

## LICENSEE EVENT REPORT (LER)

FACILITY NAME (1)  
Nine Mile Point Unit IDOCKET NUMBER (2)  
0 5 0 0 0 2 2 1 0 1 OF 0 2

TITLE (4)

Initiation of Reactor Building Emergency Ventilation System

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)		
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES		DOCKET NUMBER(S)
08	15	85	85	010	00	09	13	85			0 5 0 0 0
											0 5 0 0 0

OPERATING MODE (9)		THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: (Check one or more of the following) (11)									
POWER LEVEL (10) 0 9 9		20.402(b)		20.406(c)	<input checked="" type="checkbox"/>	50.73(a)(2)(iv)		73.71(b)			
		20.405(a)(1)(ii)		50.36(c)(1)		50.73(a)(2)(v)		73.71(e)			
		20.405(a)(1)(iii)		50.36(c)(2)		50.73(a)(2)(vii)		OTHER (Specify in Abstract below and in Text, NRC Form 366A)			
		20.405(a)(1)(iv)		50.73(a)(2)(ii)		50.73(a)(2)(viii)(A)					
		20.405(a)(1)(v)		50.73(a)(2)(iii)		50.73(a)(2)(viii)(B)					

LICENSEE CONTACT FOR THIS LER (12)  
NAME  
Robert Randall, Supervisor Technical SupportTELEPHONE NUMBER  
AREA CODE  
3 1 1 5 3 4 9 1 - 1 2 4 1 5

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)										
CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPDs	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPDs	
X	I	L	R	E						
			G	O	S					

SUPPLEMENTAL REPORT EXPECTED (14)  
YES (If yes, complete EXPECTED SUBMISSION DATE) ☒ NO ☐  
EXPECTED SUBMISSION DATE (15)  
MONTH DAY YEAR

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

## ABSTRACT

During normal operation on August 15, 1985 an accidental initiation of the Reactor Building Emergency Ventilation System occurred. The system was initiated by an electrical spike experienced by channel 12 Reactor Building Ventilation Duct Monitor. The spike occurred when the drywell CAM was being pulled out from control room panel J for inspection. It bumped the adjoining channel 12 Reactor Building Ventilation Monitor causing an electrical spike which initiated the system. Since the cause of the electrical spike was known, and the Reactor Building Emergency Ventilation system was re-set to its normal stand-by mode of operation, no additional corrective actions were required.

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S PDR

## LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

U.S. NUCLEAR REGULATORY COMMISSION

APPROVED OMB NO. 3150-0104

EXPIRES 8/31/83

FACILITY NAME (1)  Nine Mile Point Unit I	DOCKET NUMBER (2)  0 5 0 0 0 2 2 0	LER NUMBER (6)			PAGE (3)	
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER		
		8 5	0 1 0	0 0	0 2	OF 0 2

TEXT (If more space is required, use additional NRC Form 385A's) (17)

TEXT

During normal operation an accidental automatic initiation of the Reactor Building Emergency Ventilation System occurred. This happened as the drywell Continuous Air Monitor (CAM) was being pulled out from control room panel J and bumped the adjacent Reactor Building Ventilation Monitor. This caused an electrical spike which initiated the Reactor Building Emergency Ventilation System on a false high radiation signal which resulted from the electrical spike.

ASSESSMENT OF POTENTIAL SAFETY CONSEQUENCES

The Reactor Building Emergency Ventilation System is normally a standby system which must only perform in the event of an accident or normal ventilation failure. Since initiation of system operation is a conservative mode, no potential safety consequences existed.

CORRECTIVE ACTION

Since the cause of the initiation of the Reactor Building Emergency Ventilation System was identified, the system was reset to its normal standby mode and no additional corrective actions were required.

## NIAGARA MOHAWK POWER CORPORATION

NIAGARA  MOHAWK300 ERIE BOULEVARD, WEST  
SYRACUSE, N. Y. 13202

September 13, 1985

United States Nuclear Regulatory Commission  
Document Control Desk  
Washington, DC 20555RE: Docket No. 50-220  
LER 85-10

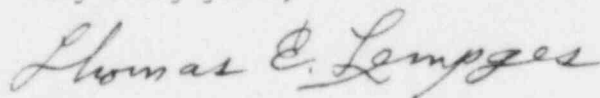
Gentlemen:

In accordance with 10 CFR 50.73, we hereby submit the following Licensee Event Report:

LER 85-10      Which is being submitted in accordance with  
10 CFR 50.73 (a)(2)(iv). "Any event or  
condition that resulted in manual or automatic  
activation of any Engineered Safety Feature."

A 10 CFR 50.72 report was made at 0031 on 8/15/85. This Licensee Event Report was completed in the format designated in NUREG-1022, dated September 1983.

Very truly yours,

Thomas E. Lempges  
Vice President  
Nuclear Generation

TEL/tg

cc: Dr. Thomas E. Murley  
Regional AdministratorIE22  
1/1