

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1) Pilgrim Nuclear Power Station - Unit 1										DOCKET NUMBER (2) 0 5 0 0 0 2 9 3				PAGE (3) 1 OF 0 2		
TITLE (4) Inadequate Recirculation Pump Start Procedure																
EVENT DATE (5)			LER NUMBER (6)				REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)						
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES				DOCKET NUMBER(S)			
1	0	3	0	8	5	8	5	0	3	0	0			0 5 0 0 0		
OPERATING MODE (9)		THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR 8: (Check one or more of the following) (11)														
N		20.402(b)				20.405(e)				50.73(a)(2)(iv)				73.71(b)		
POWER LEVEL (10)		1 0 0				20.405(a)(1)(i)				50.73(a)(2)(v)				73.71(e)		
		20.405(a)(1)(ii)				50.73(a)(2)(vi)				50.73(a)(2)(vii)(A)				OTHER (Specify in Abstract below and in Text, NRC Form 365A)		
		20.405(a)(1)(iii)				50.73(a)(2)(viii)				50.73(a)(2)(viii)(B)						
		20.405(a)(1)(iv)				50.73(a)(2)(ix)				50.73(a)(2)(ix)						
		20.405(a)(1)(v)				50.73(a)(2)(x)				50.73(a)(2)(x)						
LICENSEE CONTACT FOR THIS LER (12)																
NAME Paul J. Hamilton - Sr. Plant Engineer										TELEPHONE NUMBER						
										AREA CODE 6 1 7 7 4 6 - 7 9 0 0						
COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)																
CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPD		CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPD						
SUPPLEMENTAL REPORT EXPECTED (14)												EXPECTED SUBMISSION DATE (15)		MONTH	DAY	YEAR
YES (If yes, complete EXPECTED SUBMISSION DATE)												X NO				

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

On 10/30/85, during a review of NRC Inspection #85-26, an inadequate Technical Specification (T.S.) implementing procedure was identified. Contrary to T.S. 4.6.A.5, the procedure (2.1.9 A) did not require logging of the dome and bottom head drain temperatures when starting a recirculation (recirc.) pump.

Cause of the event was determined to be a previous management deficiency in that the review of Procedure 2.1.9.A was not adequate. The safety consequences of this event are negligible since the procedure does include a caution to compare both the dome and bottom head drain temperatures prior to starting a second recirc. pump.

Corrective action was to revise Procedure 2.1.9.A by adding a provision requiring the dome and bottom head drain temperatures to be logged when a second recirc. pump is started.

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LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

APPROVED OMB NO. 3150-0104

EXPIRES: 8/31/85

FACILITY NAME (1)	DOCKET NUMBER (2)	LER NUMBER (6)			PAGE (3)		
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER			
Pilgrim Nuclear Power Station - Unit 1	0 5 0 0 0 2 9 3	8 5	- 0 3 0	- 0 0	0 2	OF	0 2

TEXT (If more space is required, use additional NRC Form 365A's) (17)

On 10/30/85, during a review of NRC routine monthly Inspection #85-26, an inadequate Technical Specification (T.S.) implementing procedure was identified. Reactor power was 100% just prior to the discovery.

Contrary to T.S. 4.6.A.5, the recirculation (recirc.) pump start procedure (2.1.9.A) did not require the vessel dome temperature and bottom head drain temperature be logged when a recirc. pump is started. The intent of logging the bottom head drain and dome temperatures is to ensure that a recirc. pump is not started if the two temperatures vary by more than 145°F. These temperature limits are intended to reduce the probability of stratification in the vessel.

Procedure 2.1.9.A is separated into two sections. The first section is entitled "Start of Recirc. Pump from Both Pumps Stopped." This section of the procedure is adequate since it does require the two temperatures be compared and logged.

The second section of the procedure is entitled "Start of Recirc. Pump with One Pump Operating." This section of the procedure includes a caution which states, "Do not start idle pump if T between loops is greater than or equal to 50°F or if the T between the vessel dome and bottom head drain is greater than 145°F." However, contrary to T.S. 4.6.A.5, the procedure did not require the vessel dome and bottom head drain temperatures be permanently logged.

Cause of the inadequate procedure was determined to be a previous management deficiency in that a review of the T.S. implementing Procedure 2.1.9.A was not adequate (e.g., the review did not assure that the vessel dome and bottom head drain temperatures would be logged). The T.S. requirement and the inadequate procedure have been in place since plant startup.

Corrective action was to revise the second section of Procedure 2.1.9.A requiring the dome and bottom head drain temperatures be logged. Existing administrative controls for review of changes to the T.S. should preclude recurrence of issuing an inadequate T.S. implementing procedure.

Similar events describing inadequate T.S. implementing procedures are identified in LER's 83-037, 85-002, and 85-028. In response to an apparent casual link between these LER's, executive management has directed that a task to review existing T.S. implementing procedures be undertaken.

The safety consequences of this event were negligible since the caution in the procedure did require the operator to compare the dome and bottom head drain temperatures prior to starting a recirc. pump. A review of recent recorder records indicates that recirc. pump starts have been within T.S. temperature limits.

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WILLIAM D. HARRINGTON
SENIOR VICE PRESIDENT
NUCLEAR

November 27, 1985
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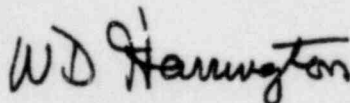
Docket Number 50-293
License DPR-35

Dear Sir:

The attached Licensee Event Report 85-030-00, "Inadequate Recirculation Pump Start Procedure," is hereby submitted in accordance with the requirements of 10CFR50.73.

If there are any questions on this subject, please do not hesitate to contact me.

Respectfully submitted,



W. D. Harrington

PJH:caw

Enclosure: LER 85-030-00

cc: Dr. Thomas E. Murley
Regional Administrator, Region I
U.S. Nuclear Regulatory Commission
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