



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D.C. 20555-0001

SOUTHERN NUCLEAR OPERATING COMPANY, INC.

GEORGIA POWER COMPANY

OGLETHORPE POWER CORPORATION

MUNICIPAL ELECTRIC AUTHORITY OF GEORGIA

CITY OF DALTON, GEORGIA

DOCKET NO. 50-321

EDWIN I. HATCH NUCLEAR PLANT, UNIT 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 206  
License No. DPR-57

1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment to the Edwin I. Hatch Nuclear Plant, Unit 1 (the facility) Facility Operating License No. DPR-57 filed by the Georgia Power Company, and Southern Nuclear Operating Company, Inc. (Southern Nuclear), acting for themselves, Oglethorpe Power Corporation, Municipal Electric Authority of Georgia, and City of Dalton, Georgia (the licensees), dated September 19, 1996, as supplemented December 17, 1996, January 23 and 31, March 21 and April 4, 1997, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations as set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations set forth in 10 CFR Chapter I;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and

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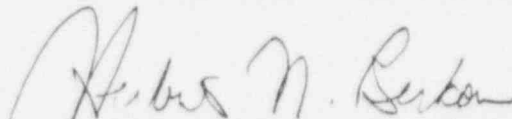
- E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is hereby amended by page changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-57 is hereby amended to read as follows:

Technical Specifications

The Technical Specifications contained in Appendix A and the Environmental Protection Plan contained in Appendix B, as revised through Amendment No. 206, are hereby incorporated in the license. Southern Nuclear shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. This license amendment is effective as of its date of issuance and shall be implemented within 30 days from the date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Herbert N. Berkow, Director  
Project Directorate II-2  
Division of Reactor Projects - I/II  
Office of Nuclear Reactor Regulation

Attachment:  
Technical Specification  
Changes

Date of Issuance: April 4, 1997

ATTACHMENT TO LICENSE AMENDMENT NO. 206

FACILITY OPERATING LICENSE NO. DPR-57

DOCKET NO. 50-321

Replace the following pages of the Appendix "A" Technical Specifications with the enclosed pages. The revised pages are identified by Amendment number and contain vertical lines indicating the areas of change.

Remove

3.4-22  
3.4-23  
3.4-24  
3.4-25  
3.4-26  
3.4-27

Insert

3.4-22  
3.4-23  
3.4-24  
3.4-25  
3.4-26  
3.4-27



SURVEILLANCE REQUIREMENTS (continued)

SURVEILLANCE	FREQUENCY
<p>SR 3.4.9.2 -----NOTE----- Only required to be met when the reactor is critical and immediately prior to control rod withdrawal for the purpose of achieving criticality. -----</p> <p>Verify RCS pressure and RCS temperature are within the criticality limits specified in Figure 3.4.9-3.</p>	<p>Once within 15 minutes prior to initial control rod withdrawal for the purpose of achieving criticality</p>
<p>SR 3.4.9.3 -----NOTE----- Only required to be met in MODES 1, 2, 3, and 4 during startup of a recirculation pump. -----</p> <p>Verify the difference between the bottom head coolant temperature and the reactor pressure vessel (RPV) coolant temperature is <math>\leq 145^{\circ}\text{F}</math>.</p>	<p>Once within 15 minutes prior to starting an idle recirculation pump</p>
<p>SR 3.4.9.4 -----NOTE----- Only required to be met in MODES 1, 2, 3, and 4 during startup of a recirculation pump. -----</p> <p>Verify the difference between the reactor coolant temperature in the recirculation loop to be started and the RPV coolant temperature is <math>\leq 50^{\circ}\text{F}</math>.</p>	<p>Once within 15 minutes prior to starting an idle recirculation pump</p>

(continued)

SURVEILLANCE REQUIREMENTS (continued)

SURVEILLANCE	FREQUENCY
<p>SR 3.4.9.5 -----NOTE----- Only required to be met when tensioning/ detensioning the reactor vessel head bolting studs. -----</p> <p>Verify reactor vessel flange and head flange temperatures are <math>\geq 76^{\circ}\text{F}</math>.</p>	<p>Once within 30 minutes prior to tensioning/ detensioning the reactor vessel head bolting studs and every 30 minutes thereafter</p>
<p>SR 3.4.9.6 -----NOTE----- Only required to be met when the reactor vessel head is tensioned. -----</p> <p>Verify reactor vessel flange and head flange temperatures are <math>\geq 76^{\circ}\text{F}</math>.</p>	<p>Once within 12 hours after RCS temperature is <math>\leq 106^{\circ}\text{F}</math> in MODE 4, and 12 hours thereafter</p> <p><u>AND</u></p> <p>Once within 30 minutes after RCS temperature is <math>\leq 86^{\circ}\text{F}</math> in MODE 4, and 30 minutes thereafter</p>

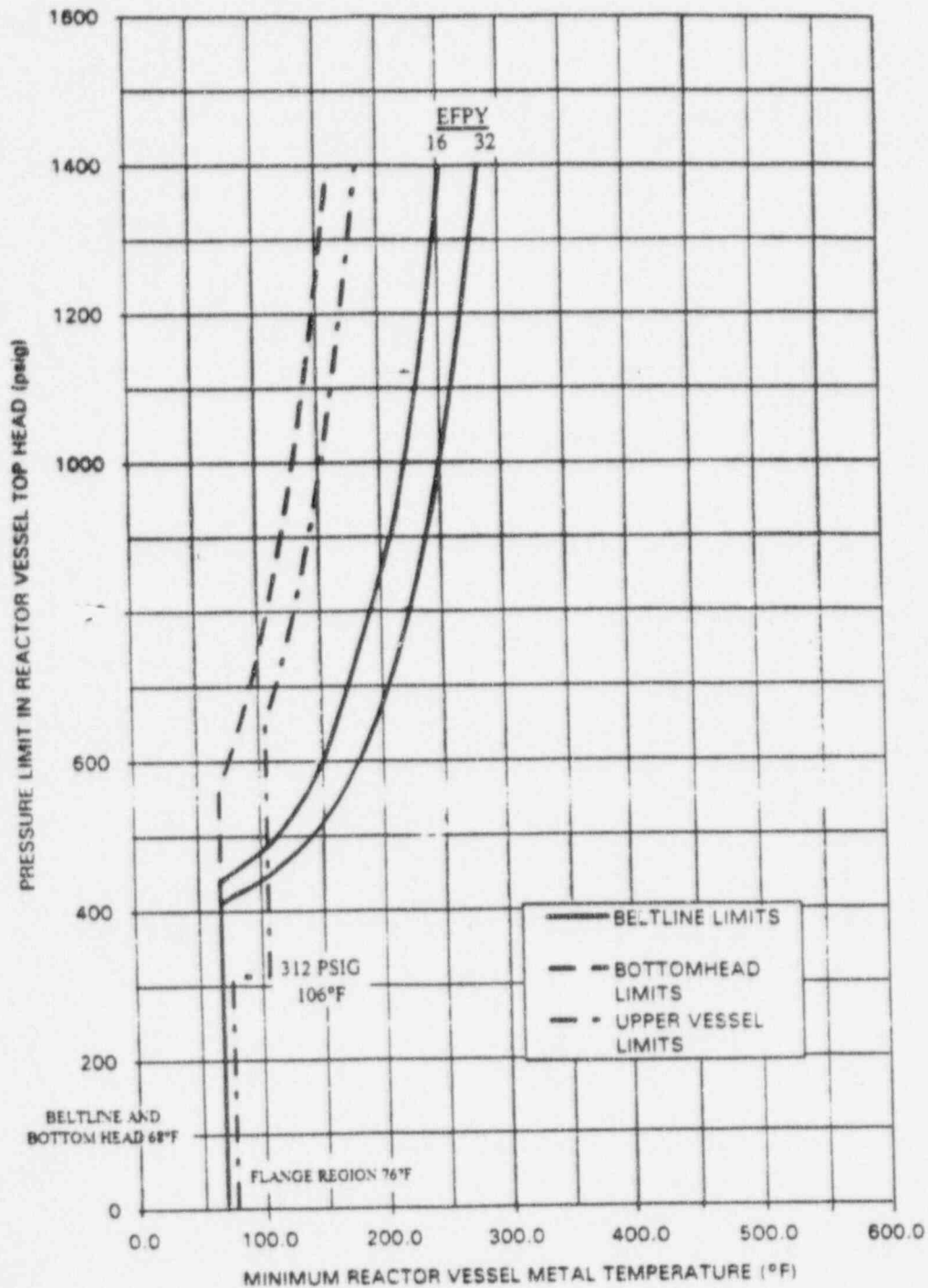


Figure 3.4.9-1 (page 1 of 1)  
Pressure/Temperature Limits for  
Inservice Hydrostatic and Inservice Leakage Tests

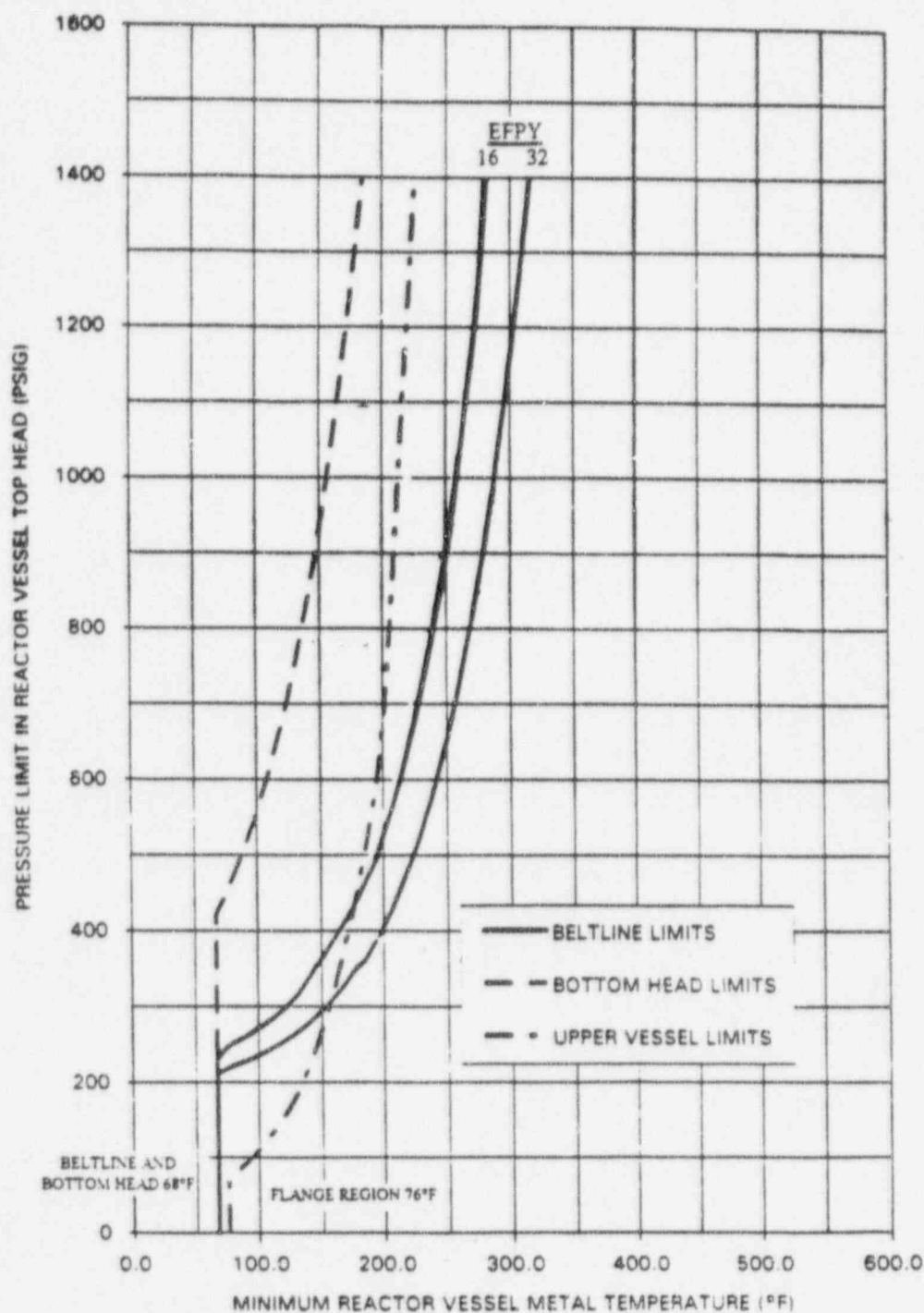


Figure 3.4.9-2 (page 1 of 1)  
Pressure/Temperature Limits for Non-Nuclear Heatup,  
Low Power Physics Tests, and Cooldown Following a Shutdown



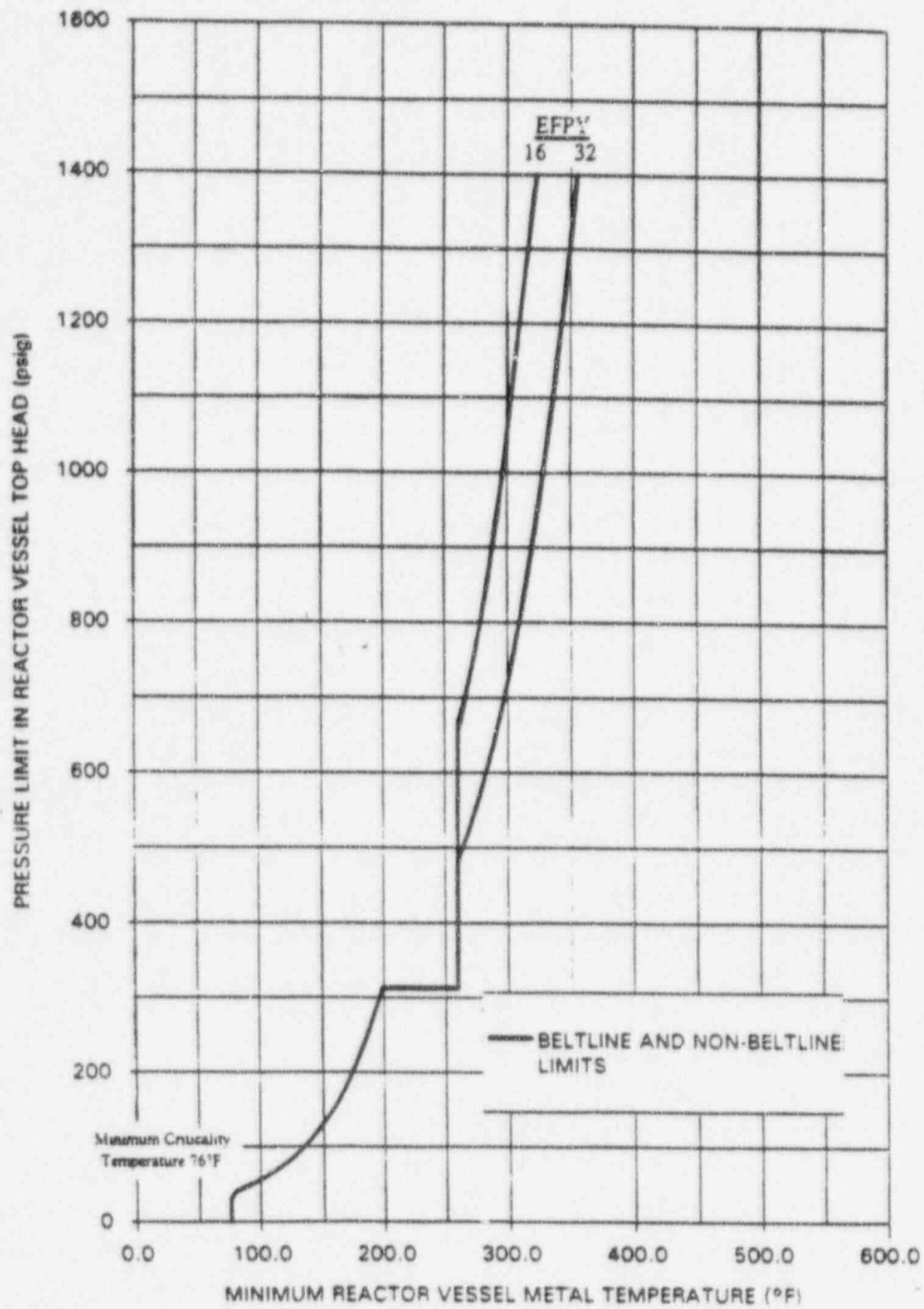


Figure 3.4.9-3 (page 1 of 1)  
Pressure/Temperature Limits for Criticality



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DOCKET NO. 50-366

EDWIN I. HATCH NUCLEAR PLANT, UNIT 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 147  
License No. NPF-5

1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment to the Edwin I. Hatch Nuclear Plant, Unit 2 (the facility) Facility Operating License No. NPF-5 filed by the Georgia Power Company, and Southern Nuclear Operating Company, Inc. (Southern Nuclear), acting for themselves, Oglethorpe Power Corporation, Municipal Electric Authority of Georgia, and City of Dalton, Georgia (the licensees), dated September 19, 1996, as supplemented December 17, 1996, January 23 and 31, March 21 and April 4, 1997, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations as set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations set forth in 10 CFR Chapter I;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and

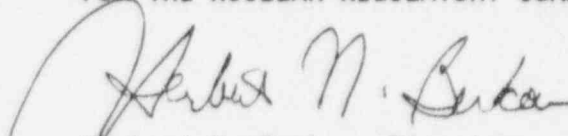
- E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is hereby amended by page changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. NPF-5 is hereby amended to read as follows:

Technical Specifications

The Technical Specifications contained in Appendix A and the Environmental Protection Plan contained in Appendix B, as revised through Amendment No. 147 are hereby incorporated in the license. Southern Nuclear shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. This license amendment is effective as of its date of issuance and shall be implemented within 30 days from the date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Herbert N. Berkow, Director  
Project Directorate II-2  
Division of Reactor Projects - I/II  
Office of Nuclear Reactor Regulation

Attachment:  
Technical Specification  
Changes

Date of Issuance: April 4, 1997

ATTACHMENT TO LICENSE AMENDMENT NO. 147

FACILITY OPERATING LICENSE NO. NPF-5

DOCKET NO. 50-366

Replace the following pages of the Appendix "A" Technical Specifications with the enclosed pages. The revised pages are identified by Amendment number and contain vertical lines indicating the areas of change.

Remove

3.4-22  
3.4-23  
3.4-24  
3.4-25  
3.4-26  
3.4-27

Insert

3.4-22  
3.4-23  
3.4-24  
3.4-25  
3.4-26  
3.4-27

ACTIONS (continued)

CONDITION	REQUIRED ACTION	COMPLETION TIME
C. -----NOTE----- Required Action C.2 shall be completed if this Condition is entered. -----	C.1      Initiate action to restore parameter(s) to within limits.	Immediately
	<u>AND</u>	
Requirements of the LCO not met in other than MODES 1, 2, and 3.	C.2      Determine RCS is acceptable for operation.	Prior to entering MODE 2 or 3

## SURVEILLANCE REQUIREMENTS

SURVEILLANCE	FREQUENCY
<p>SR 3.4.9.1    Verify:</p> <ul style="list-style-type: none"> <li>a.    RCS pressure and RCS temperature are within the limits specified in Figures 3.4.9-1 and 3.4.9-2 during RCS inservice leak and hydrostatic testing, and during RCS non-nuclear heatup and cooldown operations; and</li> <li>b.    RCS heatup and cooldown rates are <math>\leq 100^{\circ}\text{F}</math> in any 1 hour period during RCS heatup and cooldown operations, and RCS inservice leak and hydrostatic testing.</li> </ul>	<p>30 minutes</p>

(continued)

SURVEILLANCE REQUIREMENTS (continued)

SURVEILLANCE	FREQUENCY
<p>SR 3.4.9.2 -----NOTE----- Only required to be met when the reactor is critical and immediately prior to control rod withdrawal for the purpose of achieving criticality.</p> <p>Verify RCS pressure and RCS temperature are within the criticality limits specified in Figure 3.4.9-3.</p>	<p>Once within 15 minutes prior to initial control rod withdrawal for the purpose of achieving criticality</p>
<p>SR 3.4.9.3 -----NOTE----- Only required to be met in MODES 1, 2, 3, and 4 during startup of a recirculation pump.</p> <p>Verify the difference between the bottom head coolant temperature and the reactor pressure vessel (RPV) coolant temperature is <math>\leq 145^{\circ}\text{F}</math>.</p>	<p>Once within 15 minutes prior to starting an idle recirculation pump</p>
<p>SR 3.4.9.4 -----NOTE----- Only required to be met in MODES 1, 2, 3, and 4 during startup of a recirculation pump.</p> <p>Verify the difference between the reactor coolant temperature in the recirculation loop to be started and the RPV coolant temperature is <math>\leq 50^{\circ}\text{F}</math>.</p>	<p>Once within 15 minutes prior to starting an idle recirculation pump</p>

(continued)

SURVEILLANCE REQUIREMENTS (continued)

SURVEILLANCE	FREQUENCY
<p>SR 3.4.9.5 -----NOTE----- Only required to be met when tensioning/ detensioning the reactor vessel head bolting studs. -----</p> <p>Verify reactor vessel flange and head flange temperatures are <math>\geq 90^{\circ}\text{F}</math>.</p>	<p>Once within 30 minutes prior to tensioning/ detensioning the reactor vessel head bolting studs and every 30 minutes thereafter</p>
<p>SR 3.4.9.6 -----NOTE----- Only required to be met when the reactor vessel head is tensioned. -----</p> <p>Verify reactor vessel flange and head flange temperatures are <math>\geq 90^{\circ}\text{F}</math>.</p>	<p>Once within 12 hours after RCS temperature is <math>\leq 120^{\circ}\text{F}</math> in MODE 4, and 12 hours thereafter</p> <p><u>AND</u></p> <p>Once within 30 minutes after RCS temperature is <math>\leq 100^{\circ}\text{F}</math> in MODE 4, and 30 minutes thereafter</p>

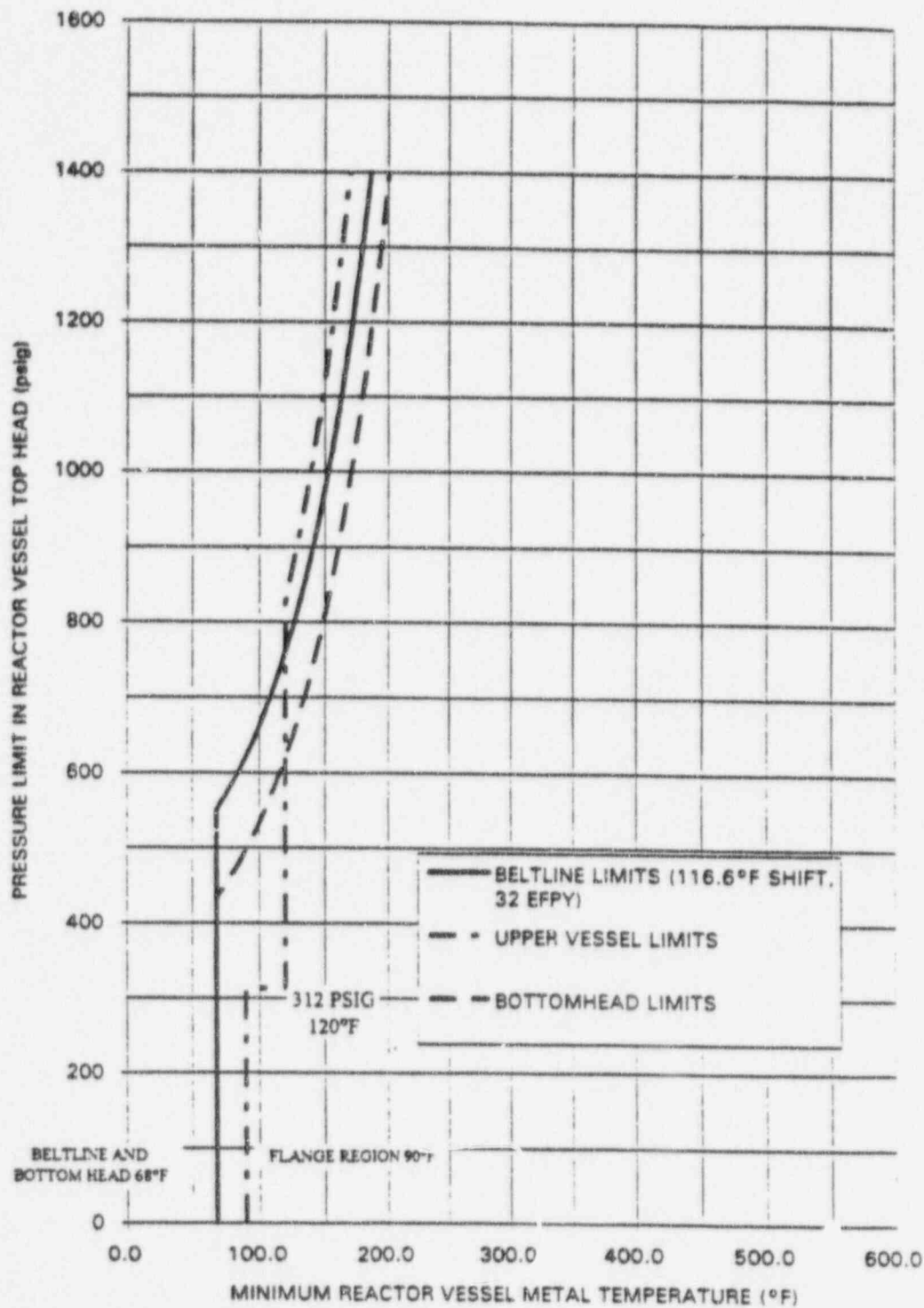


Figure 3.4.9-1 (page 1 of 1)  
Pressure/Temperature Limits for  
Inservice Hydrostatic and Inservice Leakage Tests



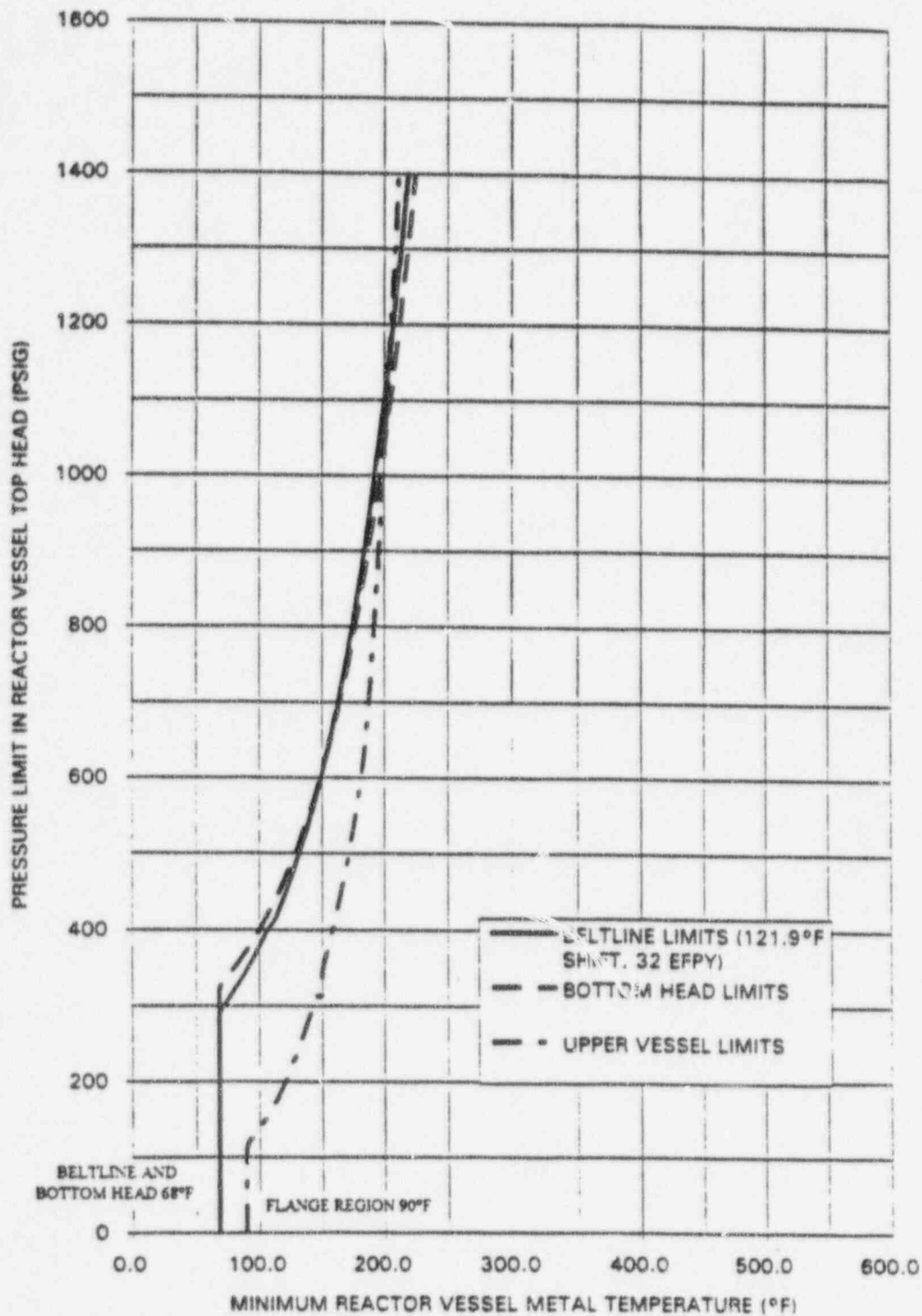


Figure 3.4.9-2 (page 1 of 1)  
Pressure/Temperature Limits for Non-Nuclear Heatup,  
Low Power Physics Tests, and Cooldown Following a Shutdown

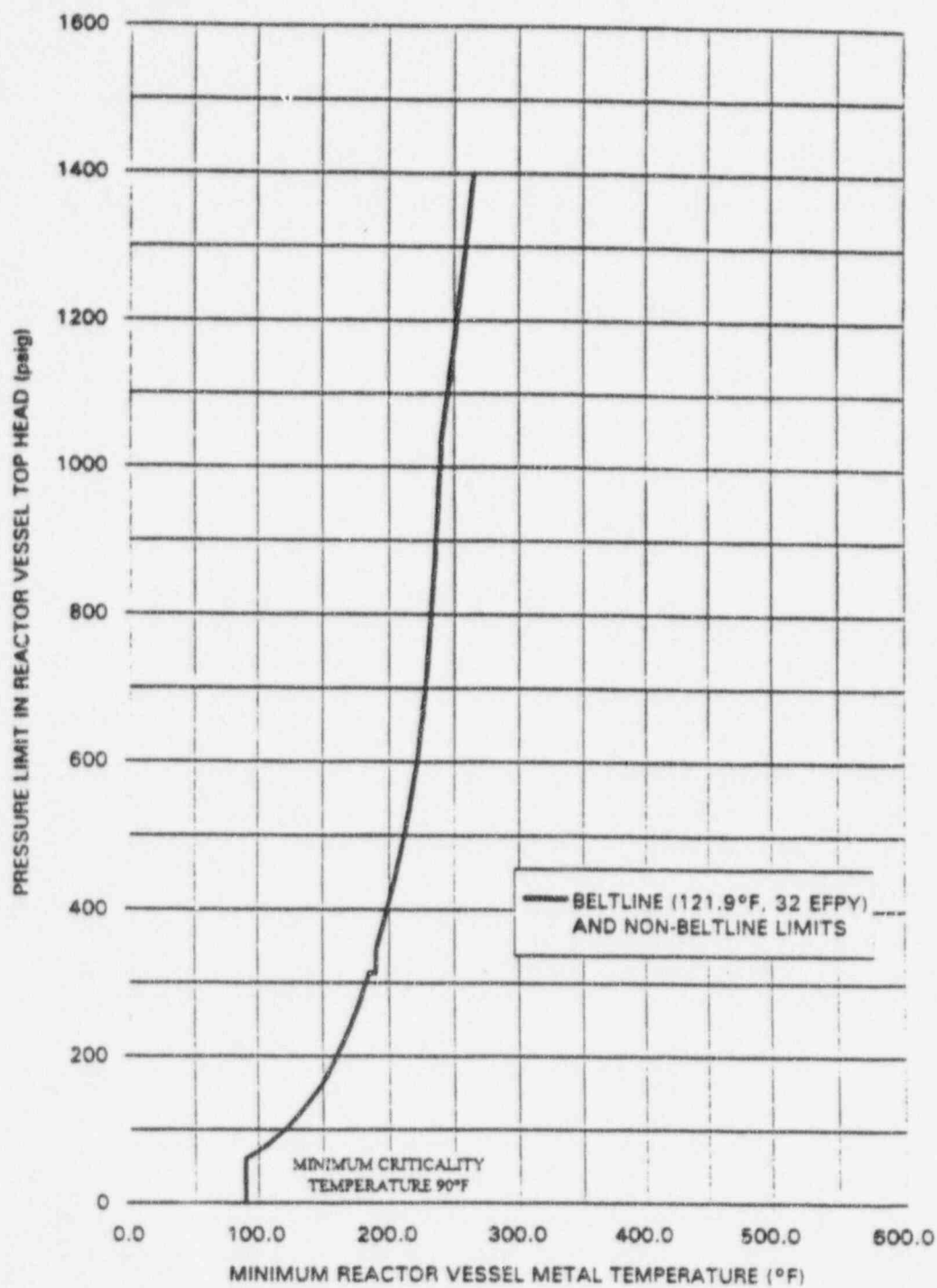


Figure 3.4.9-3 (page 1 of 1)  
Pressure/Temperature Limits for Criticality