historic data not meaningful. Prior to 1996, each site was counting Open PWO's differently. St. Lucie was counting only Work Type 5 non-outage corrective and Turkey Point was counting Work Types 1 and 5. Each site now agrees to report per the above definition.

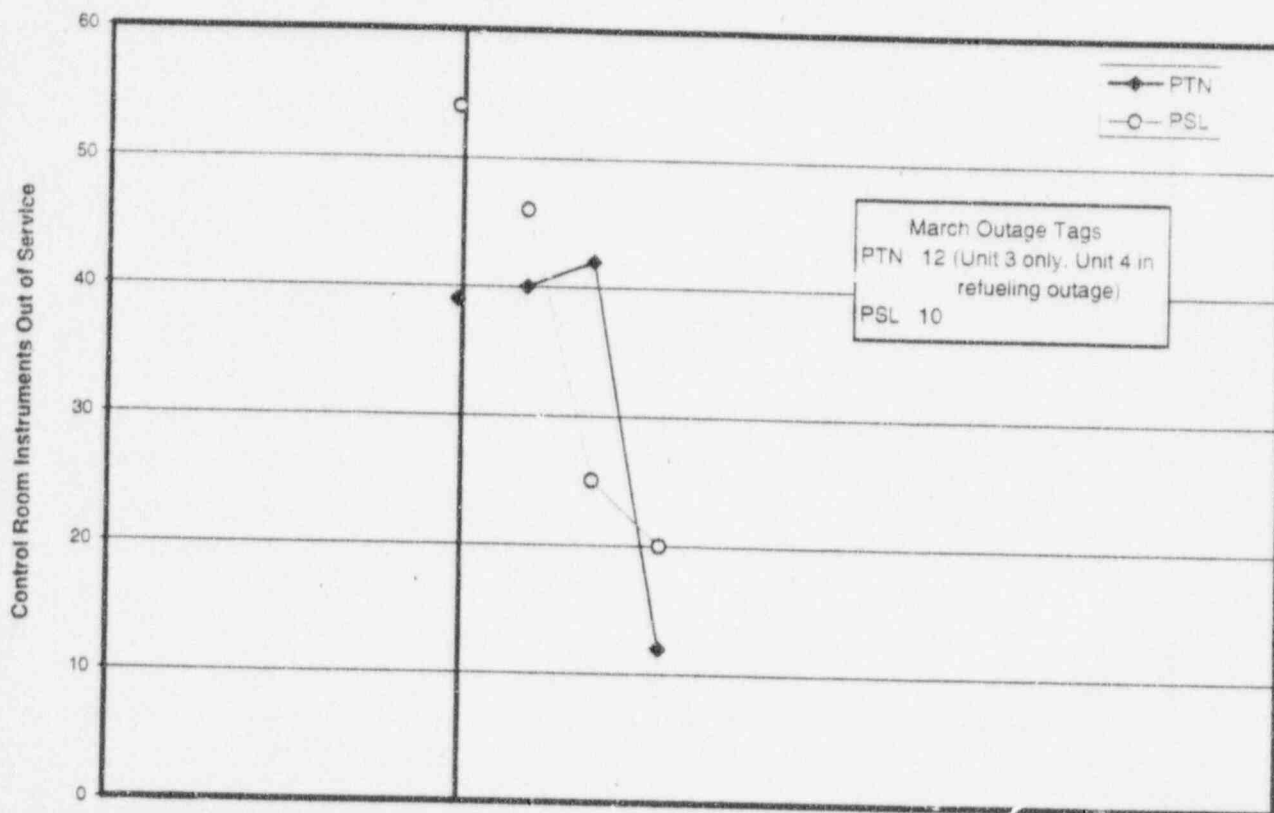
STATISTICAL SUMMARY

INDUSTRY PERFORMANCE

PERFORMANCE SUMMARY

Data Providers: (PTN) Greg Heisterman 246-6796 and (PSL) Joe M. Hesse 467-7107

CONTROL ROOM INSTRUMENTS OUT-OF-SERVICE



DEFINITION

This indicator defines the number of control room instruments for each unit that cannot perform their design function, regardless of the reason. Instruments on the control room back panels are readily available for use by the control room crews and are included; however, instruments in adjoining areas where operators are not normally stationed (such as computer rooms) are not included. Count deficiency tags that are in status 05 to 48.

STATISTICAL SUMMARY

INDUSTRY PERFORMANCE

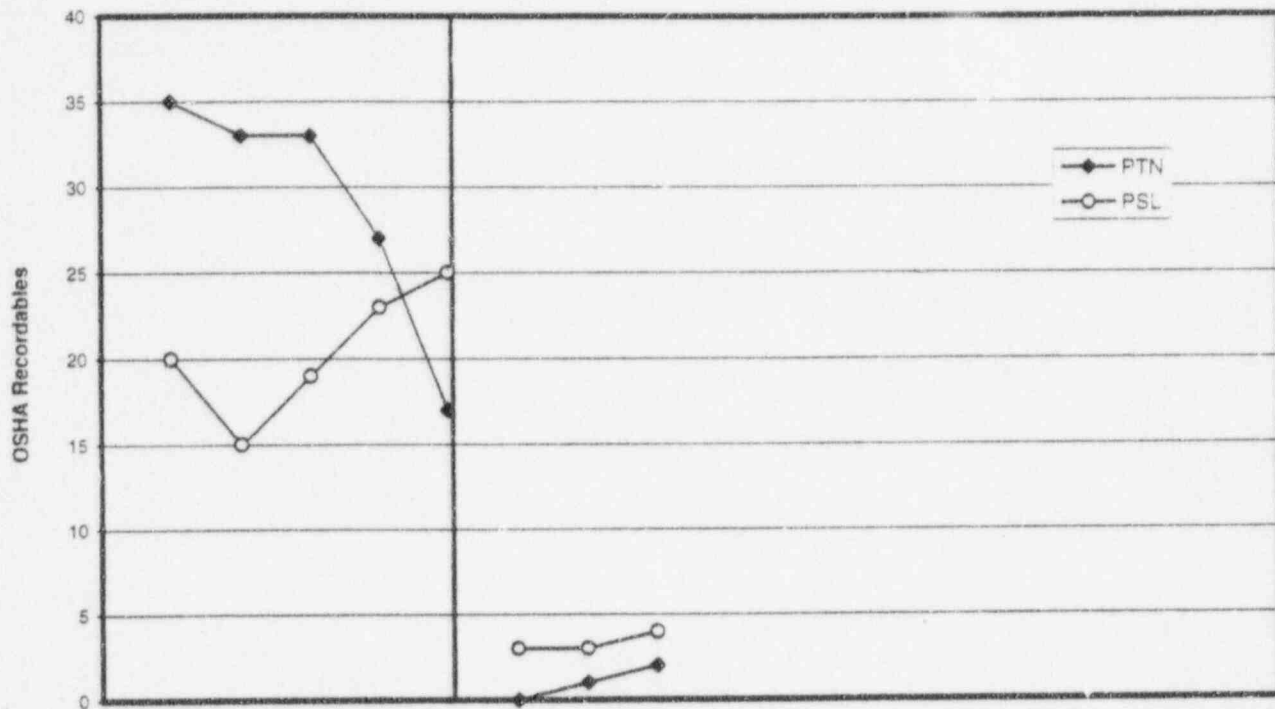
PERFORMANCE SUMMARY

A correction has been made to PTN data for Year End 1995 and January. The data reflects the corrected numbers.

Data Providers: (PTN) Greg Heisterman 246-6796 and (PSL) Joe Marchese 467-7107

OSHA RECORDABLES

(Year-to-Date)



Unit	1991	1992	1993	1994	1995	1/96	2/96	3/96	4/96	5/96	6/96	7/96	8/96	9/96	10/96	11/96	12/96
PTN	35	33	33	27	17	0	1	2									
PSL	20	15	19	23	25	3	3	4									

DEFINITION

The definition by Occupational Safety Health Administration (OSHA) is an injury occurring on the job that requires medical treatment beyond first aid as defined by 29 CFR 1904.

STATISTICAL SUMMARY

	Month	Y-T-D
PTN	1	2
PSL	1	4

Nuclear Division Y-E Target: 1.75
Corporate Y-E Target: 3.75

INDUSTRY PERFORMANCE

PERFORMANCE SUMMARY

One OSHA Recordable Injury was reported at Turkey Point

- a mechanic was swinging a sledge hammer and caught his finger between the handle and a shackle causing a hairline fracture to the finger.

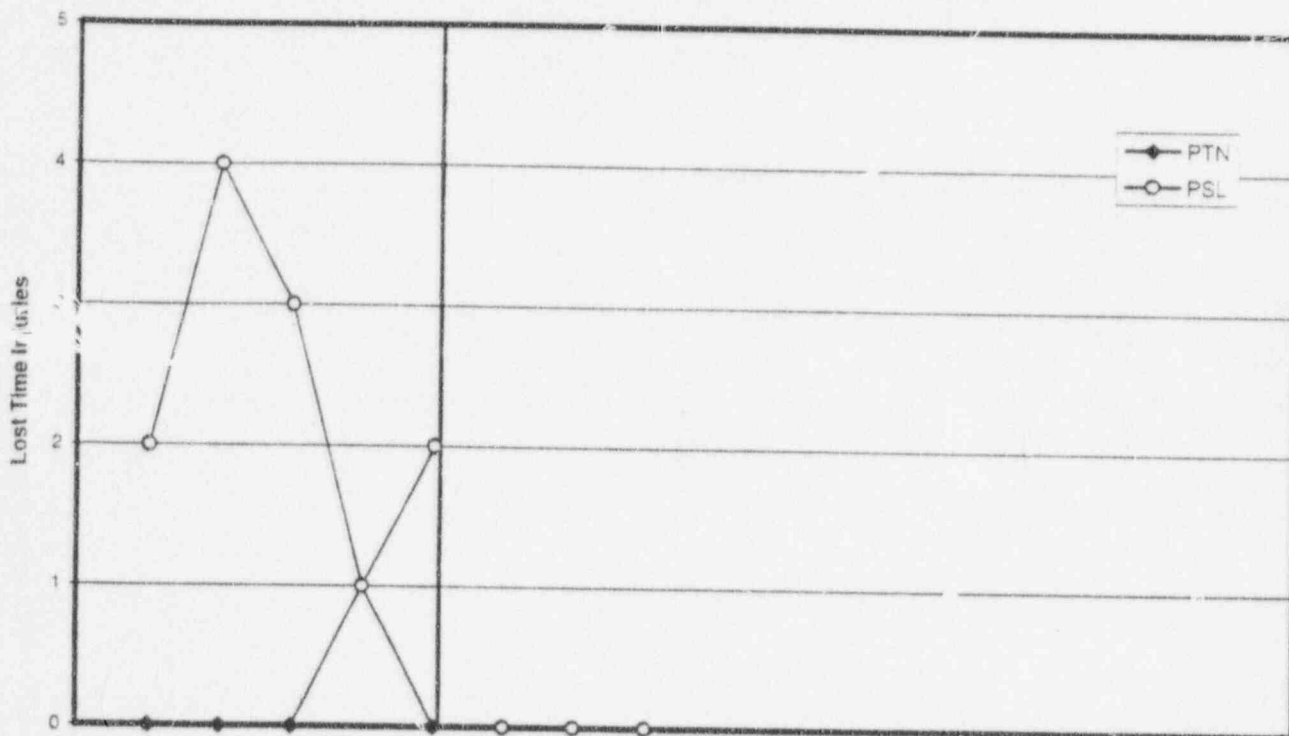
St. Lucie reported two OSHA Recordables in March:

- a mechanic was holding an elephant trunk to remove smoke from an area where another mechanic was burning off bolts on a damper. He was in a cramped area and his coveralls ignited causing 2nd degree burns to his calf.
- A SNPO injured his knee in January and was placed on restricted duty. An MRI in March revealed a torn ligament in the knee.

Data provider: W. Korte 694-4235

LOST TIME INJURIES

(Year-to-Date)



Unit	1991	1992	1993	1994	1995	1/96	2/96	3/96	4/96	5/96	6/96	7/96	8/96	9/96	10/96	11/96	12/96
PTN	0	0	0	1	0	0	0	0									
PSL	2	4	3	1	2	0	0	0									

DEFINITION

A Lost Time Injury as defined by Occupational Safety Health Administration (OSHA) is an occupational injury that requires an employee to miss a full day (8 hour shift) beyond the day of injury.

STATISTICAL SUMMARY

	Month	Y-T-D
PTN	0	0
PSL	0	0

INDUSTRY PERFORMANCE

PERFORMANCE SUMMARY

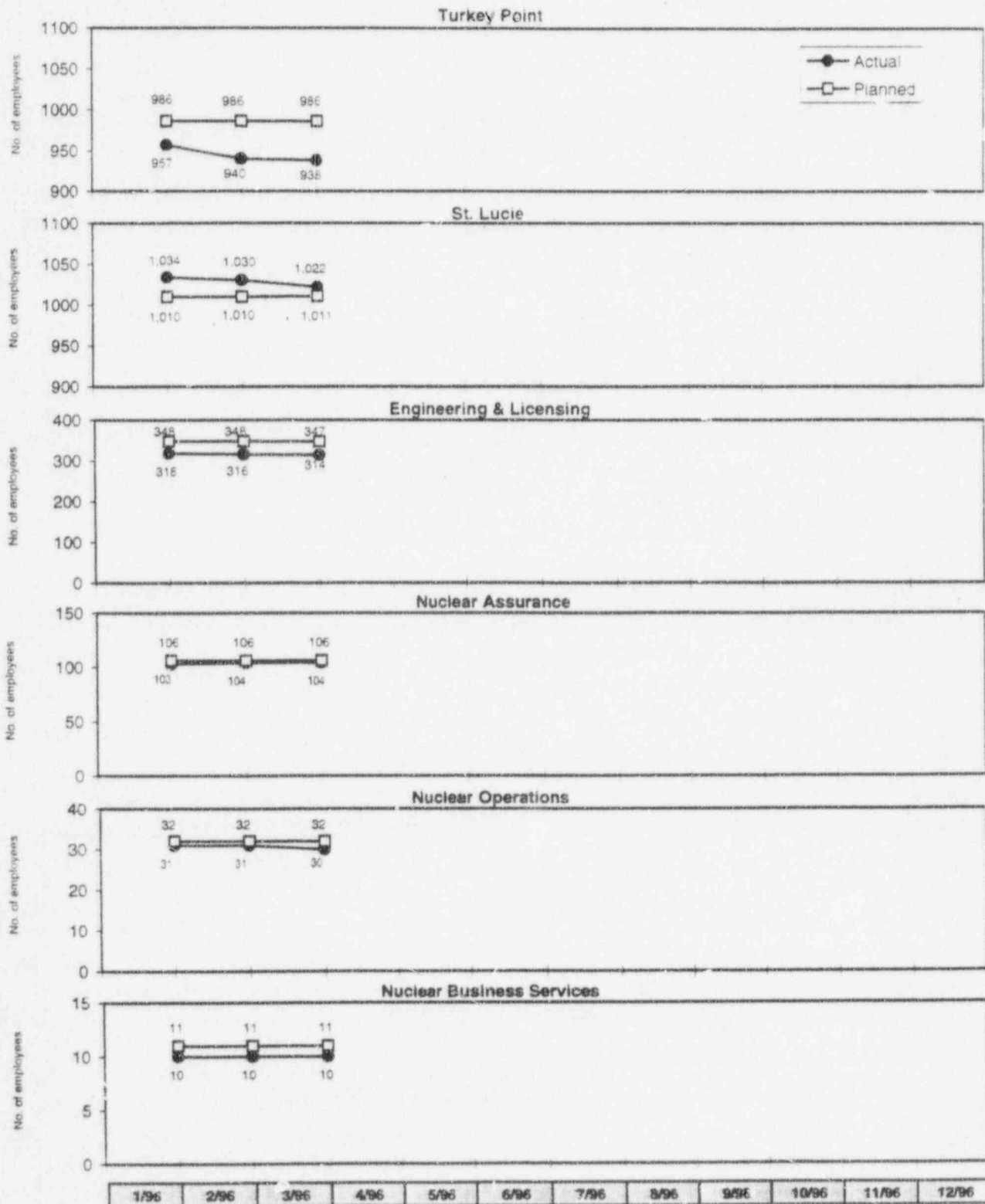
No Lost Time Injuries were reported in March 1996.

Data provider: W. Korte 694-4235

REGULAR STAFFING

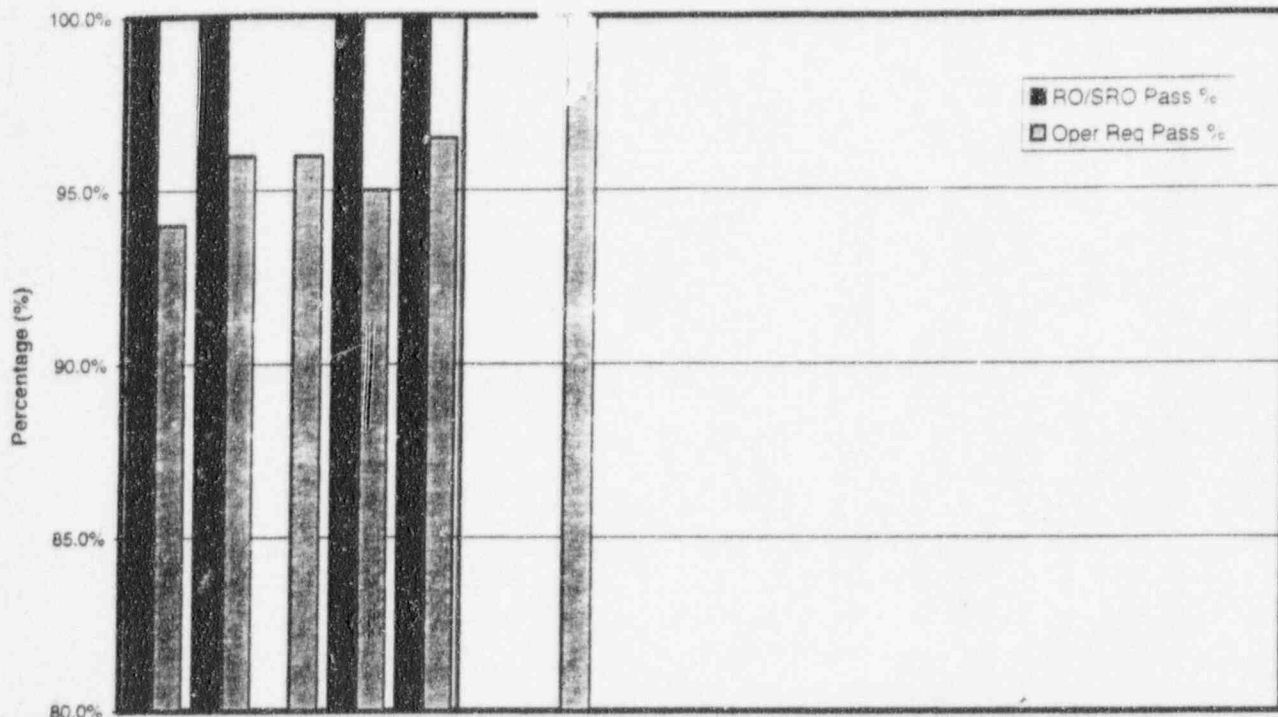
FPL Employees and Long-Term Contractors

by Department



OPERATOR EXAMINATION PERFORMANCE

Turkey Point Units 3 & 4



Exam:	1991	1992	1993	1994	1995	1/96	2/96	3/96	4/96	5/96	6/96	7/96	8/96	9/96	10/96	11/96	12/96
RO/SRO Taken	39	5	-	8	13	-	-	-	-	-	-	-	-	-	-	-	-
Passed	39	5	-	8	13	-	-	-	-	-	-	-	-	-	-	-	-
RO/SRO Pass %	100.0%	100.0%	-	100.0%	100.0%	-	-	-	-	-	-	-	-	-	-	-	-
Oper Reg Taken	47	78	69	63	58	-	64	-	-	-	-	-	-	-	-	-	-
Passed	44	75	66	60	56	-	64	-	-	-	-	-	-	-	-	-	-
Oper Reg Pass %	94.0%	96.0%	96.0%	95.0%	96.5%	-	100.0%	-	-	-	-	-	-	-	-	-	-

DEFINITION

Initial License Examination (RO/SRO) results are reported for all candidates taking an Initial License Exam as conducted by the NRC.

Operator Regualification Examination results are reported for both RO's and SRO's. This examination is administered annually by the utility and may be jointly administered by the NRC. Retests of operators who failed examinations are not included.

STATISTICAL SUMMARY

INDUSTRY PERFORMANCE

Initial RO/SRO License Exams			
	No. Taken	No. Passed	Pass Rate %
YTD 1996	0	0	0.0%
YTD 1995	13	13	100.0%

Operator Regual Exams			
	No. Taken	No. Passed	Pass Rate %
YTD 1996	64	64	100.0%
YTD 1995	58	56	96.5%

The NRC at their last Regional Training Managers Meeting (for fiscal year 1994) provided the following data:

Initial NRC Exams

NRC Regual Exams

RO's Pass Rate	94.6%	RO's Pass Rate	91.0%
Instant SRO's Pass Rate	94.4%	SRO's Pass Rate	85.0%
Upgrade SRO's Pass Rate	94.7%	Average Overall	88.0%
Average Overall	94.6%		

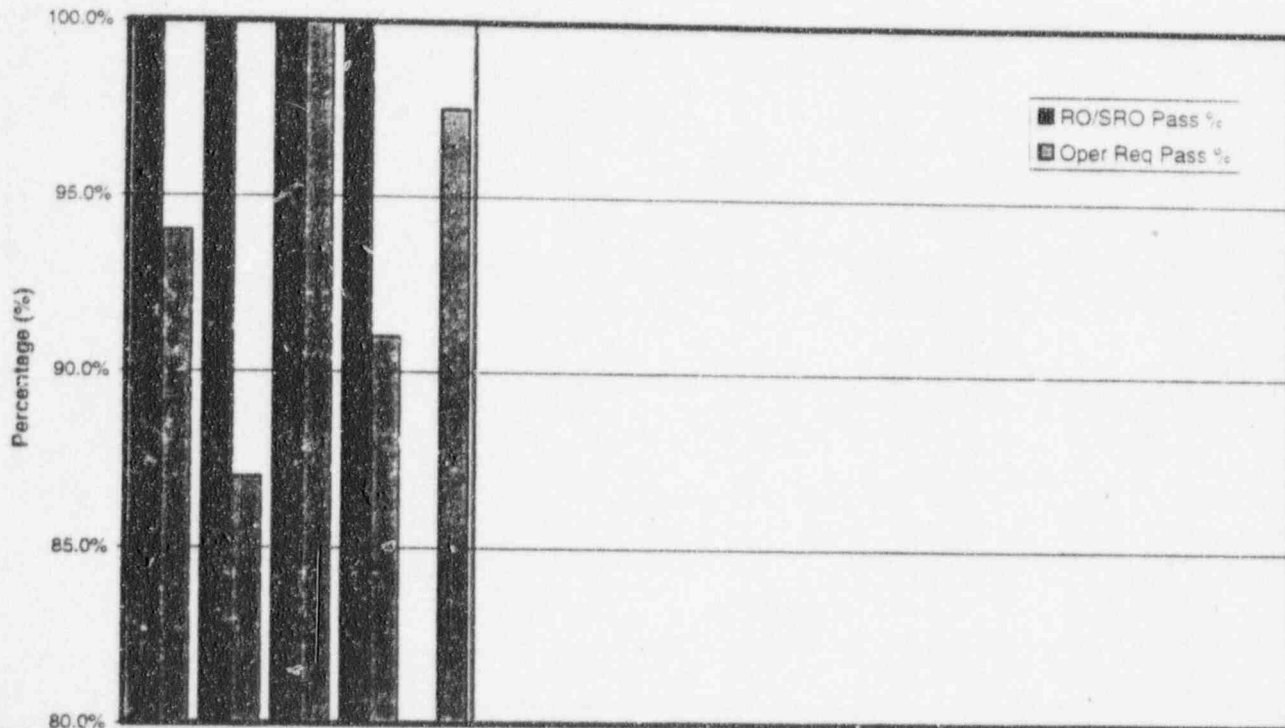
PERFORMANCE SUMMARY

No Initial RO/SRO License or Operator Regualification Exams were given in March 1996 at Turkey Point.

Data provided: K. E. Beatty 694-4217

OPERATOR EXAMINATION PERFORMANCE

St. Lucie Units 1 & 2



Exam:	1991	1992	1993	1994	1995	1/96	2/96	3/96	4/96	5/96	6/96	7/96	8/96	9/96	10/96	11/96	12/96
RO/SRO Taken	15	8	10	11	-	-	-	-	-	-	-	-	-	-	-	-	-
RO/SRO Passed	15	8	10	11	-	-	-	-	-	-	-	-	-	-	-	-	-
RO/SRO Pass %	100.0%	100.0%	100.0%	100.0%	-	-	-	-	-	-	-	-	-	-	-	-	-
Oper Req Taken	65	69	68	75	73	-	-	-	-	-	-	-	-	-	-	-	-
Oper Req Passed	61	60	68	68	71	-	-	-	-	-	-	-	-	-	-	-	-
Oper Req Pass %	94.0%	87.0%	100.0%	91.0%	97.5%	-	-	-	-	-	-	-	-	-	-	-	-

DEFINITION

Initial License Examination (RO/SRO) results are reported for all candidates taking an Initial License Exam as conducted by the NRC.

Operator Requalification Examination results are reported for both RO's and SRO's. This examination is administered annually by the utility and may be jointly administered by the NRC. Retests of operators who failed examinations are not included.

STATISTICAL SUMMARY

Initial RO/SRO License Exams			
	No. Taken	No. Passed	Pass Rate %
YTD 1996	0	0	0.0%
YTD 1995	0	0	0.0%
Operator Requal Exams			
	No. Taken	No. Passed	Pass Rate %
YTD 1996	0	0	0.0%
YTD 1995	73	71	97.5%

INDUSTRY PERFORMANCE

The NRC at their last Regional Training Managers Meeting (for fiscal year 1994) provided the following data:

Initial NRC Exams		NRC Requal Exams	
RO's Pass Rate	94.6%	RO's Pass Rate	91.0%
Instant SRO's Pass Rate	94.4%	SRO's Pass Rate	85.0%
Upgrade SRO's Pass Rate	94.7%	Average Overall	88.0%
Average Overall	94.6%		

PERFORMANCE SUMMARY

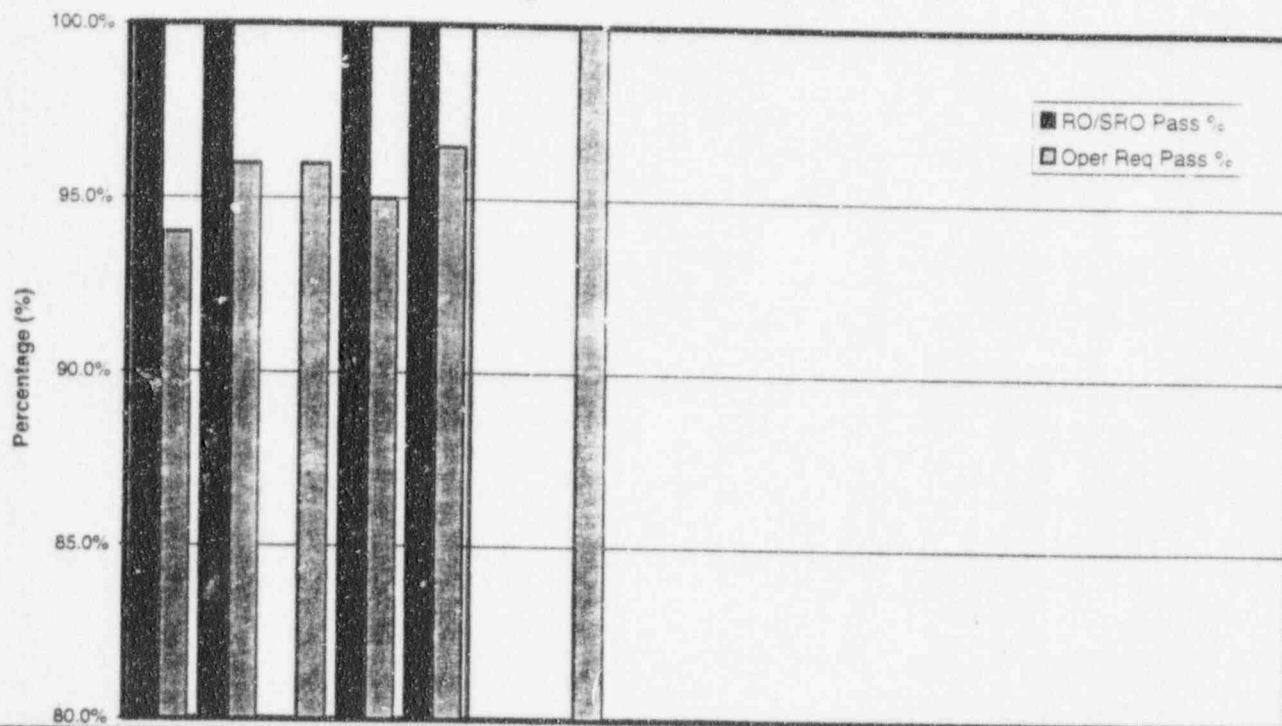
No Operator Requalification Exams were given in March 1996 at St. Lucie.

Initial RO/SRO License Exams were given to be given in March; results from the NRC are expected in April.

Data provider: K. E. Beatty 694-4217

OPERATOR EXAMINATION PERFORMANCE

Turkey Point Units 3 & 4



Exams:	1991	1992	1993	1994	1995	1/96	2/96	3/96	4/96	5/96	6/96	7/96	8/96	9/96	10/96	11/96	12/96
RO/SRO Taken	39	5	-	8	13	-	-	-	-	-	-	-	-	-	-	-	-
Passed	39	5	-	8	13	-	-	-	-	-	-	-	-	-	-	-	-
RO/SRO Pass %	100.0%	100.0%	-	100.0%	100.0%	-	-	-	-	-	-	-	-	-	-	-	-
Oper Req Taken	47	78	69	63	58	-	64	-	-	-	-	-	-	-	-	-	-
Passed	44	75	66	60	56	-	64	-	-	-	-	-	-	-	-	-	-
Oper Req Pass %	94.0%	96.0%	96.0%	95.0%	96.5%	-	100.0%	-	-	-	-	-	-	-	-	-	-

DEFINITION

Initial License Examination (RO/SRO) results are reported for all candidates taking an Initial License Exam as conducted by the NRC.

Operator Regualification Examination results are reported for both RO's and SRO's. This examination is administered annually by the utility and may be jointly administered by the NRC. Retests of operators who failed examinations are not included.

STATISTICAL SUMMARY

INDUSTRY PERFORMANCE

Initial RO/SRO License Exams			
	No. Taken	No. Passed	Pass Rate %
YTD 1996	0	0	0.0%
YTD 1995	13	13	100.0%

The NRC at their last Regional Training Managers Meeting (for fiscal year 1994) provided the following data:

Operator Regual Exams			
	No. Taken	No. Passed	Pass Rate %
YTD 1996	64	64	100.0%
YTD 1995	58	56	96.5%

Initial NRC Exams		NRC Regual Exams	
RO's Pass Rate	94.6%	RO's Pass Rate	91.0%
Instant SRO's Pass Rate	94.4%	SRO's Pass Rate	85.0%
Upgrade SRO's Pass Rate	94.7%	Average Overall	88.0%
Average Overall	94.6%		

PERFORMANCE SUMMARY

No Initial RO/SRO License or Operator Regualification Exams were given in March 1996 at Turkey Point.

Data provider: K. E. Beatty 694-4217

OPERATOR EXAMINATION PERFORMANCE

St. Lucie Units 1 & 2



Exams:	1991	1992	1993	1994	1995	1/96	2/96	3/96	4/96	5/96	6/96	7/96	8/96	9/96	10/96	11/96	12/96
RO/SRO Taken	15	8	10	11	-	-	-	-	-	-	-	-	-	-	-	-	-
RO/SRO Passed	15	8	10	11	-	-	-	-	-	-	-	-	-	-	-	-	-
RO/SRO Pass %	100.0%	100.0%	100.0%	100.0%	-	-	-	-	-	-	-	-	-	-	-	-	-
Operator Taken	65	69	68	75	73	-	-	-	-	-	-	-	-	-	-	-	-
Operator Passed	61	60	68	68	71	-	-	-	-	-	-	-	-	-	-	-	-
Operator Pass %	94.0%	87.0%	100.0%	91.0%	97.5%	-	-	-	-	-	-	-	-	-	-	-	-

DEFINITION

Initial License Examination (RO/SRO) results are reported for all candidates taking an Initial License Exam as conducted by the NRC

Operator Requalification Examination results are reported for both RO's and SRO's. This examination is administered annually by the utility and may be jointly administered by the NRC. Retests of operators who failed examinations are not included.

STATISTICAL SUMMARY

Initial RO/SRO License Exams			
	No. Taken	No. Passed	Pass Rate %
YTD 1996	0	0	0.0%
YTD 1995	0	0	0.0%
Operator Requal Exams			
	No. Taken	No. Passed	Pass Rate %
YTD 1996	0	0	0.0%
YTD 1995	73	71	97.5%

INDUSTRY PERFORMANCE

The NRC at their last Regional Training Managers Meeting (for fiscal year 1994) provided the following data:

Initial NRC Exams		NRC Requal Exams	
RO's Pass Rate	94.6%	RO's Pass Rate	91.0%
Instant SRC's Pass Rate	94.4%	SRO's Pass Rate	85.0%
Upgrade SRO's Pass Rate	94.7%	Average Overall	88.0%
Average Overall	94.6%		

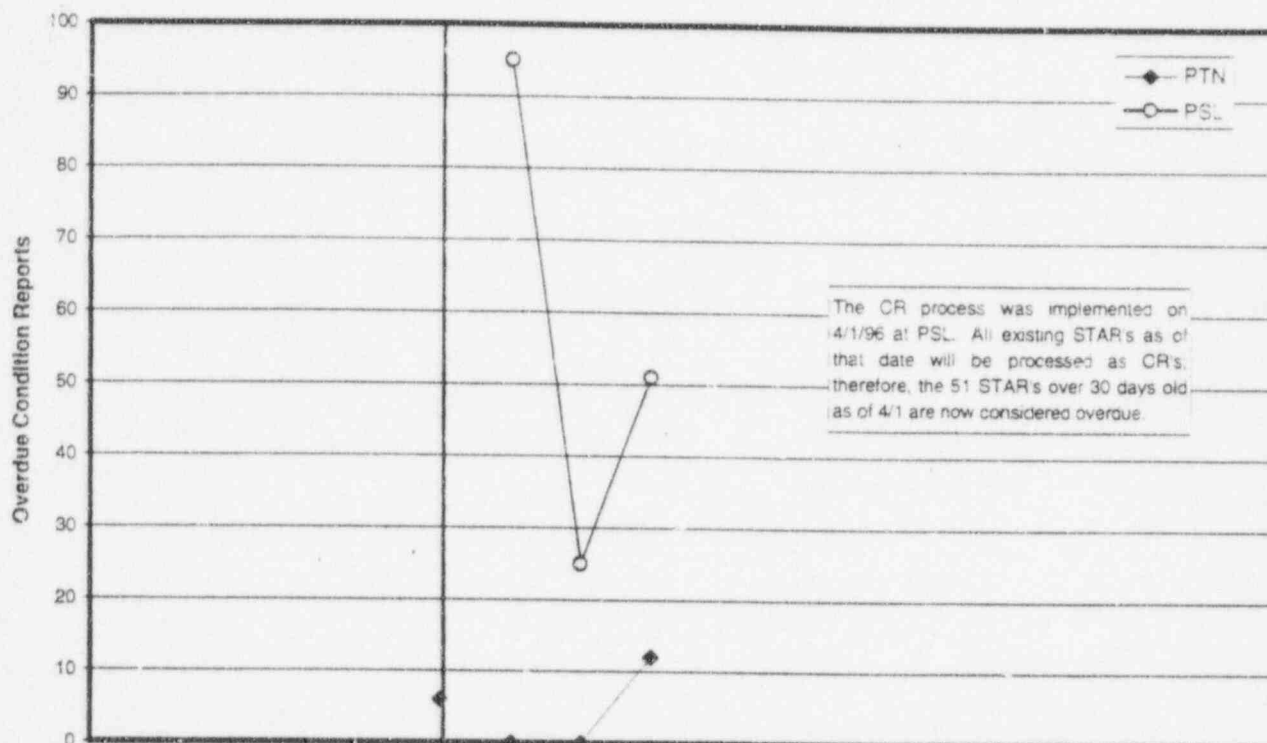
PERFORMANCE SUMMARY

No Operator Requalification Exams were given in March 1996 at St. Lucie.

Initial RO/SRO License Exams were given to be given in March; results from the NRC are expected in April.

Data provider: K. E. Beatty 694-4217

OVERDUE CONDITION REPORTS



Unit	1991	1992	1993	1994	1995	1/96	2/96	3/96	4/96	5/96	6/96	7/96	8/96	9/96	10/96	11/96	12/96
PTN					6	0	0	12									
PSL						95	25	51									

DEFINITION

Currently, the Nuclear Division is transitioning to a consistent Condition Report. The project is scheduled to be completed by May 1, 1996 at all locations. This graph shows Condition Reports that exceed assigned priority timeframe.

Severity Levels are as follows:

A = 3 working days

B = 10 calendar days

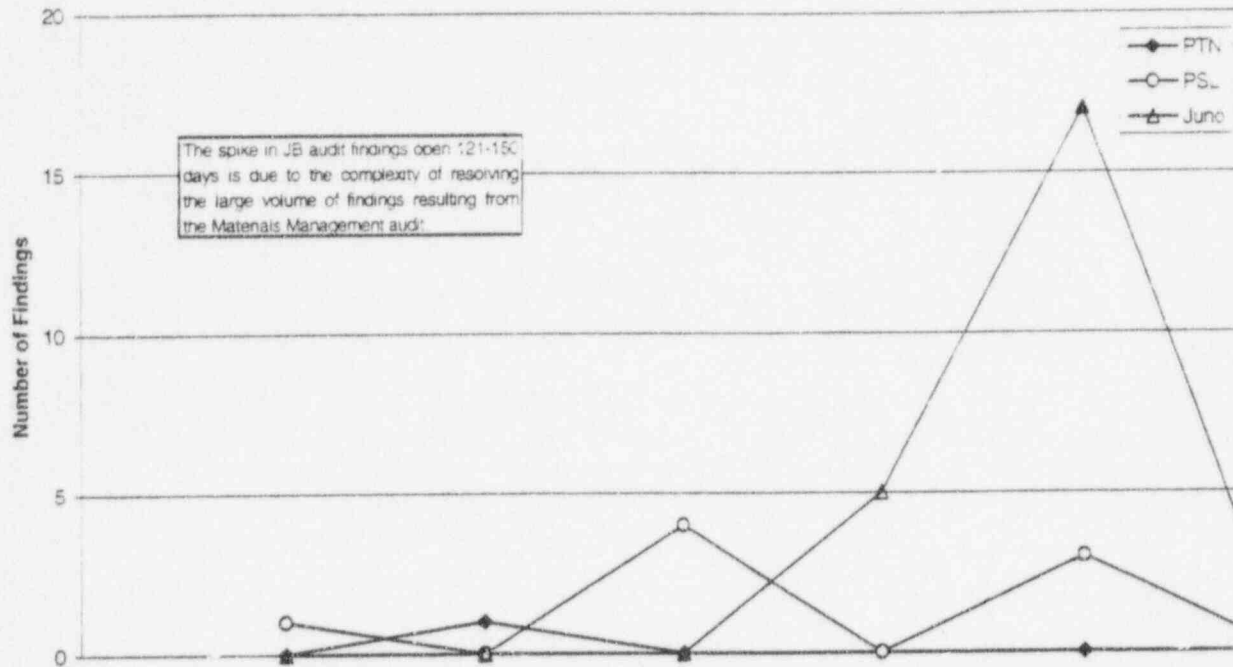
C = 30 calendar days

STATISTICAL SUMMARY

INDUSTRY PERFORMANCE

PERFORMANCE SUMMARY

QA FINDINGS



Unit	0-30	31-60	61-90	91-120	121-150	151-180
PTN	0	1	0	0	0	0
PSL	1	0	4	0	3	0
June	0	0	0	5	17	0

DEFINITION

This indicator represents the time it takes for verification of implementation of corrective action for audit findings. The clock for the QA group starts when the audited organization notifies QA that implementation of audit finding corrective actions are complete and ready for verification. The process is complete when QA closes the audit finding and notifies the audited organization that the audit finding corrective action is complete.

STATISTICAL SUMMARY

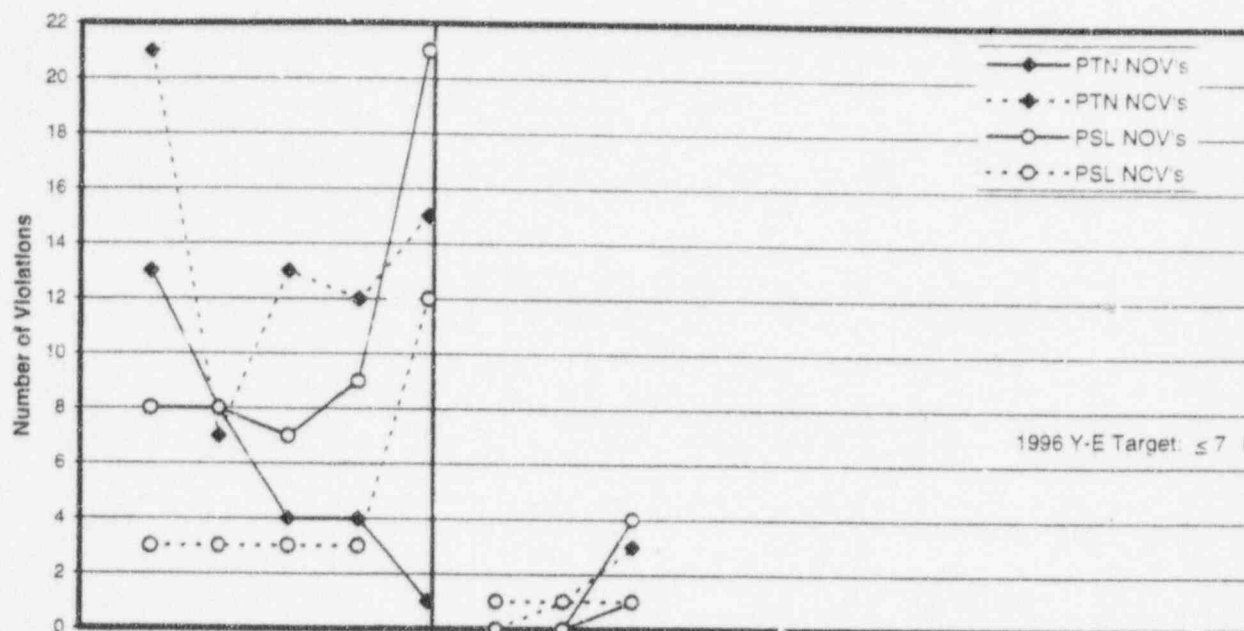
INDUSTRY PERFORMANCE

PERFORMANCE SUMMARY

Data Providers: (PTN) Tom Abbatelio 246-6581 and (PSL) Wes Bladow 467-7111

NRC VIOLATIONS Cited and Non-Cited

(Year-to-Date)



DEFINITION

Violations are categorized in terms of five levels of severity to show their relative importance. Severity Levels I and II are violations that involve actual or high potential impact on the public. Severity Level III Violations are cause for significant concern. Severity Level IV violations are less serious, but are of more than minor concern; i.e., if left uncorrected, could lead to a more serious concern. Severity Level V violations are minor safety or environmental concern. Violations are counted on the date of the inspection exit meeting. Violations are now counted with respect to the date of occurrence (using the date of the inspection exit meeting) instead of the date of the inspection report, as was done in the past.

STATISTICAL SUMMARY

	Mar	YTD	Year End Target
PTN Cited	1	1	7
PTN Non-Cited	2	3	
PSL Cited	0	4	7
PSL Non-Cited	0	1	

INDUSTRY PERFORMANCE

NRC Violations

1994 IBG Group Mean	16.2
1994 Region II Mean	13.1
1994 IBG Top Quartile Mean	9.0
1994 Region II Top Quartile Entry	8.0

PERFORMANCE SUMMARY

Turkey Point reported one NRC Violations for March:

#96-02-04: Failure to meet requirements of 10CFR50.71(e) in that the FSAR was not appropriately updated to reflect the ofload of a full core during a refueling evolution. Exit Meeting date: 3/29/96.

St. Lucie reported no NRC Violations for the month; however, four violations were reported for February:

#96-01-01: Temporary Procedure changes were made which involved changes of intent, without prior FRG review as required by Technical Specifications. Exit Meeting Date: 2/20/96.

#96-03-01: *Operators failed to follow procedures for boron dilution. Exit Meeting Date: 2/8/96.

#96-03-02: *Inadequate design control of RCS boron dilution procedure. Exit Meeting Date: 2/5/96.

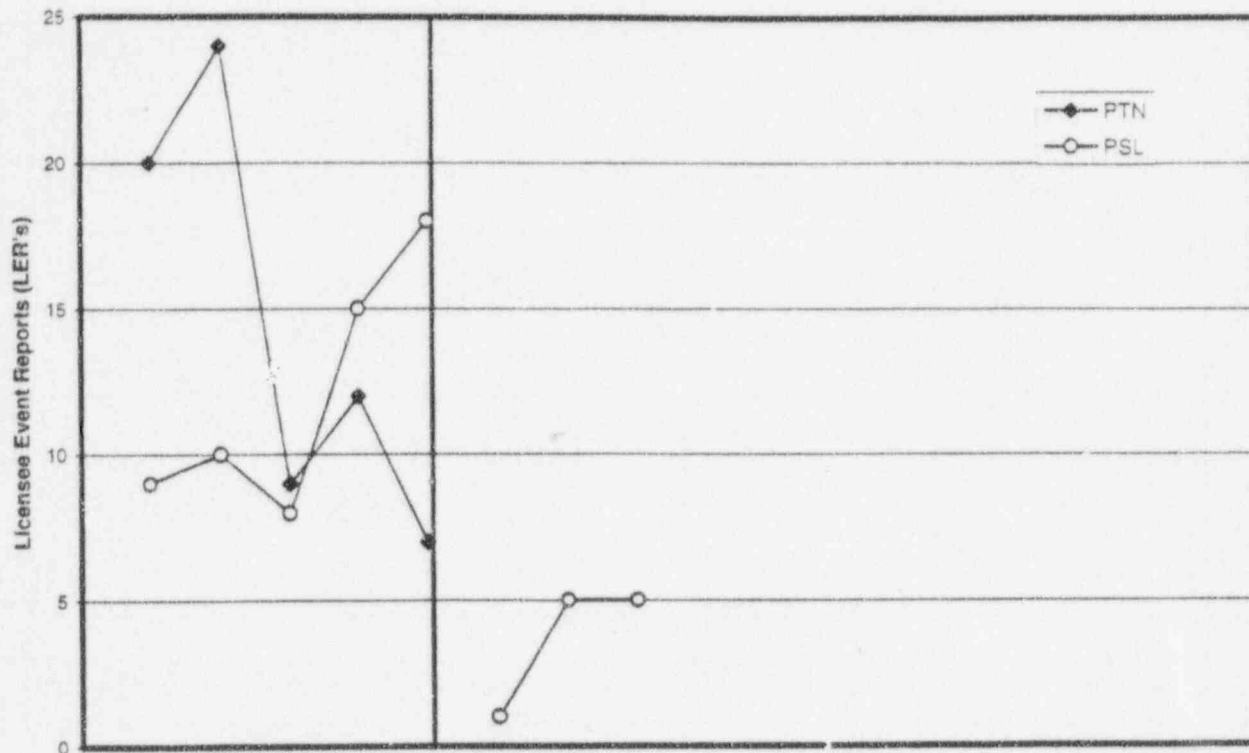
#96-03-03: *Inadequate 50.59 safety evaluation of changes to boron dilution procedure. Exit Meeting Date: 2/8/96.

*On April 1, St. Lucie was informed that these violations represent a Severity Level III problem for Unit 1 only. A Civil Penalty of \$50,000 was imposed.

Data Providers: (PTN) Garv Hollinger 246-6078 and (PSL) Ed Weinkam 467-7162

LICENSEE EVENT REPORTS

(Year-to-Date)



Unit	1991	1992	1993	1994	1995	1/96	2/96	3/96	4/96	5/96	6/96	7/96	8/96	9/96	10/96	11/96	12/96
PTN	20	24	9	12	7	1	5	5									
PSL	9	10	8	15	18	1	5	5									

DEFINITION

License Event Reports (LER) are submitted to the NRC by the licensee to report unusual occurrences prescribed by 10CFR50.73.

STATISTICAL SUMMARY

	Mar	YTD
PTN	0	5
PSL	0	5

INDUSTRY PERFORMANCE

PERFORMANCE SUMMARY

No LER's were reported in March 1996.

Data Providers: (PTN) Gary Hollinger 246-6078 and (PSL) Ed Weinkam 467-7162

INPO ASSESSMENT RATINGS



Unit	1986	1987	1988	1989	1990	1991	1992	1993	1994	1995	1996
PTN	4.00	4.00	3.00	4.00	3.00	-	2.00	2.00	-	1.00	-
PSL	-	-	2.00	-	1.00	-	1.00	-	-	1.00	-

INPO ASSESSMENT PROGRAM DESCRIPTION

The Institute of Nuclear Power Operations (INPO) conducts periodic evaluations of site activities to make an overall determination of plant safety, to evaluate management systems and controls, and to identify areas needing improvement. Information is assembled from discussions, interviews, observations, and reviews of documentation.

Evaluation Frequency: INPO's goal is to visit each plant on an average of every 18 months. However, this frequency may vary depending upon the previous assessment ratings. For instance, if a plant is rated a "1" or "2", the interval between assessments is usually 20-24 months; if a plant is rated a "4" or a "5", the assessment interval is ≤ 18 months.

Performance Category Ratings:

- Category 1: Overall performance is excellent. Industry standards of excellence are met in most areas. No significant weaknesses noted.
- Category 2: Overall performance is exemplary. Industry standards of excellence are met in most areas. No significant weaknesses noted.
- Category 3: Overall performance is generally in keeping with the high standards required in nuclear power. However, improvements are needed in a number of areas. A few significant weaknesses may exist.
- Category 4: Overall performance is acceptable, but improvements are needed in a wide range of areas. Significant weaknesses are noted in several areas.
- Category 5: Overall performance does not meet the industry standard of acceptable performance. The margin of nuclear safety is measurably reduced. Strong and immediate management action to correct deficiencies is required. Special attention, assistance, and follow-up are required.

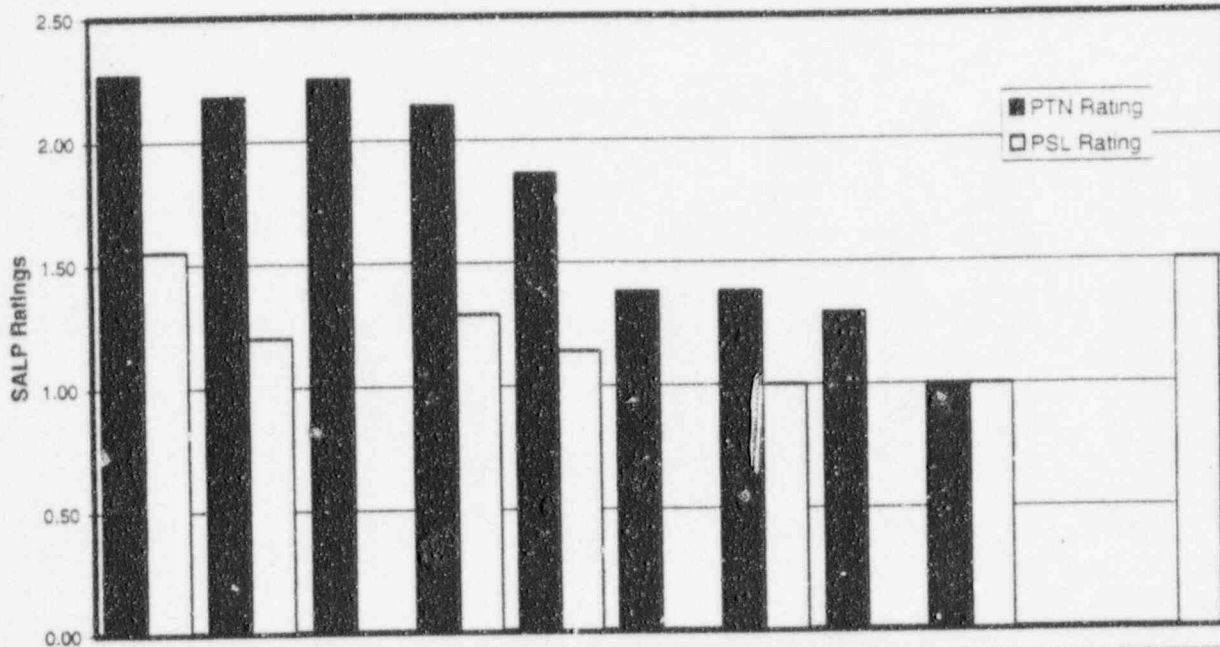
NOTE: If a plant is found to be operating without an adequate margin of nuclear safety, INPO will request that the plant be shutdown, or not started up.

PERFORMANCE SUMMARY

Turkey Point and St. Lucie received an INPO category rating of "1" in 1995. The next evaluations for Turkey Point and St. Lucie are expected in the last quarter of 1996 and first quarter of 1997, respectively.

Data Providers: (PTN) Gary Hollinger 246-6078 and (PSL) Ed Weinkam 467-7162

NRC SALP CATEGORY RATINGS



	1986	1987	1988	1989	1990	1991	1992	1993	1994	1995	1996
PTN Rating	2.27	2.18	2.25	2.14	1.86	1.38	1.38	1.29	1.00	-	-
Period Ending	4/30/86	5/31/87	6/30/88	7/31/89	7/31/90	9/28/91	-	1/30/93	6/27/94	-	-
PSL Rating	1.55	1.20	-	1.29	1.14	-	1.00	-	1.00	-	1.50
Period Ending	4/30/86	10/31/87	-	4/30/89	10/31/90	-	5/2/92	-	1/1/94	-	1/6/96

SALP PROGRAM DESCRIPTION

It is the policy of the NRC to use the Systematic Assessment of Licensee Performance (SALP) process to articulate the agency's observations and insights on licensee safety performance. The SALP report communicates those observations and insights.

Evaluation Frequency: The NRC will normally review and evaluate each power reactor licensee that possesses an operating license at least every 18 months. When the NRC determines that the performance warrants a more frequent evaluation, the normal SALP frequency may be increased. The assessment period may be extended to a maximum of 24 months when a plant receives a Category 1 rating in all four functional areas.

Functional Areas: Performance is generally evaluated in four (4) functional areas:

- Plant Operations.** This functional area consists chiefly of the control and execution of activities directly related to operating a plant. It includes activities such as plant startup, power operations, plant shutdown, and system lineups. It also includes initial and requalification training of licensed operators.
- Maintenance.** This functional area includes all activities associated with either diagnostic, predictive, preventive, or corrective maintenance of plant structures, systems, and components, or maintenance of the physical condition of the plant.
- Engineering.** This functional area addresses the adequacy of technical and engineering support for all plant activities. It includes all licensee activities associated with design control; the design, installation, and testing of plant modifications; engineering and technical support for operations, outages, maintenance, testing, surveillance, and procurement activities; configuration management; design-basis information and its retrieval; and support for licensing activities.
- Plant Support.** This functional area covers all activities related to plant support functions, including radiological controls, emergency preparedness, security, chemistry, and fire protection. Housekeeping controls are also included in this area.

Performance Category Ratings: Licensee performance in each functional area is assessed by assigning a category rating as discussed below:

Category 1. Licensee attention and involvement have been properly focused on safety and resulted in a superior level of safety performance.

Category 2. Licensee attention and involvement are normally well focused and resulted in a good level of safety performance.

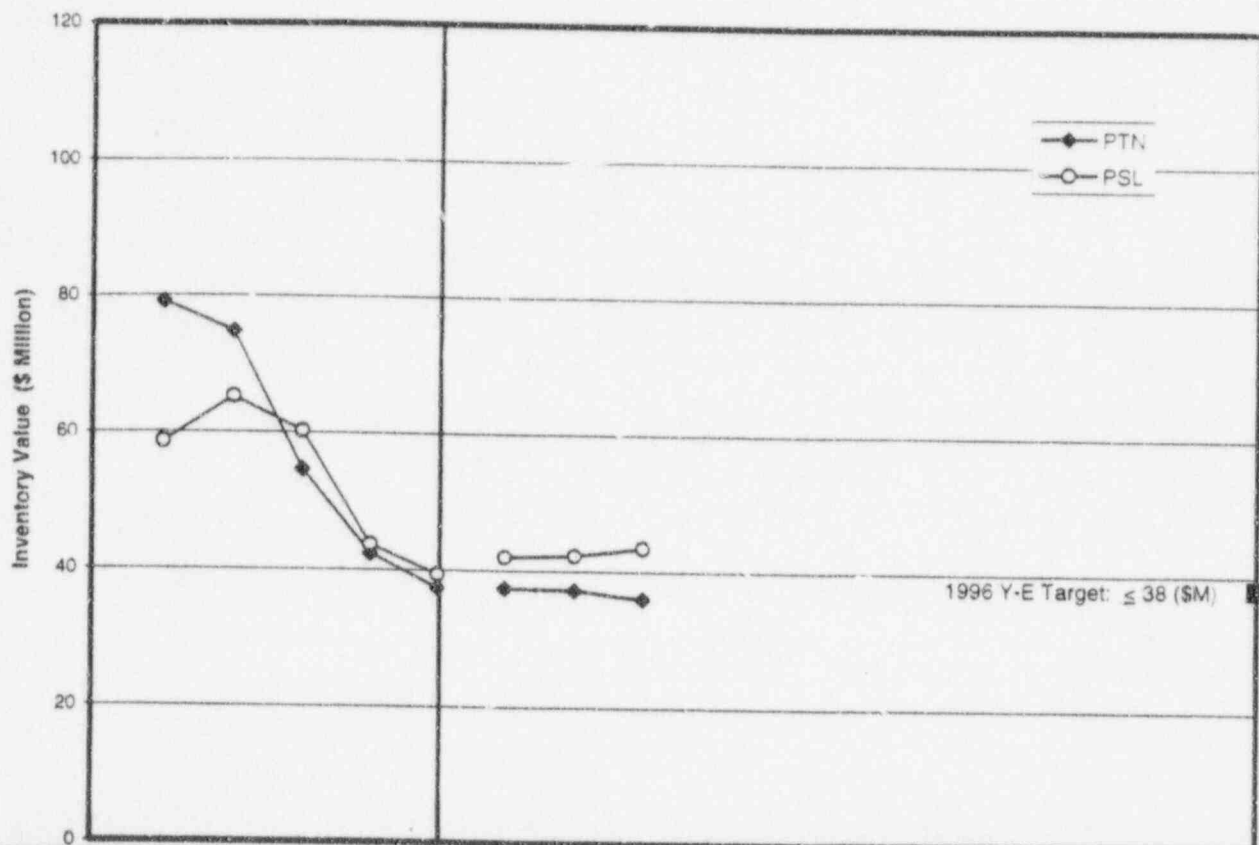
Category 3. Licensee attention and involvement have resulted in an acceptable level of performance. However, performance may exhibit one or more of the following characteristics: ineffective programs and significant issues, lack of corrective action thoroughness, and deficiencies in root cause analysis. Because the margin to unacceptable performance in important aspects is small, increased NRC and licensee attention is required.

PERFORMANCE SUMMARY

Functional Area	Turkey Point		St. Lucie	
	Prior	Most Recent	Prior	Most Recent
Plant Operations	1	1	1	2
Maintenance	2 Improving	1	1	2
Engineering	2 Improving	1	1	1
Plant Support	n/a	1	1	1
Emergency Preparedness	1	n/a	n/a	n/a
Radiological Controls	1	n/a	n/a	n/a
Security	1	n/a	n/a	n/a
Self Assessment / Quality Verification	1	n/a	n/a	n/a
Overall:	1.29	1.00	1.00	1.50

Data Providers: (PTN) Gary Hollinger 246-6078 and (PSL) Ed Weinkam 467-7162

INVENTORY VALUE ACCOUNT 154.300



Year	1991	1992	1993	1994	1995	1/96	2/96	3/96	4/96	5/96	6/96	7/96	8/96	9/96	10/96	11/96	12/96
PTN	79.1	74.7	54.6	42.4	37.4	37.4	37.2	36.0									
PSL	58.6	65.2	60.2	43.7	39.4	42.0	42.3	43.5									

DEFINITION

This indicator reflects the value of Account 154.300. This account reflects materials needed to keep operational the physical equipment and facilities of the plant (e.g., spare parts, consumables, commodities, tools). The information is pulled from SAR Report #G0009R72-501. The PassPort system utilizes SAR for system data reporting.

STATISTICAL SUMMARY

INDUSTRY PERFORMANCE

		Start	End	% Change	YE Target
PTN	Monthly	37.2	36.0	-3.2%	
	Y-T-D	37.4	36.0	-3.7%	≤38M
PSL	Monthly	42.3	43.5	2.8%	
	Y-T-D	39.4	43.5	10.4%	≤38M

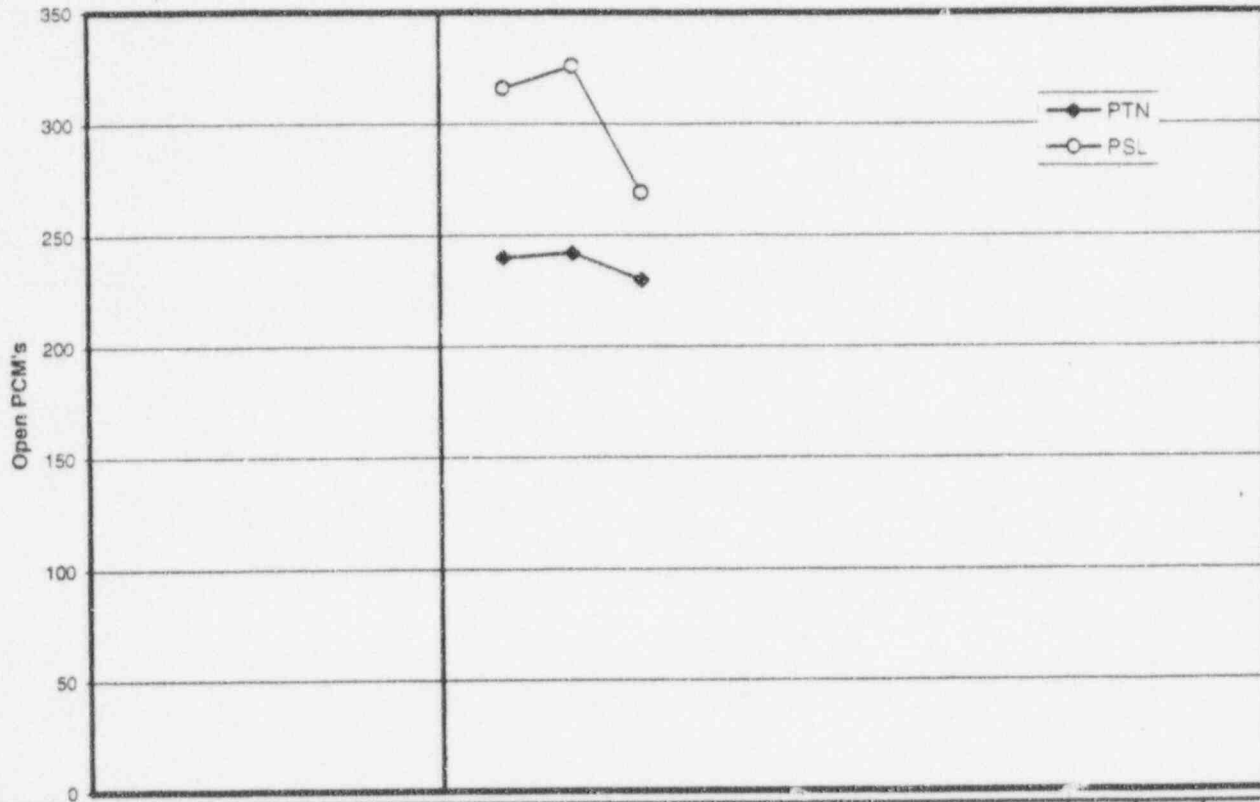
IBG (Year 1994)	*Total
Average	\$43.6 Million
Top Quartile Entry	\$34.7 Million
Top Quartile Average	\$28.9 Million
*Value does not include Capital as defined by PRUC	

PERFORMANCE SUMMARY

- Turkey Point Regular Inventory decreased 1.2M in March.
- St. Lucie's Regular Inventory increased by 1.2M for the month.

Data Providers: (PTN) Dick Rose 246-6692 and (PSL) Tom Kreinberg 465-4183

OPEN PLANT CHANGES / MODIFICATIONS (PC/Ms)



Unit	1991	1992	1993	1994	1995	1/96	2/96	3/96	4/96	5/96	6/96	7/96	8/96	9/96	10/96	11/96	12/96
PTN						240	242	230									
PSL						316	326	269									

DEFINITION

This indicator tracks the total number of PC/M's in the engineering department. The purpose is to provide management a snapshot of the number of PC/M's. Includes all jobs that have been approved by plant management, and are in design, operability review, or implementation phases. The PC/M is considered open until System Acceptance Turnover Sheet (SATS) and drawing update is complete.

STATISTICAL SUMMARY

	Design	Review	Working	Update	Total
PTN	74	14	70	72	230
PSL	42	52	81	94	269

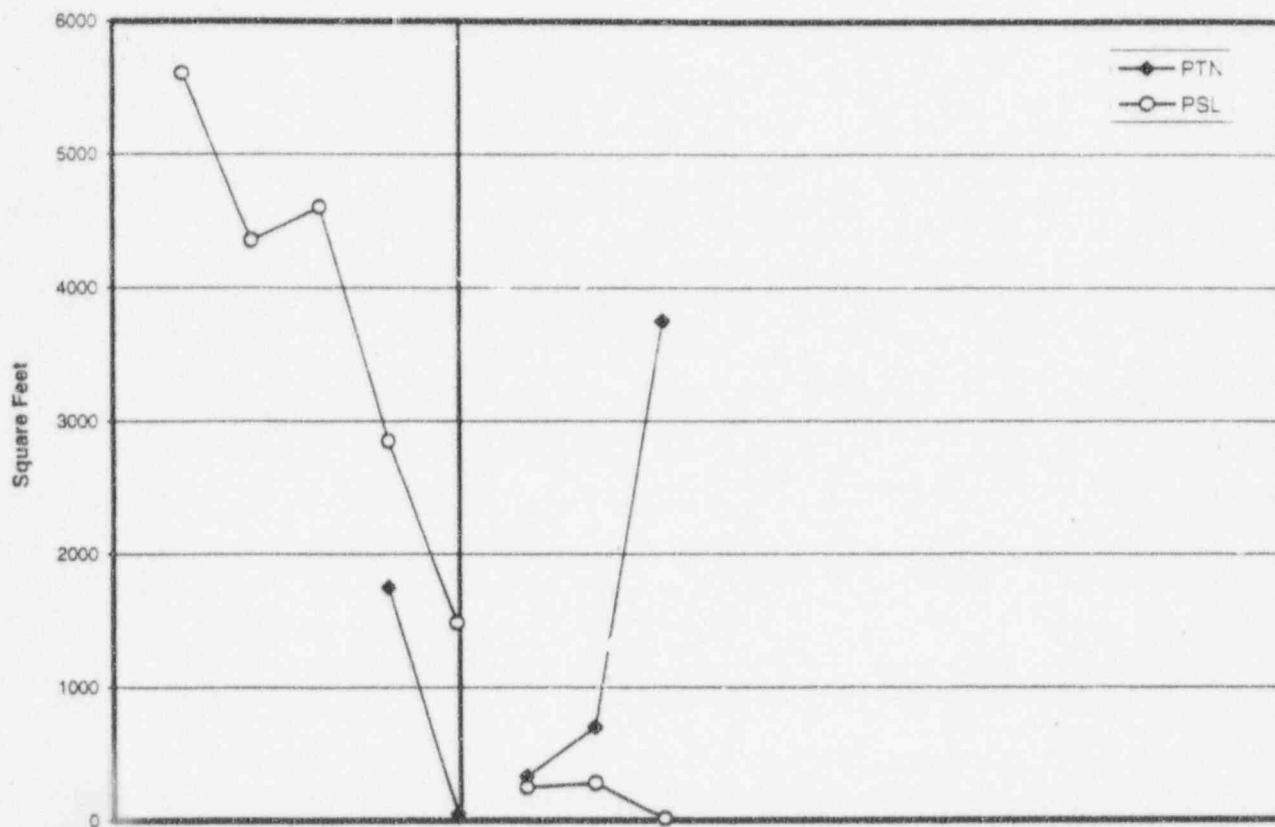
INDUSTRY PERFORMANCE

PERFORMANCE SUMMARY

St. Lucie has excluded the Steam Generator Replacement Project PC/M's which accounts for 20-30 PC/M's. The increase of PC/M's in Review is due to the upcoming PSL1 outage preparation.

Data Providers: (PTN) Jim Reed 246-6903 and (PSL) Kris Mohindroo 467-7482

CONTAMINATED FLOOR SPACE



Year	1991	1992	1993	1994	1995	1/96	2/96	3/96	4/96	5/96	6/96	7/96	8/96	9/96	10/96	11/96	12/96
PTN				1750	50	330	700	3748									
PSL	5610	4351	4600	2848	1480	250	280	12									

DEFINITION

This indicator, designed to measure contaminated floor space with removable activity ≥ 1000 dpm/100cm sq. beta/gamma or ≥ 20 dpm/100cm sq. alpha, is counted against the base. Areas that can be specifically exempted from the base include: reactor containment building, chemical volume control system demineralizer room and long term process areas such as the decontamination facility. Contaminated components such as charging pumps, evaporators, etc. are not included as part of "recoverable" floor space (i.e. not considered floor area you can walk or step on).

PTN: Total Base(117,746 sq. ft.) Exempted Area (6,110 sq. ft.)

PSL: Total Base(112,422 sq. ft.) Exempted Area (7,722 sq. ft.)

STATISTICAL SUMMARY

	Mar	Target
PTN	3748	50
PSL	12	50

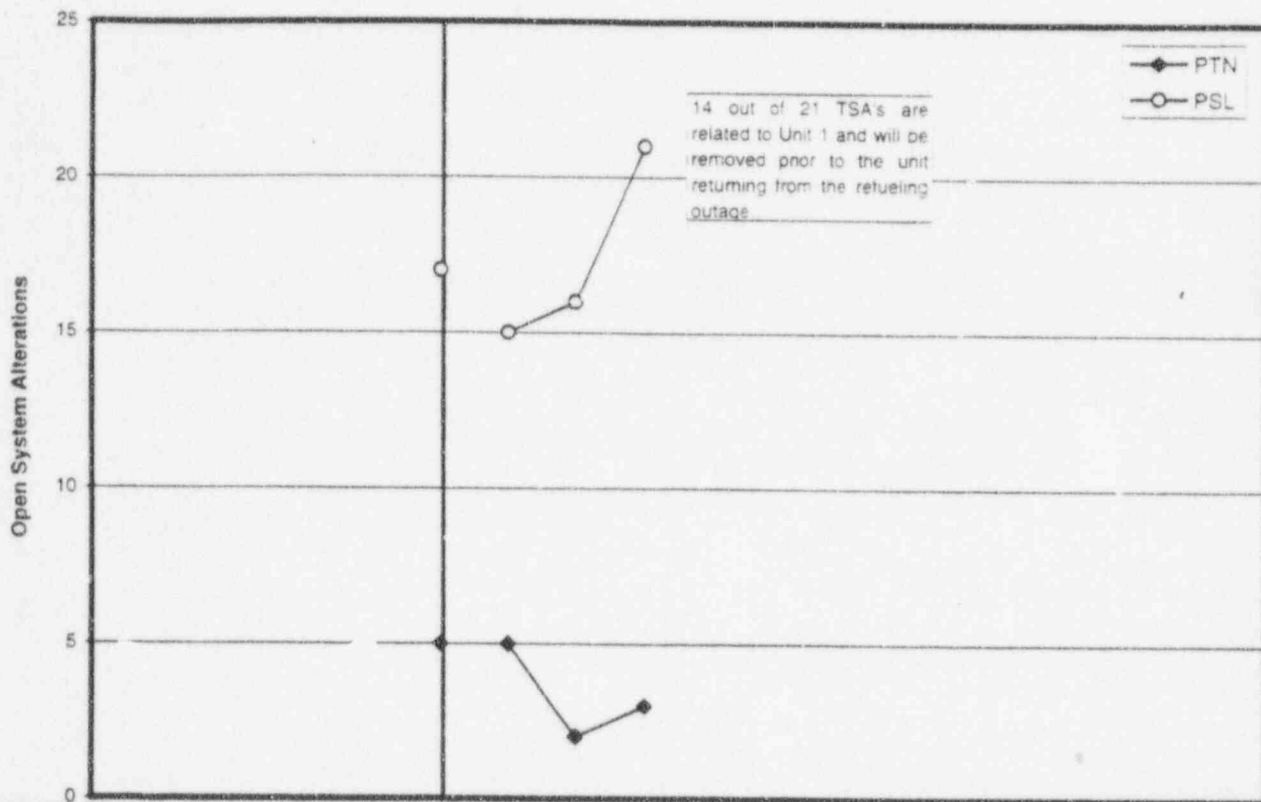
INDUSTRY PERFORMANCE

PERFORMANCE SUMMARY

Turkey Point increase of Contaminated Floor Space is the result of Unit 4 outage in process during March.

Data Providers: (PTN) John Lindsay 246-6548 and (PSL) Hank Buchannon 467-7300

OPEN TEMPORARY SYSTEM ALTERATIONS



Unit	1991	1992	1993	1994	1995	1/96	2/96	3/96	4/96	5/96	6/96	7/96	8/96	9/96	10/96	11/96	12/96
PTN					5	5	2	3									
PSL					17	15	16	21									

DEFINITION

A temporary system alteration is a modification made to plant equipment, components, or systems that does not conform with approved drawings or other design documents; a modification that is necessary for continued safe plant operation; a modification that will remove a nuisance or distraction to the Plant Operators; a modification necessary to enable the plant to start up in a safe manner.

STATISTICAL SUMMARY

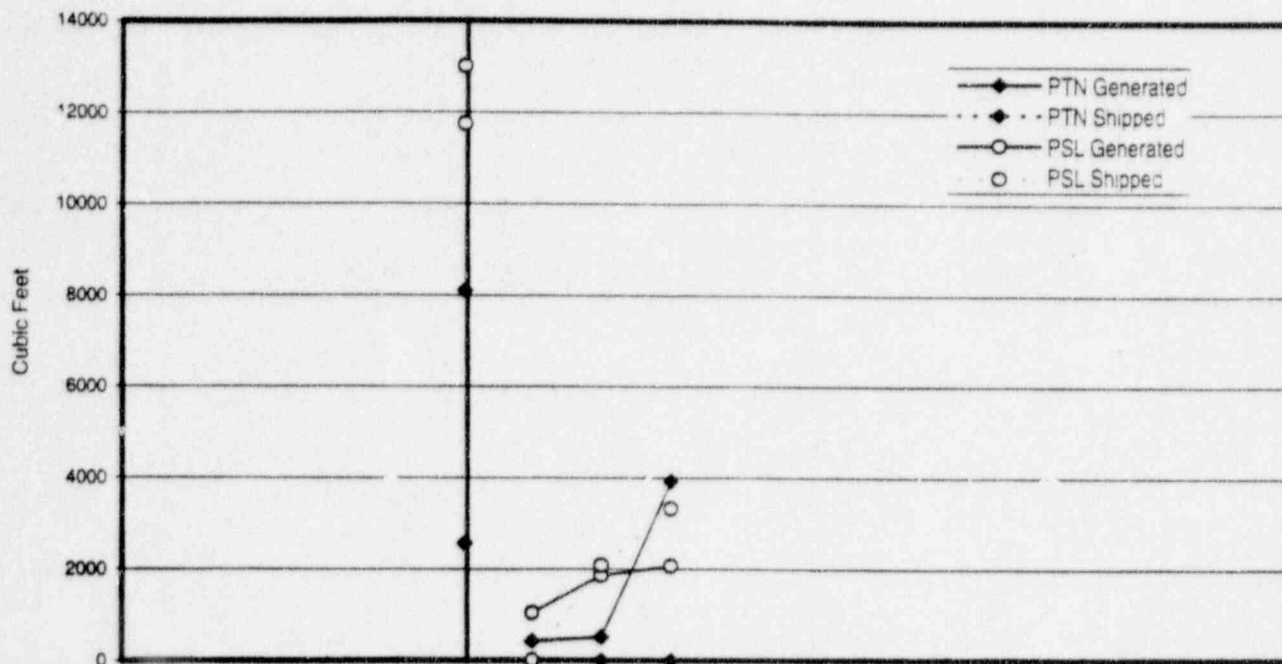
	Mar	Year End Goal
PTN	3	5
PSL	21	10

INDUSTRY PERFORMANCE

PERFORMANCE SUMMARY

Data Providers: (PTN) Julio Balaguero 246-6971 and (PSL) Kris Mohindroo 467-7482

(Year-to-Date)



Unit	1991	1992	1993	1994	1995	1/96	2/96	3/96	4/96	5/96	6/96	7/96	8/96	9/96	10/96	11/96	12/96
PTN Generated					8100	425	515	3925									
PTN Shipped					2560	0	0	0									
PSL Generated					11740	1040	1860	2065									
PSL Shipped					13003	0	2080	3322									

DEFINITION

Generated - Is an estimate based on the number of "yellow bags" initially generated prior to surveying for free release or shipment as radwaste.

Calculation: Number of yellow bags x 5 cubic feet = Estimated Monthly Generated Waste figure.

Shipped offsite - The amount of dry active radioactive waste that FPL ships to either Scientific Ecology Group Inc. (SEG) or American Ecology Recycle Center (AERC) for processing.

STATISTICAL SUMMARY

PSL made 6 shipments to SEG in 1995.

PTN made 3 shipments to SEG in 1995.

INDUSTRY PERFORMANCE

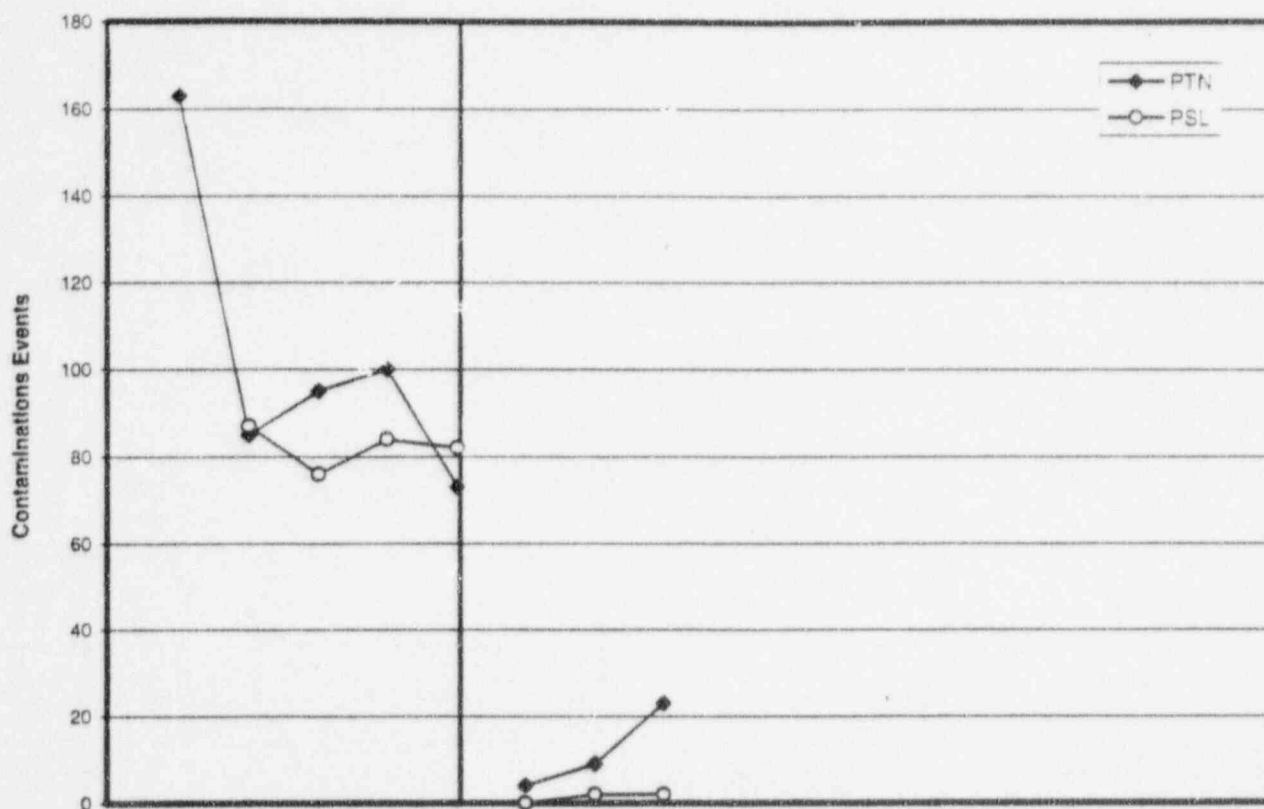
PERFORMANCE SUMMARY

- PTN projects to ship 5,120 cubic feet in 1996.
- Year-to-date, PSL has shipped 3,322 cubic feet of 1995 outage waste.

Data Providers: (PTN) John Lindsay 246-6548 and (PSL) Hank Buchannon 467-7300

PERSONNEL CONTAMINATION EVENTS

(Year-to-Date)



Year	1991	1992	1993	1994	1995	1/96	2/96	3/96	4/96	5/96	6/96	7/96	8/96	9/96	10/96	11/96	12/96
PTN	163	85	95	100	73	4	9	23									
PSL		87	76	84	82	0	2	2									

DEFINITION

This indicator is designed to monitor personnel contamination. A personnel contamination exists when 5000 dpm per 100cm² on skin or personal clothing as detected by Personal Contamination Monitor and ≥ 100 counts per minute (net) using the Frisker is observed.

STATISTICAL SUMMARY

	Mar	Y-T-D
PTN	14	23
PSL	0	2

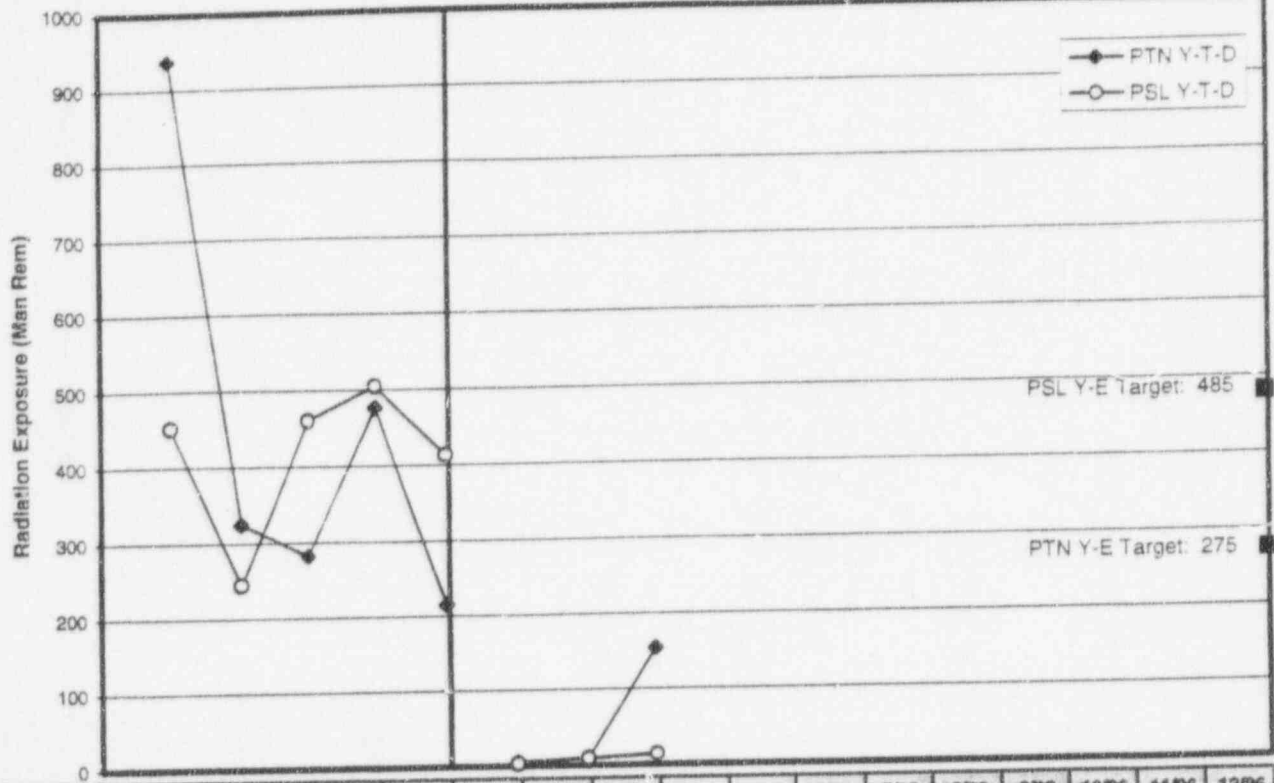
INDUSTRY PERFORMANCE

PERFORMANCE SUMMARY

Data Providers: (PTN) John Lindsay 246-6548 and (PSL) Hank Buchannon 467-7300

RADIATION EXPOSURE

(Year-to-Date)



Unit	1991	1992	1993	1994	1995	1/96	2/96	3/96	4/96	5/96	6/96	7/96	8/96	9/96	10/96	11/96	12/96
PTN Y-T-D	938.0	324.9	281.8	476.2	214.6	5.3	9.5	153.9									
PSL Y-T-D	451.3	244.5	459.9	504.7	412.8	3.0	9.0	13.7									

DEFINITION

Collective Radiation Exposure is the total effective dose equivalent received by all on-site personnel (including contractors and visitors), it includes external deep dose as measured by the thermoluminescent dosimeters (TLD's) plus internal dose. It is reported in man-rem for the station. Current month readings may be taken from the direct reading dosimeters (DRD's).

STATISTICAL SUMMARY

	Mar	YTD	YTD Target	12-Mo. Ending
PTN	144.4*	153.9	225.0	353.5
PSL	4.7*	13.7	33.0	382.6
Year-End Targets:				
PTN: 275.0				
PSL: 485.0				

INDUSTRY PERFORMANCE

WANO (PWR's)	Man-Rem (two units)
1995 Median	306
1995 Goal	370

PERFORMANCE SUMMARY

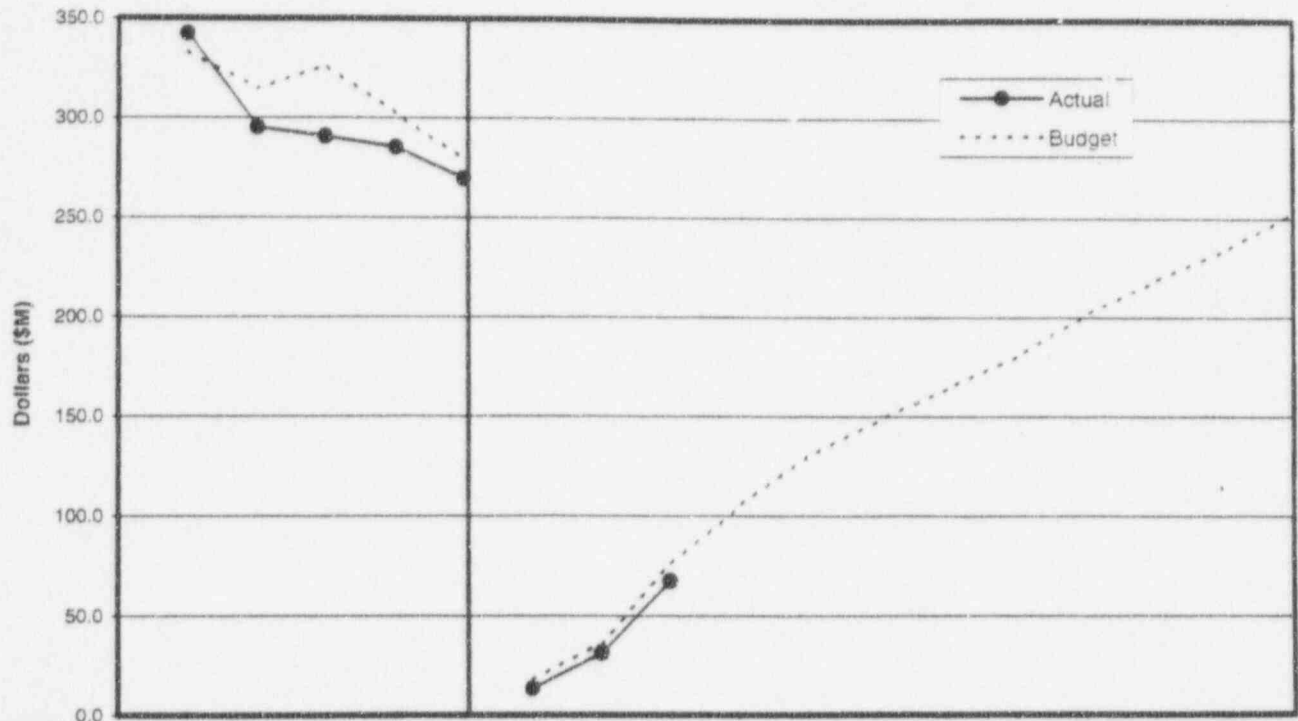
- Turkey Point's performance Y-T-D was below the Y-T-D target; performance for 12-mo.'s ending of 353.5 was higher than the industry median in March.
- St. Lucie's Man-Rem Y-T-D was below the Y-T-D target; performance for 12-mo.'s ending totaled 382.6 which is higher than the industry median.

* DRD count

Data Providers: (PTN) John Lindsay 246-6548 and (PSL) Hank Buchannon 467-7300

O&M BUDGET - DIVISION

(Year-to-Date)



	1991	1992	1993	1994	1995	1/96	2/96	3/96	4/96	5/96	6/96	7/96	8/96	9/96	10/96	11/96	12/96
Actual	342.4	295.2	290.6	285.2	269.5	13.4	31.2	67.4									
Budget	333.3	314.6	326.4	302.0	279.2	17.8	36.7	75.8	104.8	129.6	147.4	164.0	180.0	201.2	218.0	233.9	252.8
Variance (%)	2.7	-6.2	-11.0	-5.6	-3.5	-24.9	-14.9	-11.1									

DEFINITION

Operating and Maintenance Expenditures include Nuclear Division operation and maintenance expenses associated with direct employees, contractors and consultants, equipment, tools, design, engineering and other items/activities required to sustain the electrical generation of the plants and to provide required support. Fuel costs, corporate administrative and general expenses, and charges from other departments outside the Nuclear Division are excluded.

$$\frac{\text{Y-T-D Actual Expenses} - \text{Y-T-D Budgeted Expenses}}{\text{Y-T-D Budgeted Expenses}} \times 100\% = \text{O\&M Variance \%}$$

STATISTICAL SUMMARY

	Y-T-D Actual (\$M)	Y-T-D Variance (\$M)	%
Jan '96	67.4 (\$M)	- 8.4 (\$M)	-11.1%
Jan '95	45.8 (\$M)	-8.4 (\$M)	-15.6%

1996 Y-E Budget: 252.8 (\$M)

INDUSTRY PERFORMANCE

	O&M Budget
IBG Actual 1994 Avg.	\$297.0M
IBG Top Quartile Entry	\$270.0M
IBG Top Quartile Avg.	\$213.2M
IBG Projected 1995 Avg.	\$284.0M

Projection (per dual unit site) derived by trending 1986-94 Actual data and 1995 Budgeted data for IBG Group.

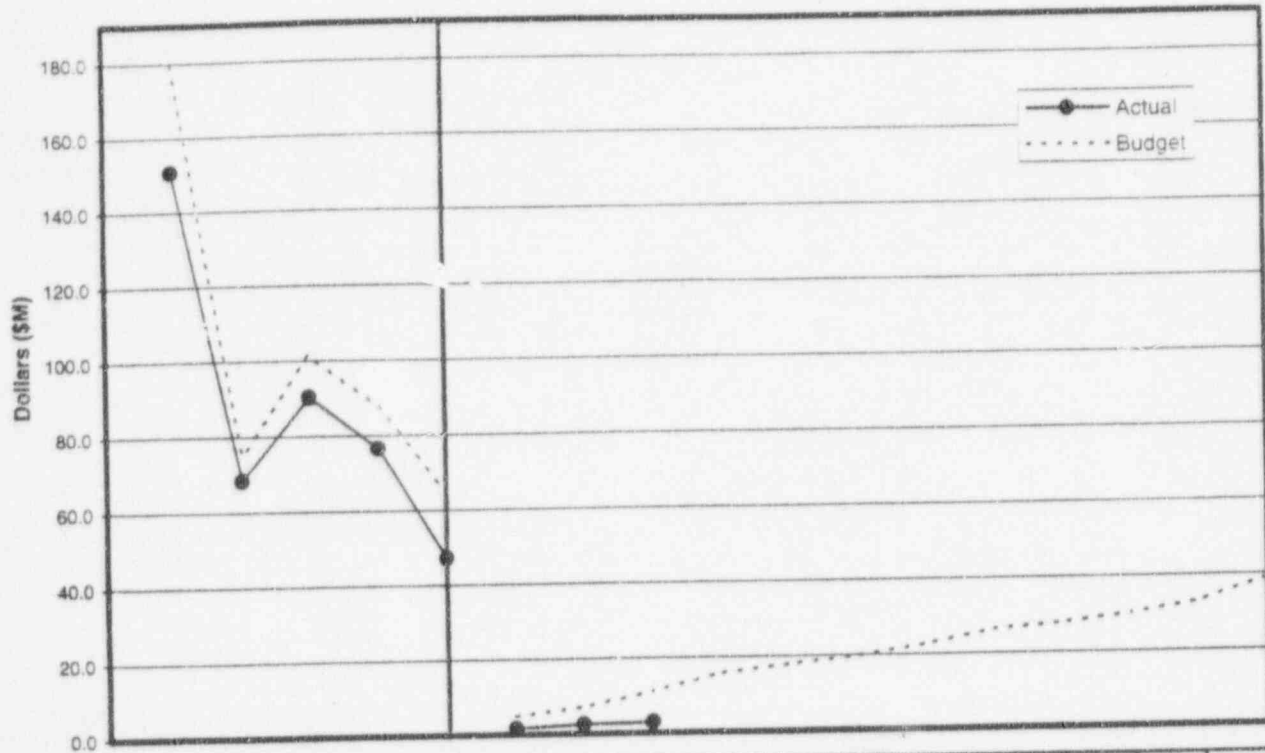
PERFORMANCE SUMMARY

O&M Expenditures through March 1996 were \$67.4 million which represented a budget underrun of \$8.4 million (or 11.1%). The variance was the result of: St. Lucie Unit 1 outage schedule change; St. Lucie non-outage project underruns; and overtime payroll underrun associated with Turkey Point Unit 4 outage.

Data provider: T. O. Nasby 694-4188

CAPITAL BUDGET - DIVISION

(Year-to-Date)



	1991	1992	1993	1994	1995	1/96	2/96	3/96	4/96	5/96	6/96	7/96	8/96	9/96	10/96	11/96	12/96
Actual	150.7	68.4	90.3	76.5	47.2	1.5	2.5	2.9									
Budget	179.6	75.1	101.4	87.7	64.7	4.8	7.1	11.1	15.7	17.9	19.8	22.6	26.3	27.8	30.1	33.1	39.2
Variance (%)	-16.1	-9.0	-11.0	-12.7	-27.0	-67.6	-65.0	-73.7									

DEFINITION

Capital Expenditures are those directly incurred/budgeted by the Nuclear Division for the construction of new utility plant additions and improvements made to increase efficiency, reliability or safety. Capital fuel costs are excluded.

$$\frac{\text{Y-T-D Actual Expenses} - \text{Y-T-D Budgeted Expenses}}{\text{Y-T-D Budgeted Expenses}} \times 100\% = \text{Capital Variance } \%$$

STATISTICAL SUMMARY				INDUSTRY PERFORMANCE	
	Y-T-D Actual (\$M)	Y-T-D Variance (\$M)	%		Capital Budget
Jan '96	2.9 (\$M)	-8.2 (\$M)	-73.7%	IBG Actual 1994 Avg.	\$63.1M
Jan '95	6.0 (\$M)	-4.8 (\$M)	-44.2%	IBG Top Quartile Entry	\$28.3M
				IBG Top Quartile Avg.	\$21.1M
				IBG Projected 1995 Avg.	\$58.5M
1996 Y-E Budget: 39.2 (\$M)				Projection (per dual unit site) derived by trending 1986-94 Actual data and 1995 Budgeted data for IBG Group.	

PERFORMANCE SUMMARY

Capital Expenditures through March 1996 were \$2.9 million which represented a budget underrun of \$8.2 million (or 73.7%). The variance was primarily due to: underrun in Steam Generator Replacement Project (SGRP) reflecting a revision to the SGRP outage start date; St. Lucie Unit 2 Reactor Head Seal Ring Replacement Project determined to be O&M; and plant cancelled projects.

Data provider: T. O. Nasby 694-4188

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