

ATTACHMENT I TO JPN-86-01

PROPOSED TECHNICAL SPECIFICATIONS CHANGES RELATED
TO SAFETY REVIEW COMMITTEE ALTERNATE VICE-CHAIRMAN
(JPTS-85-13)

NEW YORK POWER AUTHORITY
JAMES A. FITZPATRICK NUCLEAR POWER PLANT
DOCKET NO. 50-333
DPR-59

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- f. Radiological safety
- g. Mechanical engineering
- h. Electrical engineering
- i. Administrative controls and quality assurance practices
- j. Environment
- k. Civil/Structural Engineering
- l. Nuclear Licensing
- m. Emergency Planning
- n. Other appropriate fields associated with the unique characteristics of a nuclear power plant

MEMBERSHIP

6.5.2.2 The SRC shall be composed of the following voting members:

Chairman:	Manager-Nuclear Safety Evaluation
Vice-Chairman:	Director-Quality Assurance
Member:	Vice President Nuclear Support-BWR
Member:	Vice President Nuclear Support-PWR
Member:	Vice President-Generic Nuclear Support
Member:	Vice President-Design and Analysis
Member:	Resident Manager-IP3
Member:	Resident Manager-JAF
Member:	Consultant

ALTERNATES

6.5.2.3 All alternate members shall be appointed in writing by the SRC chairman. An Alternate Vice-Chairman shall be appointed in writing by the Executive Vice President - Nuclear Generation if necessary. However, no more than two alternates shall participate as voting members in SRC activities at any one time.

CONSULTANTS

6.5.2.4 Consultants shall be utilized as determined by the SRC Chairman to provide expert advice to the SRC.

MEETING FREQUENCY

6.5.2.5 The SRC shall meet at least once per calendar quarter during the initial year of facility operation following initial fuel loading and at least once per six months, thereafter.

QUORUM

6.5.2.6 A quorum of SRC shall consist of the Chairman, Vice-Chairman or Alternate Vice-Chairman and four members, including alternates. No more than a minority of the quorum shall have a direct line responsibility for the operation of the plant.

REVIEW

6.5.2.7 The SRC shall review:

- a. The safety evaluation for 1) changes to procedures, equipment or systems and 2) tests or experiments completed under the provision of Section 50.59, 10 CFR, to verify that such actions did not constitute an unreviewed safety question.
- b. Proposed changes to procedures, equipment or systems which involve an unreviewed safety question as defined in Section 50.59, 10 CFR.
- c. Proposed tests or experiments which involve an unreviewed safety question as defined in Section 50.59, 10 CFR.
- d. Proposed changes to Technical Specifications of this Operating License.
- e. Violations of code, regulations, orders, Technical Specifications, license requirements, or of internal procedures or instructions having nuclear safety significance.
- f. Significant operating abnormalities or deviations from normal and expected performance of plant equipment that affect nuclear safety.
- g. Events requiring 24 hour written notification to the Commission.
- h. All recognized indications of an unanticipated deficiency in some aspect of design or operation of safety related structures, systems or components.
- i. Reports and meetings minutes of the Plant Operating Review Committee.

AUDIT

6.5.2.8 Audits of facility activities shall be performed under the cognizance of the SRC. These audits shall encompass:

- a. The conformance of facility operation to provisions contained within the Technical Specifications and applicable license conditions at least once per 12 months.
- b. The performance, training and qualifications of the entire facility staff at least once per 12 months.

ATTACHMENT II TO JPN-86-01

SAFETY EVALUATION FOR
PROPOSED TECHNICAL SPECIFICATIONS CHANGES RELATED
TO SAFETY REVIEW COMMITTEE ALTERNATE VICE-CHAIRMAN
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Section I - Description of the Proposed Changes

The proposed changes to the Technical Specifications are shown in Attachment I to the Application for Amendment. The changes are as follows:

- a) Section 6.5.2.3, "Alternates", on page 251, shall incorporate an Alternate Vice-Chairman as a member to participate in SRC activities. Specifically, a new second sentence states; "An Alternate Vice Chairman shall be appointed in writing by the Executive Vice President - Nuclear Generation if necessary."
- b) Section 6.5.2.6, "Quorum", on page 252 shall include the words, "or Alternate Vice-Chairman" in the first sentence.

Section II - Purpose of the Proposed Changes

The purpose of the proposed changes is to facilitate Safety Review Committee meetings at headquarters, should the Chairman and Vice-Chairman be unable to attend, by providing an Alternate Vice-Chairman.

The inclusion of the Alternate Vice-Chairman as a voting member is consistent with the current Technical Specifications which allow up to two alternate voting members in SRC activities at any one time. The duties and responsibilities of the Alternate Vice-Chairman are identical to those of the Vice-Chairman.

Section III - Impact of the Proposed Changes

The proposed change is purely an administrative change to the SRC organization and cannot have any impact on plant safety. The proposed change would not, therefore, involve significant hazards considerations since it does not:

- (1) involve a significant increase in the probability or consequences of an accident previously evaluated; or create the possibility of a new or different kind of accident from any accident previously evaluated, because it is strictly administrative in nature.
- (2) involve a significant reduction in a margin of safety, because it does not result in any change in SRC functions.

The proposed changes do not change any system or subsystem and will not alter the conclusions of either the FSAR or SER accident analyses.

The Commission has provided guidance concerning the application of the standards for making a "no significant hazard considerations" determination by providing certain examples in the Federal Register (F.R.) Vol. 48, No. 67 dated April 6, 1984, page 14870. The proposed changes to the Technical Specifications match Commission example (i): "A purely administrative change to technical specifications...".

Section IV - Implementation of the Proposed Changes

Implementation of the proposed changes will not impact the ALARA or fire protection programs at FitzPatrick, nor will the changes impact the environment.

Section V - Conclusion

The change, as proposed, does not constitute an unreviewed safety question as defined in 10 CFR 50.59, that is, it (a) will not increase the probability or the consequences of an accident or malfunction of equipment important to safety as evaluated previously in the Safety Analysis Report (SAR); (b) will not increase the possibility of an accident or malfunction of a type other than that evaluated previously in the SAR; (c) will not reduce the margin of safety as defined in the basis for any Technical Specification; (d) does not constitute an unreviewed safety question; and (e) involves no significant hazards considerations, as defined in 10 CFR 50.92.

Section VI - References

- 1) James A. FitzPatrick Nuclear Power Plant Final Safety Analysis Report (FSAR), Rev. 3, July 1985.