

# FPL NUCLEAR DIVISION

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TURKEY POINT NUCLEAR STATION  
ST. LUCIE NUCLEAR STATION

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## MONTHLY INDICATOR REPORT

OCTOBER 1996

BBB/40

Issued: November 19, 1996

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## MANAGEMENT SUMMARY

### OPERATING PERFORMANCE:

Equivalent Availability Factor performance in October was as follows:

**Turkey Point Unit 3.** Unit 3 operated at full power in October; Y-T-D EAF was 94.4% which is slightly below the Y-T-D target of 95.0%.

**Turkey Point Unit 4.** Unit 4 operated at 88.2% for the month; unavailability was due to RPI Problems (0.2 hours) and repairs to the 5A Feedwater valve (11.4 hours), 4A SGFP Suction Relief Valve (2.5 hours), and U4 Low Pressure Feedwater Heater Tube (73.8 hours). For the year, EAF was 85.2% which is higher than the Y-T-D target of 79.7%.

**St. Lucie Unit 1.** EAF for Unit 1 was 98.5%. Power losses were the result of: 4B LP Heater leak (0.6 hours) and a Condenser Tube leak (10.8 hours). EAF performance Y-T-D was 66.1% which is below the 75.2% Y-T-D target.

**St. Lucie Unit 2.** Unit 2 operated at full power in October; Y-T-D EAF was 93.8% which is below the 95.0% Y-T-D target.

**Nuclear Division.** Equivalent Availability was 96.7% for the month and 84.9% Y-T-D which is below the 86.2% targeted Y-T-D. The year-end forecast for the Division is 86.5%.

A Summary of key plant operating statistics is summarized below:

No Unplanned Automatic Trips occurred in October. Year-to-date, one trip was experienced at Turkey Point Unit 3 in February.

	PTN Unit 3		PTN Unit 4		PSL Unit 1		PSL Unit 2	
	Month	Y-T-D	Month	Y-T-D	Month	Y-T-D	Month	Y-T-D
Gross Generation (MWh)	509,786	4,894,241	426,665	4,287,027	652,660	4,237,340	657,760	6,079,520
Net Generation (MWh)	486,755	4,659,874	404,697	4,075,415	619,473	3,990,520	623,933	5,752,922
Net Heat Rate (Btu/KWh)	11042.6	11043.4	11199.5	11220.4	10924.2	11077.7	10986.8	10937.7
Equivalent Availability	100.0%	94.4%	88.2%	85.2%	98.5%	66.1%	100.0%	93.8%
Capacity Factor	104.9%	96.2%	87.2%	84.2%	99.1%	65.0%	99.8%	93.7%
Auto Trips	0	1	0	0	0	0	0	0
Forced Outage Rate	0.0%	4.0%	0.0%	0.4%	0.0%	3.6%	0.0%	3.4%

## MANAGEMENT SUMMARY

### REGULATORY PERFORMANCE:

**Turkey Point.** One Notice of Violation (NOV) was reported in October.

#96-11-03 - Fire Zones 64 and 143 not covered by exemption to Appendix R. Exit Meeting date: 9/27/96.

Year-to-date, three NOV's have been reported.

**St. Lucie.** Three NOV's were reported in October for the month of September:

#96-13-01 - Failure to include some systems within the scope of the Maintenance Rule Program (Post Accident Sampling, Communications, Service Air, Radiation Monitors). Exit Meeting date: 9/20/96.

#96-13-02 - Failure to follow procedures (multiple examples). Exit Meeting date: 9/20/96.

#96-13-03 - Improper use of Most Probable Functional Failure (MPFF's) as indicators of reliability for safety significant systems. Exit Meeting date: 9/20/96.

Year-to-date, St. Lucie has reported 23 confirmed and 11 potential NOV's. (See Page F-4 for list of confirmed NOV's).

### **COST PERFORMANCE:**

**O&M Expenditures** through October 1996 were \$246.9 Million which represented a budget overrun of \$9.9 Million or (4.2%) against the revised budget. Variance is primarily due to St. Lucie Unit 1 outage scope changes partially offset by transfer of the Turkey Point Thermal Uprate Project from O&M and by project and materials purchase cancellations.

Year-to-date, O&M budget performance variances are stratified as follows:

Turkey Point Site Specific	\$7.3 Million (or 7.4%) below budget
St. Lucie Site Specific	\$21.9 Million (or 23.2%) above budget
Other Nuclear Division	\$4.7 Million (or 10.7%) below budget

**Capital Expenditures** through October 1996 were \$21.5 Million which represented a budget underrun of \$8.5 Million or (28.3%). Variance is primarily due to: plant projects cancelled/deferred/or reclassified as O&M; and, an underrun in the Steam Generator Replacement Project (SGRP).

Year-to-date, O&M budget performance variances are stratified as follows:

Turkey Point Site Specific	\$1.7 Million (or 69.9%) below budget
St. Lucie Site Specific	\$3.8 Million (or 15.6%) below budget
Other Nuclear Division	\$3.0 Million (or 90.9%) below budget

# NUCLEAR DIVISION BUSINESS PLAN INDICATOR OVERVIEW

(Through October 31, 1996)

INDICATOR	PLANT	1996 Y-E TARGET	1996 Y-T-D TARGET	1996 Y-T-D ACTUALS	COMMENTS
Collective Radiation Exposure	PTN	≤ 275.0	≤ 265.0	181.4*	Turkey Point and St. Lucie were below YTD Man-Rem targets. * Current month includes DRD count.
	PSL	≤ 485.0	≤ 468.0	381.5*	
NRC SALP Ratings	PTN	1.00		1.00	For the second consecutive time, PTN received superior ratings (1.00) in all areas for the SALP period ending 8/17/96. The next SALP period for PSL is scheduled for March 1997.
	PSL	1.50		1.50	
NRC Violations	PTN	≤ 7		3	PTN reported one NOV in 10/96 for 9/96 for a total of 3 Y-T-D. PSL reported three NOV's in 10/96 for the month of 9/96; Y-T-D, PSL has received 23 confirmed and 11 potential NOV's.
	PSL	≤ 7		23	
Unplanned Automatic Trips	PTN3	≤ 3		1	No unplanned automatic trips were reported in October.
	PTN4	for		0	
	PSL1	Division		0	
	PSL2			0	
Budget Performance (\$ Millions)	Capital	39.2	30.1	21.5	Capital Y-T-D actuals were below the Y-T-D target. O&M actuals exceeded Y-T-D target primarily as a result of PSL's outage scope changes.
	O&M	288.9	237.0	246.9	
	Div. Total	328.1	267.1	268.4	
Equivalent Availability Factor (%)	PTN3	95.0	95.0	94.4	PTN Unit 4 performance Y-T-D through October was higher than the Y-T-D target. For the Division, EAF Y-T-D was 84.9% which is below the 86.2% Y-T-D target.
	PTN4	82.0	79.7	85.2	
	PSL1	78.5	75.2	66.1	
	PSL2	95.0	95.0	93.8	
	Division	88.0	86.2	84.9	
M&S Inventory Levels (\$ Millions)	PTN	≤ 38		34.5	PSL exceeded the Year-End target of \$38.0 by \$0.4M in October.
	PSL	≤ 38		38.4	
Cost Production Cost (O&M and Fuel)	PTN	1.75	1.78	1.81	PTN and PSL Production Costs (¢/KWh) were higher than Y-T-D targets in October.
	PSL	1.55	1.59	1.62	
	Division Total	4.56	4.65	4.68	In October, Division Total Cost (¢/KWh) was slightly higher than the Y-T-D target.
Nuclear Division Staffing Levels	FPL	2045.0		1825.0	Total Nuclear Division Staffing Level was below the Year-End target.
	LTC	463.0		331.0	
	Total	2508.0		2156.0	
Refueling Outage Duration (Days)	PTN3			0	Dates for future refueling outages are being revised and should be available in next month's report.
	PTN4	45		35	
	PSL1	63		88	
	PSL2	NA		0	
Lost Time Injuries per 200,000 Hrs. Worked (12 Mo. Running)	PTN	0.30		0.28	No Lost Time Injuries were reported in October.
	PSL	0.30		0.26	

# NRC INDICATOR PERFORMANCE OVERVIEW

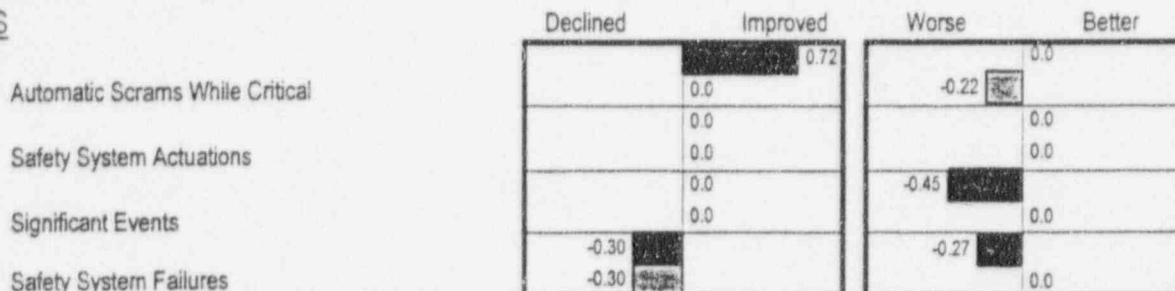
## for Turkey Point (Data through Quarter Ending September 30, 1995)

LEGEND	
PTN 3	
PTN 4	

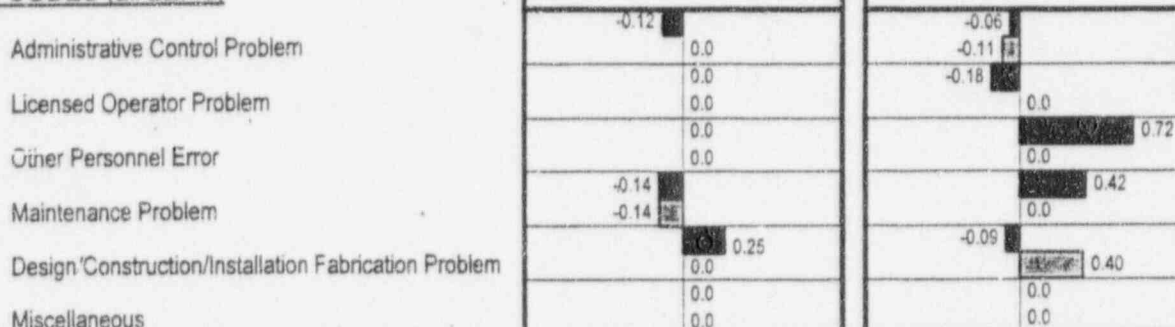
Plant Self-Trend  
Short Term

Deviations From Peer  
Group Median Long Term

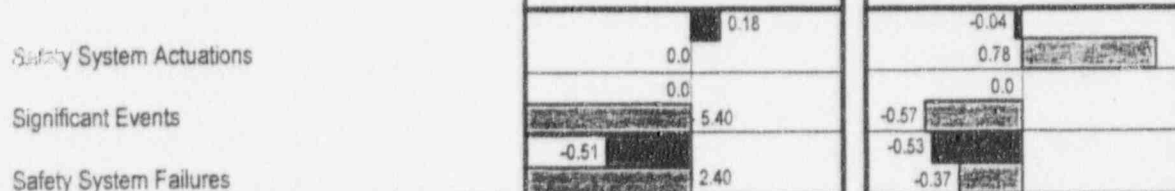
### OPERATIONS



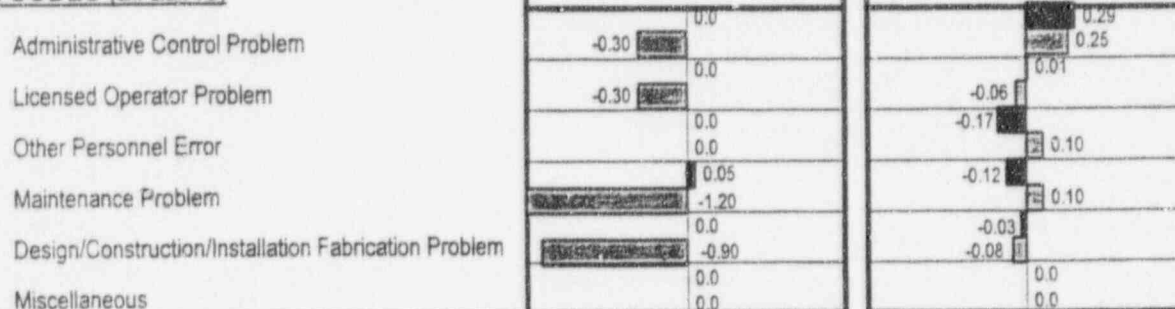
### CAUSE CODES (all LER's)



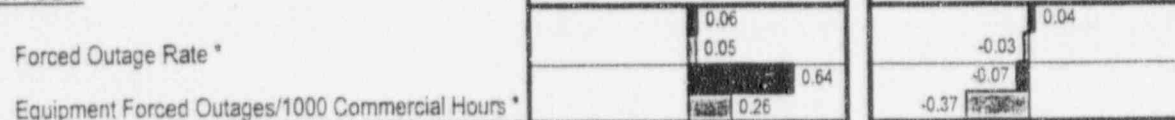
### SHUTDOWN



### CAUSE CODES (all LER's)



### FORCED OUTAGES



\* Not Calculated for Operational Cycle

Performance Index

Performance Index

#### NOTES:

Plant Self-Trend Short Term: Based on the slope of a linear regression line plotted over each plant's data. Time intervals used in the trends are 4 Quarters for "Operations" and "Forced Outages" indicators and 6 Quarters for "Shutdown" indicators.

Deviations from Peer Group Median Long Term: Comparisons are made of each plant to the performance of its peers over a 12 Quarter time interval.

Peer Groups: PTN 3&4 - Older Westinghouse 3-Loop.



# NRC INDICATOR PERFORMANCE OVERVIEW

## for St. Lucie (Data through Quarter Ending September 30, 1995)

LEGEND	
PSL 1	
PSL 2	

Plant Self-Trend  
Short Term

Deviations From Peer  
Group Median Long Term

### OPERATIONS

Automatic Scrams While Critical

Safety System Actuations

Significant Events

Safety System Failures

### CAUSE CODES (all LER's)

Administrative Control Problem

Licensed Operator Problem

Other Personnel Error

Maintenance Problem

Design/Construction/Installation Fabrication Problem

Miscellaneous

### SHUTDOWN

Safety System Actuations

Significant Events

Safety System Failures

### CAUSE CODES (all LER's)

Administrative Control Problem

Licensed Operator Problem

Other Personnel Error

Maintenance Problem

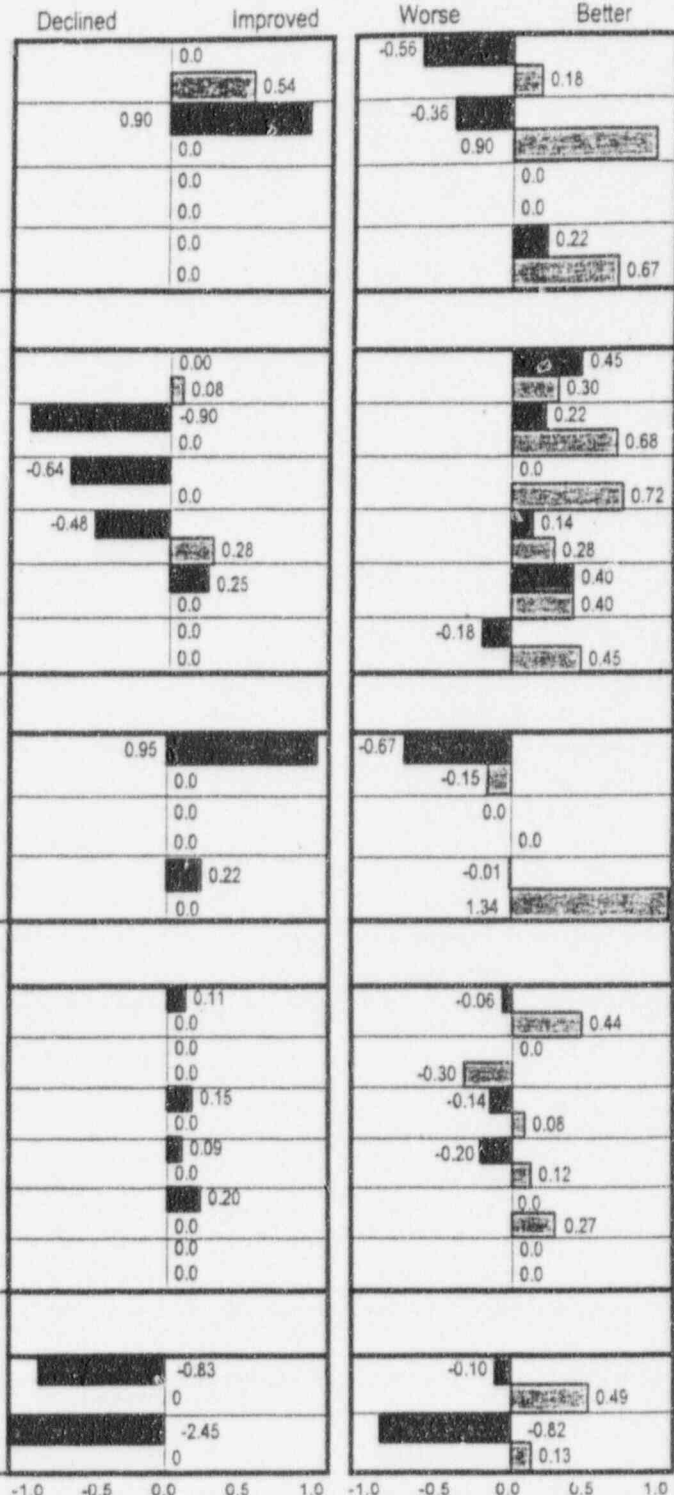
Design/Construction/Installation Fabrication Problem

Miscellaneous

### FORCED OUTAGES

Forced Outage Rate \*

Equipment Forced Outages/1000 Commercial Hours \*



\* Not Calculated for Operational Cycle

#### NOTES:

Plant Self-Trend Short Term: Based on the slope of a linear regression line plotted over each plant's data. Time intervals used in the trends are 4 Quarters for "Operations" and "Forced Outages" indicators and 6 Quarters for "Shutdown" indicators.

Deviations from Peer Group Median Long Term: Comparisons are made of each plant to the performance of its peers over a 12 Quarter time interval.

Peer Groups: PSL 1&2 - Combustion Engineering with core protection calculator plants.

# WANO OVERALL INDICATOR PERFORMANCE OVERVIEW

(October 31, 1996)

PERFORMANCE INDICATORS		Unit or Station Values				Industry Median Values *	
		PTN 3	PTN 4	PSL 1	PSL 2	3-Yr. Distribution (7/93 - 6/96)	2000 Goals
Unit Capability Factor (Unit %, 3-Yr. Distribution Ending October '96)		89.2%	89.4%	77.0%	81.5%	81.4%	87.0%
Unplanned Capability Loss Factor (Unit %, 3-Yr. Distribution Ending October '96)		3.4%	3.4%	15.5%	8.0%	5.7%	3.0%
Unplanned Automatic Scrams Per 7000 Hours Critical (Per Unit, 3-Yr. Distribution Ending October '96)		0.6	0.6	1.6	0.6	0.9	1.0
Safety System Performance:	High Pressure Safety Injection System (per Unit, 3-Yr. Distribution Ending September '96)	0.009	0.009	0.026	0.009	0.003 - 0.008	not available
	Auxiliary Feedwater System (per Unit, 3-Yr. Distribution Ending September '96)	0.013	0.010	0.009	0.013	0.004 - 0.009	not available
	Emergency AC Power system (per Unit, 3-Yr. Distribution ending September '96)	0.006	0.003	0.019	0.013	0.009 - 0.014	not available
Thermal Performance (Ratio of Design to Actual Gross Heat Rate, (1-Yr. Distribution Ending October '96)		100.0%	99.0%	99.7%	99.7%	99.6% ***	99.5%
Collective Radiation Exposure (Man-Rem per unit per year, 3-Yr. running avg. ending October '96)		147.4	147.4	217.6	217.6	153.0	110.0
Volume of Low-level Solid Radioactive Waste (Cubic meters per unit per year, 3-Yr. avg. ending October '96)		31.5	31.5	52.0	52.0	46.0	45.0
Chemistry Index (12-mo. weighted average through October '96)		1.03	1.05	1.17	1.18	1.16 **	1.10
Industrial Safety Lost-Time Accident Rate (Station rate per 200,000 man-hours worked ending October '96)		0.28		0.26		0.44 **	0.40
Fuel Reliability (Unit microcuries/g, month ending October '96)		1.00E-06	2.25E-05	1.00E-06	8.90E-05	Fuel Defect Reference 5.0E-04 ***	not available

NOTE: Shaded area denotes FPL performance is unfavorable to actual industry median.

Source of Industry Data:

\* 1996 Mid-Year Report for Performance Indicators for the U. S. Nuclear Utility Industry (7/93-6/96 Distribution).

\*\* 1996 Mid-Year Report for Performance Indicators for the U. S. Nuclear Utility Industry (7/95-6/96 Distribution).

\*\*\* 1996 Mid-Year Report for Performance Indicators for the U. S. Nuclear Utility Industry (4/96-6/96 Distribution).

# WANO OVERALL INDICATOR PERFORMANCE OVERVIEW

Discussion of FPL Performance Unfavorable to Industry Median  
(October 31, 1996)

## UNIT CAPABILITY FACTOR (3-Years Ending 10/31/96)

St. Lucie Unit 1. The 3-Year running Capability Factor through 10/31/96 was 77.0%. Capability loss is attributed to the following: Hot Leg Valve MV-3480 leak repairs from 3/29/94 to 4/2/94 (0.4%); Main Transformer trip from 6/6/94 to 6/11/94 (0.4%); Quench Tank leak repairs from 2/27/95 to 3/8/95 (0.9%); 1A2 Reactor Coolant Pump seal repairs from 8/2/95 to 8/9/95 (0.7%); inoperable Power Operated Relief Valves from 8/9/95 to 8/17/95 (0.7%); inadvertent Containment Spray actuation and clean-up from 8/17/95 to 9/3/95 (1.3%); 1B2 Diesel Generator failure from 9/1/95 to 9/6/95 (0.5%); 1A Diesel Generator Radiator leakage from 9/6/95 to 9/10/95 (0.4%); Code Safety Valve repairs and modifications from 9/11/95 to 10/13/95 (2.4%); a refueling outage from 4/29/96 to 7/25/96 (8.0%); and, 1B Main Transformer repairs (0.8%). Other miscellaneous unplanned outages and derates accounted for the remaining 6.5% Unit Capability Factor loss.

## UNPLANNED CAPABILITY LOSS FACTOR (3-Years Ending 10/31/96)

St. Lucie Unit 1. The Unplanned Capability Loss Factor for the three years ending 10/31/96 was 15.5% compared to an industry median of 5.7%. Unplanned outages and power reductions contributing to this performance included: Hot Leg Valve MV-3480 leak repairs from 3/29/94 to 4/2/94 (0.4%); Main Transformer trip from 6/6/94 to 6/11/94 (0.4%); Quench Tank leak repairs from 2/27/95 to 3/8/95 (0.9%); 1A2 Reactor Coolant Pump seal repairs from 8/2/95 to 8/9/95 (0.7%); inoperable Power Operated Relief Valves from 8/9/95 to 8/17/95 (0.7%); inadvertent Containment Spray actuation and clean-up from 8/17/95 to 9/1/95 (1.3%); 1B2 Diesel Generator Failure from 9/1/95 to 9/6/95 (0.5%); 1A Diesel Generator Radiator leakage from 9/6/95 to 9/10/95 (0.4%); Code Safety Valve repairs and modifications from 9/11/95 to 10/31/95 (2.4%); dropped CEA's from 2/22/96 to 2/25/96 (0.2%); a refueling outage extension from 6/22/96 to 7/25/96 (3.1%); and, 1B Main Transformer repairs (0.8%). Other miscellaneous unplanned outages and derates accounted for the remaining 3.7% Capability Loss.

St. Lucie Unit 2. The Unplanned Capability Loss Factor for the three years ending 10/31/96 for 8.0%. Major unplanned occurrences contributing to this performance included: shutdown for auto reactor trip investigation on 4/23/94 (0.3%); refueling outage extension from 12/1/95 to 1/5/96 (3.2%); and, Hydrogen System problems from 1/5/96 to 1/7/96 (0.2%); and, shutdown from Hydrogen Cooling System Valve and 2C Auxiliary Feedwater Pump repairs from 6/6/96 to 6/13/96 (0.7%). Other unplanned outages and power reductions accounted for the remaining 3.6% in Capability Loss.

## UNPLANNED AUTOMATIC SCRAMS PER 7000 HOURS CRITICAL (3-Years Ending 10/31/96)

St. Lucie Unit 1. Increased rate for Unit 1 was the result of five auto trips occurring on 3/28/94, 4/3/94, 6/6/94, 10/26/94, and 7/8/95.



#### HIGH PRESSURE SAFETY INJECTION SYSTEM (3-Years Ending 9/30/96)

Turkey Point Units 3 & 4. Average performance was affected due to the on-line replacement of the HPSI pump motors following discovery of cracked rotor bars.

St. Lucie Unit 1. Average performance for the last three years was affected by on line Motor Operated Valve testing in the 3rd Quarter of 1994 and a breaker failure on 2B HPSI pump in the 1st Quarter of 1995.

St. Lucie Unit 2. In the 1st Quarter of 1995, average performance was affected as a result of Component Cooling Water (CCW) Heat Exchanger cleaning which placed the respective HPSI pump OOS due to lack of dedicated seal cooling.

#### AUXILIARY FEEDWATER SYSTEM (3-Years Ending 9/30/96)

Turkey Point Units 3 & 4. Average performance for both units was affected by the B AFW Turbine failure in the 4th Quarter of 1994 due to malfunction of the mechanical overspeed trip device; in the 3rd Quarter of 1995, performance was affected by: Part 21 repairs on the Trip and Throttle Valves, and Unit 3 outage work.

NOTE: Turkey Point's Auxiliary Feedwater system is configuratively unique because it is a shared system between two units. For example, during a refueling outage, the non-refueling unit will always be impacted with unavailable hours. Due to its uniqueness, it is unlikely Turkey Point will be within the Industry Median value (0.003 - 0.008).

St. Lucie Unit 2. Average performance for three-years was affected by failure of the AFW PP 2C Steam Admission Valve MV-08-13 to open, a mechanical trip linkage for AFW PP 2C when the Electrical Overspeed Solenoid was energized, and a discrepancy between field wiring and plant wiring drawing for the AFW PP 2B.

#### EMERGENCY DIESEL GENERATOR SYSTEM (3-Years Ending 9/30/96)

St. Lucie Unit 1. Unit 1's average performance for three years was the result of a high water jacket temperature trip of 1A EDG and failure of the governor on 1B2 during monthly surveillance run which closed off fuel to the 12 cylinder engine in the 2nd Quarter of 1995, 1B diesel 12 cylinder engine valve failure in the 3rd Quarter of 1995, and 1B diesel due to replacement of the cooling water valves in the 4th Quarter 1995.

#### COLLECTIVE RADIATION EXPOSURE - MAN-REM (3-Years Ending 10/31/96)

St. Lucie. Collective Radiation Exposure three-year running average level for St. Lucie was 217.6 Man-Rem per unit which was greater than the industry median of 153 Man-Rem. Site performance was influenced by unplanned and scheduled outages.

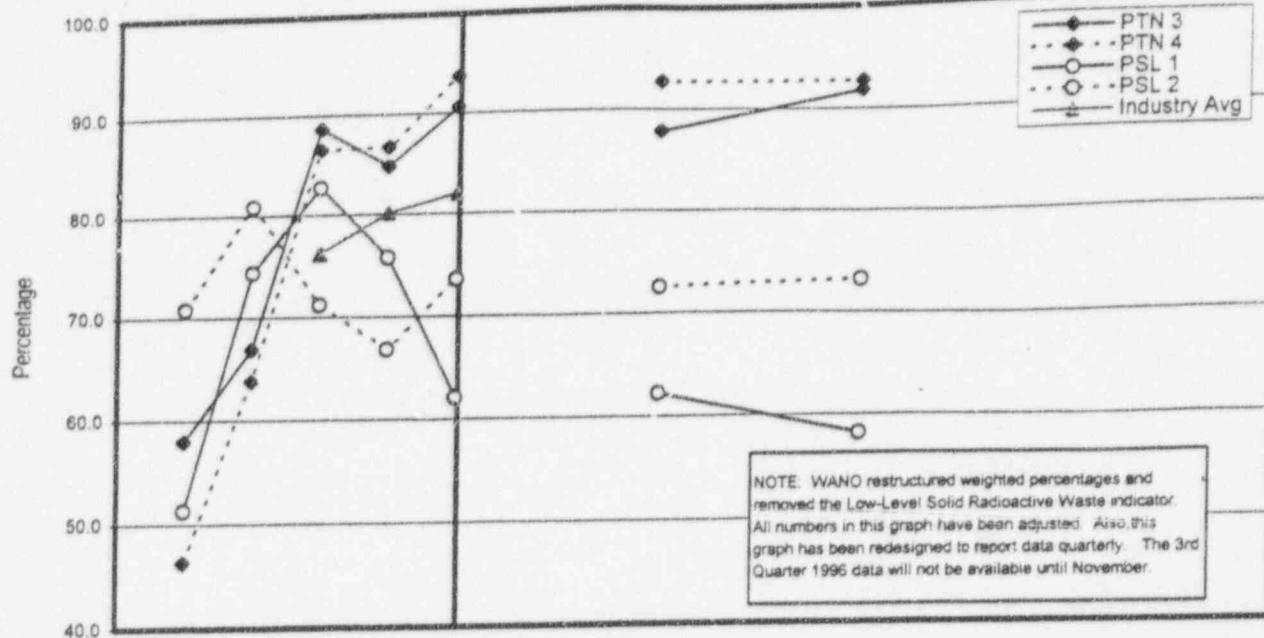
#### VOLUME OF SOLID RADIOACTIVE WASTE (Annual Avg. for 3-Years Ending 10/31/96)

St. Lucie. Volume of Solid Radioactive Waste for the 3-year annual average was higher than industry median due to 1994 waste volume; the 1995 value of 41.9 cubic meters was below the median value.

#### CHEMISTRY PERFORMANCE (12-Mo. Weighted Average Ending 10/31/96)

St. Lucie. Performance at St. Lucie is affected by the elevated condensate dissolved oxygen.

# WANO WEIGHTED OVERALL PERFORMANCE



Unit	1991	1992	1993	1994	1995	3/96	6/96	9/96	12/96
PTN 3	58.0	66.9	88.6	84.8	90.6	87.8	91.8		
PTN 4	46.3	63.8	86.5	86.7	93.7	92.7	92.7		
PSL 1	51.3	74.4	82.7	75.8	62.1	62.2	58.2		
PSL 2	70.8	80.9	71.2	66.8	73.6	72.5	73.2		
Industry Avg	n/a	n/a	76.2	80.1	82.0				

## DEFINITION

The WANO Overall Performance Index is a composite indicator utilized to trend nuclear station performance. The index is a weighted combination of the following 11 individual performance indicators over the time period noted:

1. Unit Capability Factor (16%) (2 yrs)\*
2. Unplanned Capability Loss Ftr (12%) (2 yrs)\*
3. High Pressure Safety Injection (10%) (2 yrs)\*
4. Auxiliary Feedwater System (10%) (2 yrs)\*
5. Emergency AC Power (10%) (2 yrs)\*
6. Unplanned Auto-Scrams (8%) (2 yrs)\*
7. Collective Radiation Exposure (8%) (2 yrs)\*
8. PWR Fuel Reliability (8%) (most recent qtr)
9. Thermal Performance (6%) (1 yr)
10. Chemistry Indicator (7%) (1 yr)
11. Industrial Safety Accidents (5%) (1 yr)

## STATISTICAL SUMMARY

## INDUSTRY PERFORMANCE

### WANO:

Through 4th Quarter 1995 - Industry Median: 84.0%

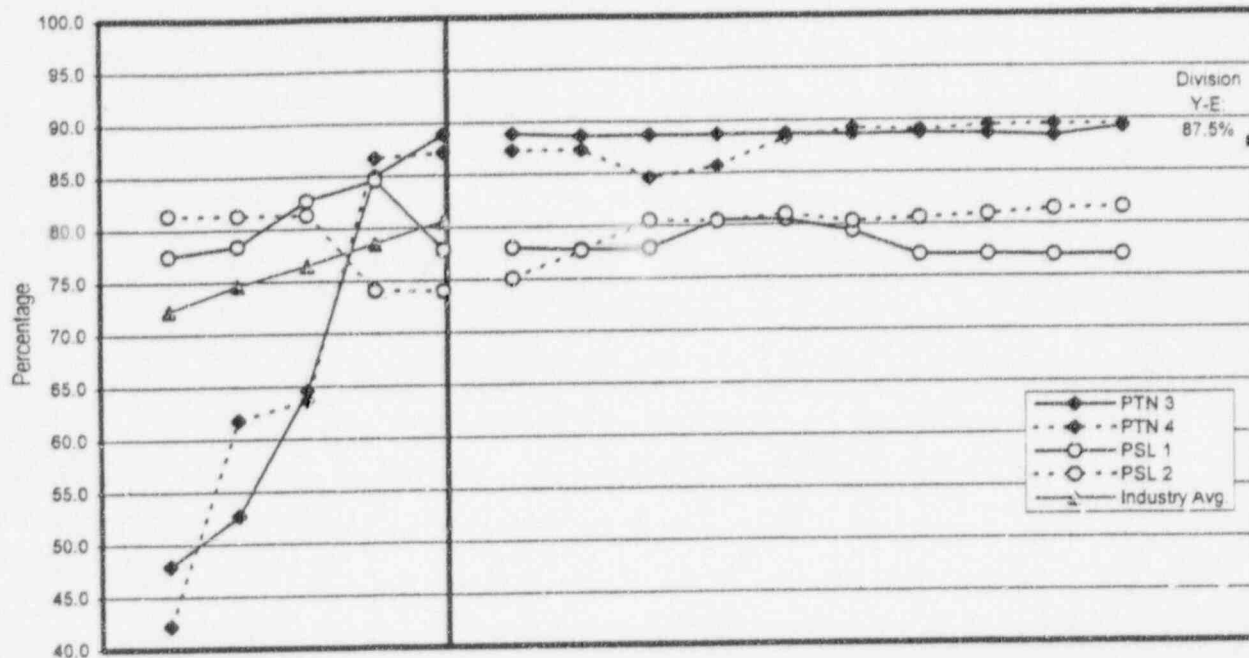
## PERFORMANCE SUMMARY

St. Lucie Unit 1's decline from 1st Quarter to 2nd Quarter is primarily the result of a decrease in Unit Capability Factor, increased Safety System performance rates, and increased Collective Radiation Exposure.\*

\* See above for length of period used in weighted calculations.

# UNIT CAPABILITY FACTOR

(3-Year Running Average)



Unit	1991	1992	1993	1994	1995	1/96	2/96	3/96	4/96	5/96	6/96	7/96	8/96	9/96	10/96	11/96	12/96
PTN 3	47.9	52.7	64.6	85.0	88.9	88.9	88.6	88.7	88.7	88.7	88.6	88.7	88.6	88.4	89.2		
PTN 4	42.2	61.8	63.7	86.7	87.2	87.3	87.3	84.6	85.7	88.3	89.2	89.0	89.4	89.5	89.4		
PSL 1	77.5	78.4	82.7	84.6	77.9	78.0	77.8	77.9	80.5	80.5	79.4	77.1	77.1	77.0	77.0		
PSL 2	81.4	81.3	81.3	74.1	74.0	75.1	77.7	80.5	80.4	81.0	80.3	80.6	80.9	81.4	81.5		
Industry Avg.	72.3	74.6	76.5	78.6	80.6												

## DEFINITION

Unit Capability Factor is the ratio of the available energy generation over a given time period to the reference energy generation over the same time period, expressed as a percentage with both energy generation terms determined relative to reference ambient conditions. Available energy generation is the energy that could have been produced under reference ambient conditions considering only limitations within control of plant management, i.e., plant equipment and personnel performance, and work control. Reference energy generation is the energy that could be produced if the unit were operated continuously at full power under reference ambient conditions throughout the period. Reference ambient conditions are environmental conditions representative of the annual mean (or typical) ambient conditions for the unit.

## STATISTICAL SUMMARY

	Oct	Y-T-D	3-yr Running	Y-E Target
PTN 3	100.0%	94.4%	89.2%	≥ 95.0%
PTN 4	88.2%	85.2%	89.4%	≥ 82.0%
PSL 1	98.5%	66.1%	77.0%	≥ 78.5%
PSL 2	100.0%	93.8%	81.5%	≥ 95.0%

## INDUSTRY PERFORMANCE

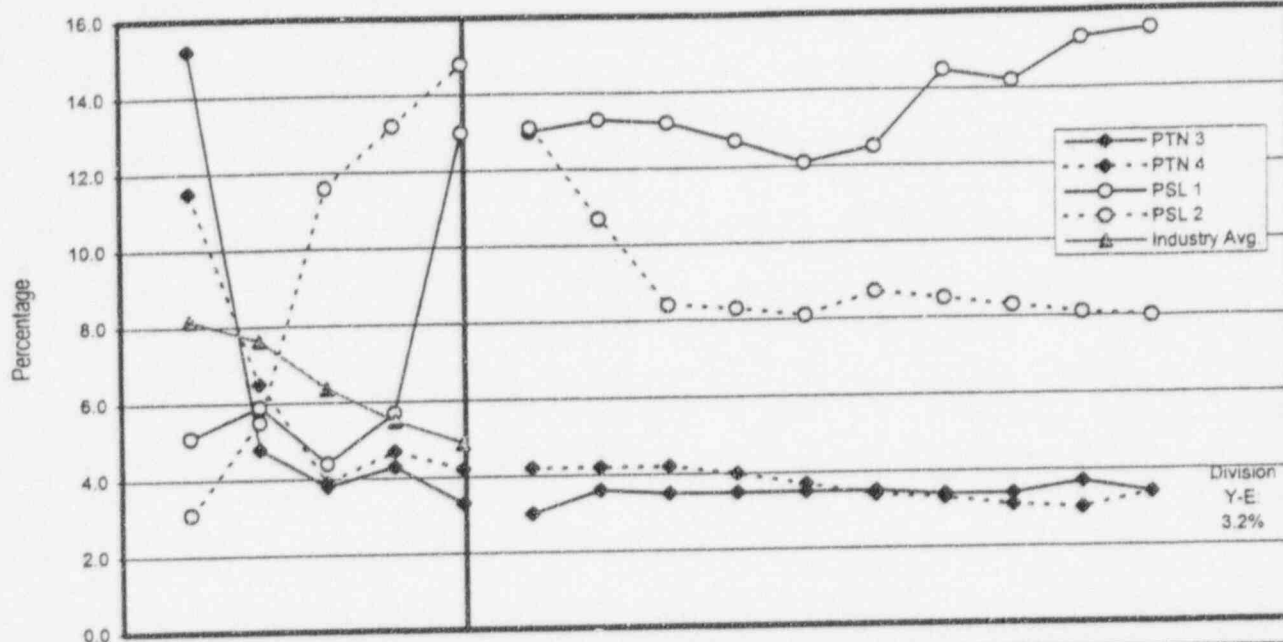
WANO	
7/93 - 6/96 Median	81.4%
7/95 - 6/96 Median	84.2%
7/95 - 6/96 Average	79.8%
Year 2000 Goal	87.0%

## PERFORMANCE SUMMARY

See comments on Page ix.

# UNPLANNED CAPABILITY LOSS FACTOR

(3-year Running Average)



Unit	1991	1992	1993	1994	1995	1/96	2/96	3/96	4/96	5/96	6/96	7/96	8/96	9/96	10/96	11/96	12/96
PTN 3	15.2	4.8	3.8	4.3	3.3	3.0	3.6	3.5	3.5	3.5	3.5	3.4	3.4	3.7	3.4		
PTN 4	11.5	6.5	3.9	4.7	4.2	4.2	4.2	4.2	4.0	3.7	3.4	3.3	3.1	3.0	3.4		
PSL 1	5.1	5.9	4.4	5.7	13.0	13.0	13.3	13.2	12.7	12.1	12.5	14.5	14.2	15.3	15.5		
PSL 2	3.1	5.5	11.6	13.2	14.8	13.1	10.7	8.4	8.3	8.1	8.7	8.5	8.3	8.1	8.0		
Industry Avg.	8.2	7.6	6.4	5.5	4.9												

## DEFINITION

Unplanned Capability Loss Factor is defined as the ratio of the unplanned energy losses during a given period of time to the reference energy generation, expressed as a percentage. Unplanned energy loss is energy that was not produced during the period because of unplanned shutdowns, outage extensions, or unplanned load reductions due to causes under plant management control. Causes of energy losses are considered to be unplanned if they are not scheduled at least four weeks in advance. Reference energy generation is the energy that could be produced if the unit were operated continuously at full power under reference ambient conditions throughout the period. Reference ambient conditions are environmental conditions representative of the annual mean (or typical) ambient conditions for the unit.

## STATISTICAL SUMMARY

	3-yr Running
PTN 3	3.4%
PTN 4	3.4%
PSL 1	15.5%
PSL 2	8.0%

1996 Target (all units):  $\leq 3.2\%$

## INDUSTRY PERFORMANCE

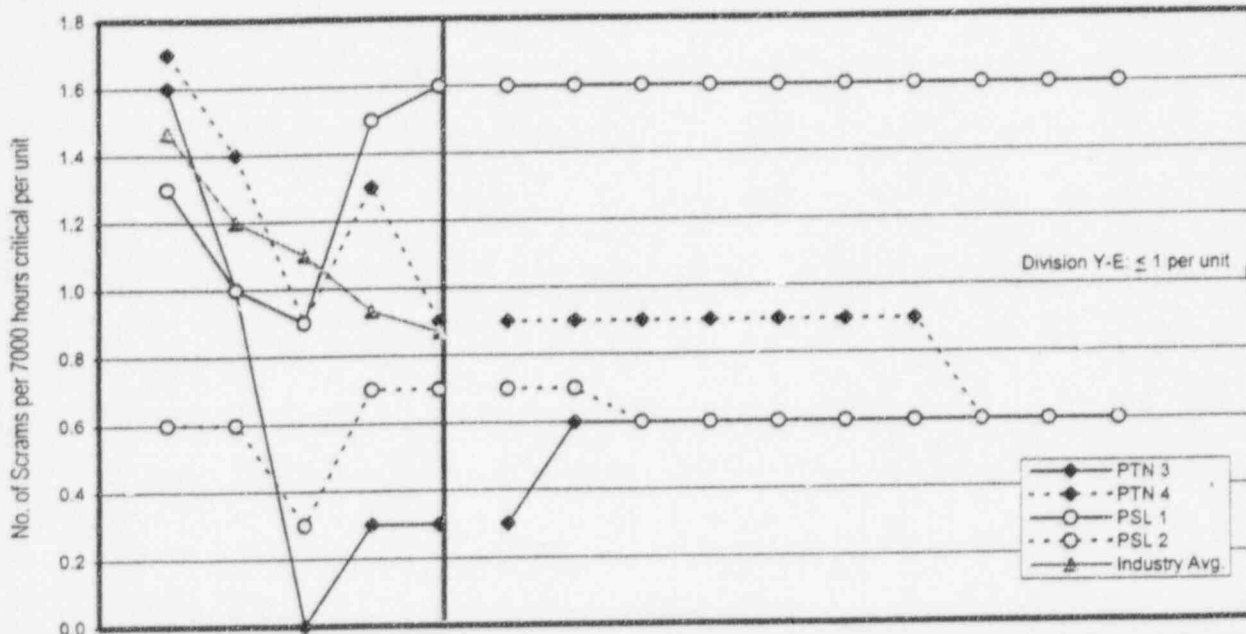
WANO	
7/93 - 6/96 Median	5.7%
7/95 - 6/96 Median	5.0%
Year 2000 Goal	3.0%

## PERFORMANCE SUMMARY

See comments on Page ix.

# UNPLANNED AUTO TRIPS PER 7000 HOURS CRITICAL

(3-year Running Average)



Unit	1991	1992	1993	1994	1995	1/96	2/96	3/96	4/96	5/96	6/96	7/96	8/96	9/96	10/96	11/96	12/96
PTN 3	1.6	1.0	0.0	0.3	0.3	0.3	0.6	0.6	0.6	0.6	0.6	0.6	0.6	0.6	0.6	0.6	
PTN 4	1.7	1.4	0.9	1.3	0.9	0.9	0.9	0.9	0.9	0.9	0.9	0.9	0.6	0.6	0.6		
PSL 1	1.3	1.0	0.9	1.5	1.6	1.6	1.6	1.6	1.6	1.6	1.6	1.6	1.6	1.6	1.6		
PSL 2	0.6	0.6	0.3	0.7	0.7	0.7	0.7	0.6	0.6	0.6	0.6	0.6	0.6	0.6	0.6		
Industry Avg.	1.5	1.2	1.1	0.9	0.9												

## DEFINITION

Unplanned Automatic Scrams per 7000 Hours Critical is defined as the number of unplanned automatic scrams that occur per 7000 hours of critical operation. Unplanned means that the scram was not an anticipated part of a planned test. Scram means the automatic shutdown of the reactor by a rapid insertion of negative reactivity (by control rods, liquid injection shutdown system, etc.) caused by actuation of the reactor protection system. The scram signal may have resulted from exceeding a setpoint or may have been spurious. Automatic means that the initial signal that caused actuation of the reactor protection system logic was provided from one of the sensors monitoring plant parameters and conditions rather than from the manual scram switches or, in certain cases described in INPO 94-009, from manual turbine trip switches provided in the main control room. Critical means that during the steady-state condition prior to the scram, the effective reactor multiplication factor was essentially equal to one.

## STATISTICAL SUMMARY

	Through October	
	12-mo	36-mo
Division Totals:		
1996	0.2	0.8
1995	0.9	0.9

Division Target:  $\leq 3.0$  auto trips

## INDUSTRY PERFORMANCE

WANO	
7/93 - 6/96 Median	0.9%
7/95 - 6/96 Median	0.8%
Year 2000 Goal	1.0%

## PERFORMANCE SUMMARY

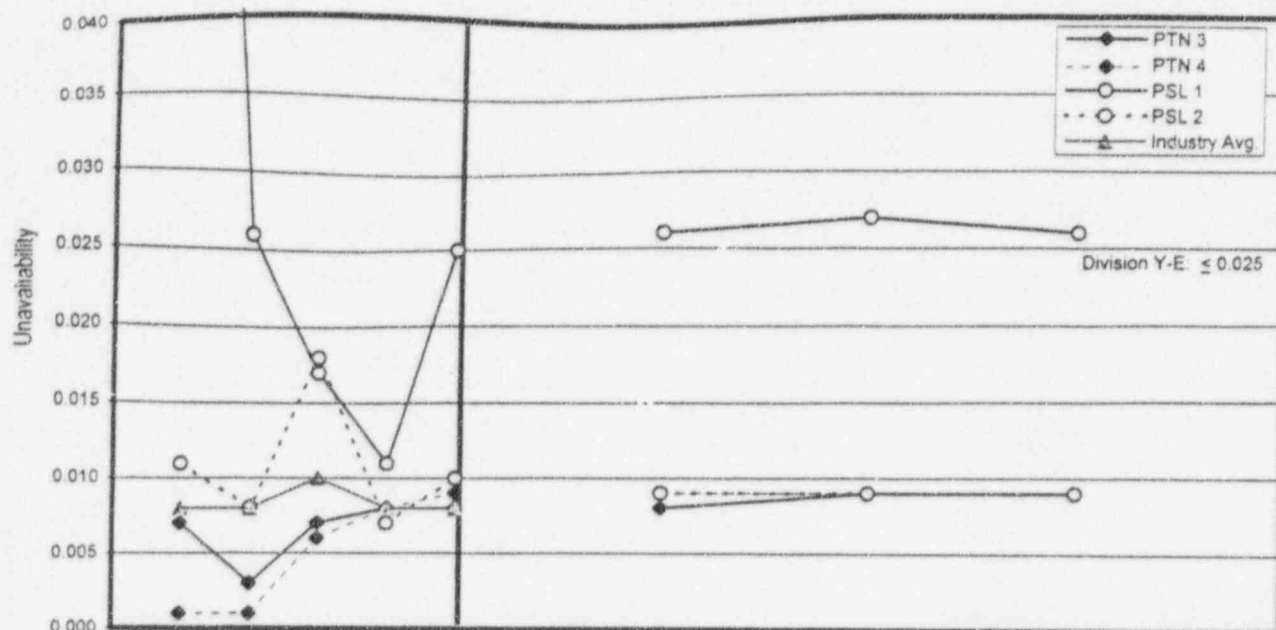
See Page ix for comments.

Data Providers: (PTN) Jim Knorr 246-6757 and (PSL) Kelly Korth 467-7054



# HIGH PRESSURE SAFETY INJECTION SYSTEM PERFORMANCE

(3-yr Running Average)



Unit	1991	1992	1993	1994	1995	3/96	6/96	9/96	12/96
PTN 3	0.007	0.003	0.007	0.006	0.008	0.008	0.009	0.009	
PTN 4	0.001	0.001	0.006	0.008	0.009	0.009	0.009	0.009	
PSL 1	0.091	0.026	0.017	0.011	0.025	0.026	0.027	0.026	
PSL 2	0.011	0.008	0.018	0.007	0.010	0.009	0.009	0.009	
Industry Avg.	0.008	0.008	0.010	0.008	0.008				

## DEFINITION

This Safety System Performance indicator monitors the readiness of the Safety Injection (SI) System at Turkey Point and the High Pressure Safety Injection (HPSI) System at St. Lucie to respond to off-normal events or accidents. The indicator is determined from the unavailabilities, due to all causes, of the components in the system during a time period, divided by the number of trains in the system. The definition is further explained: component unavailability is the ratio of the hours the component was unavailable (unavailable hours) to the hours the system was required to be available for service. Data is reported on a quarterly basis.

$$\text{Unavailability} = \frac{(\text{Known Unavailable Hours}) + (\text{Estimated Unavailable Hours})}{(\text{Hours System Required}) \times (\text{Number of Trains})}$$

STATISTICAL SUMMARY			INDUSTRY PERFORMANCE	
	<u>3rd Qtr 1996</u>	<u>3-yr Avg Ending 9/96</u>		
PTN 3	0.000	0.009	<u>WANO</u>	<u>Unavailability</u>
PTN 4	0.001	0.009		
PSL 1	0.000	0.026	7/93 - 6/96 Median (PWR)	0.003 - 0.008
PSL 2	0.004	0.009	2000 Goal	0.020
Targets:	<u>PTN</u>	<u>PSL</u>		
1995 Y-E Target	≤ 0.016	≤ 0.023		
1996 Y-E Target	≤ 0.025	≤ 0.025		

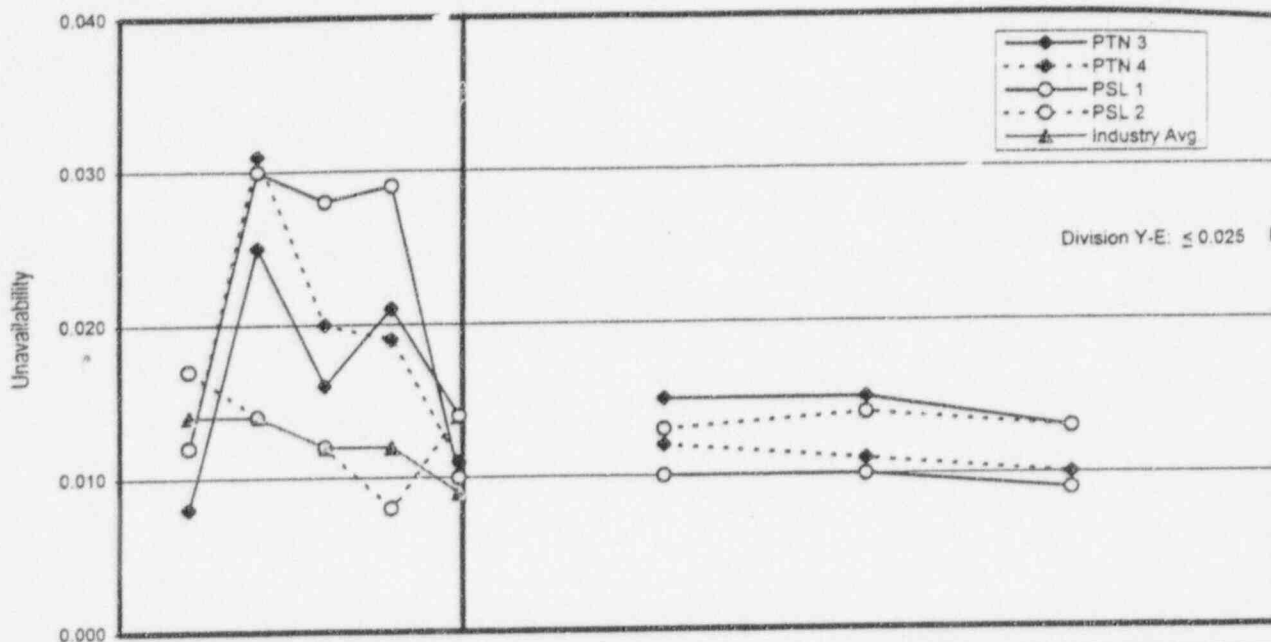
## PERFORMANCE SUMMARY

See Page x for comments.

Data Providers: (PTN) Brian Dunn 246-6173/Carlos Melchor 246-6964  
(PSL) Chuck Wood 467-7034/Juni Krumins 467-7136

# SAFETY SYSTEM PERFORMANCE - AUXILIARY FEEDWATER SYSTEM

(3-year Running Average)



Unit	1991	1992	1993	1994	1995	3/96	6/96	9/96	12/96
PTN 3	0.008	0.025	0.016	0.021	0.014	0.015	0.015	0.013	
PTN 4	0.012	0.031	0.020	0.019	0.011	0.012	0.011	0.010	
PSL 1	0.012	0.030	0.028	0.029	0.010	0.010	0.010	0.009	
PSL 2	0.017	0.014	0.012	0.008	0.014	0.013	0.014	0.013	

## DEFINITION

This Safety System Performance indicator monitors the readiness of the Auxiliary Feedwater (AFW) System to respond to off-normal events or accidents. The indicator is determined from the unavailabilities, due to all causes, of the components in the system during a time period, divided by the number of trains in the system. This definition is further explained: component unavailability is the ratio of hours the component was unavailable (unavailable hours) to the hours the system was required to be available for service.

$$\text{AFW Unavailability} = \frac{(\text{Known Unavailable Hours}) + (\text{Estimated Unavailable Hours})}{(\text{Hours System Required}) \times (\text{Number of Trains})}$$

## STATISTICAL SUMMARY

	3rd Qtr 1996	3-yr Avg Ending 9/96
PTN 3	0.004	0.013
PTN 4	0.004	0.010
PSL 1	0.004	0.009
PSL 2	0.003	0.013

Targets:	PTN	PSL
1995 Y-E Target	≤ 0.020	≤ 0.021
1996 Y-E Target	≤ 0.025	≤ 0.025

## INDUSTRY PERFORMANCE

WANO	Unavailability
7/93 - 6/96 Median (PWR)	0.004 - 0.009
2000 Goal	0.020

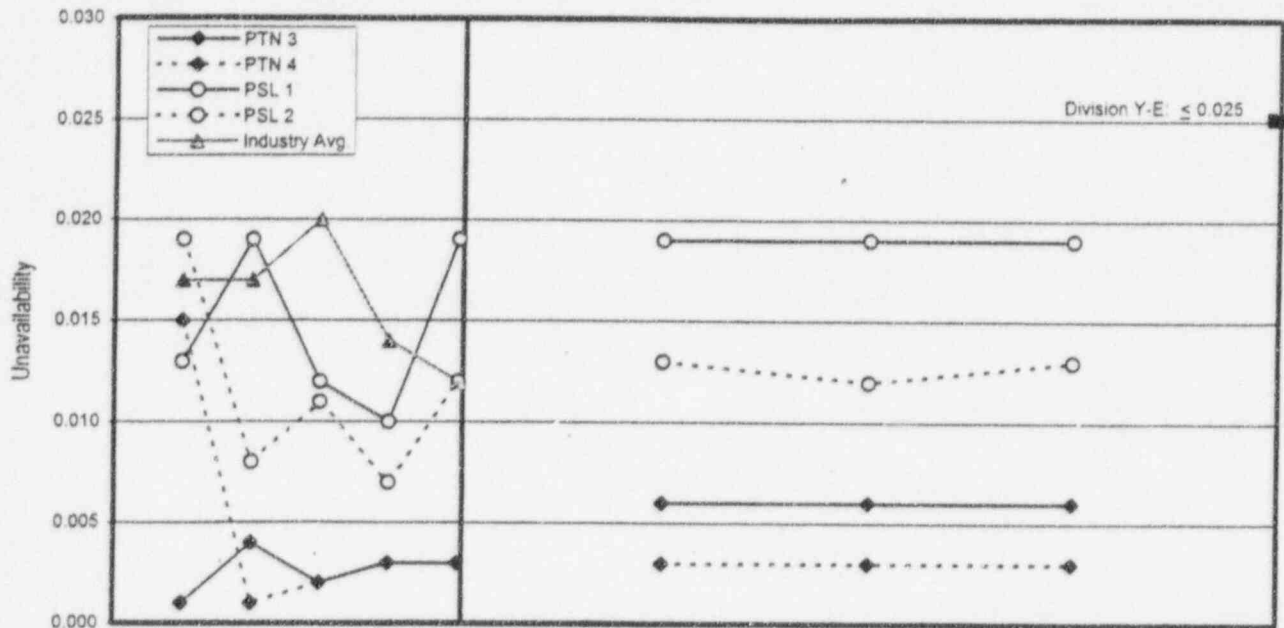
## PERFORMANCE SUMMARY

See Page x for comments.

Data Providers: (PTN) Brian Dunn 246-6173/Woody Raasch 246-6527  
(PSL) Chuck Wood 467-7034/Mark Wolaver 467-7083

# SAFETY SYSTEM PERFORMANCE - EMERGENCY AC POWER SYSTEM

(3-year Running Average)



Unit	1991	1992	1993	1994	1995	3/96	6/96	9/96	12/96
PTN 3	0.001	0.004	0.002	0.003	0.003	0.006	0.006	0.006	
PTN 4	0.015	0.001	0.002	0.003	0.003	0.003	0.003	0.003	
PSL 1	0.013	0.019	0.012	0.010	0.019	0.019	0.019	0.019	
PSL 2	0.019	0.008	0.011	0.007	0.012	0.013	0.012	0.013	
Industry Avg.	0.017	0.017	0.020	0.014	0.012				

## DEFINITION

Emergency AC Power System is defined as the sum of the emergency diesel generator unavailabilities divided by the number of emergency generators at a station. Data is collected at the train level. The emergency generator includes subsystems such as air start, lube oil, fuel oil, cooling water, etc. However, for this safety system performance indicator, unavailable hours are counted only when the emergency generator is unavailable to start or load-run. For example, if a component fails in one train of a redundant support system the emergency generator is still operable, and no unavailable hours are counted.

## STATISTICAL SUMMARY

	3rd Qtr 1996	3-yr Avg Ending 9/96
PTN 3A&B	0.001	0.006
PTN 4A&B	0.002	0.003
PSL 1A&B	0.001	0.019
PSL 2A&B	0.015	0.013
Targets:		
1995 Y-E Target	≤ 0.015	≤ 0.016
1996 Y-E Target	≤ 0.025	≤ 0.025

## INDUSTRY PERFORMANCE

WANO	Unavailability
7/93 - 6/96 Median (PWR)	0.009 - 0.014
2000 Goal	0.025

## PERFORMANCE SUMMARY

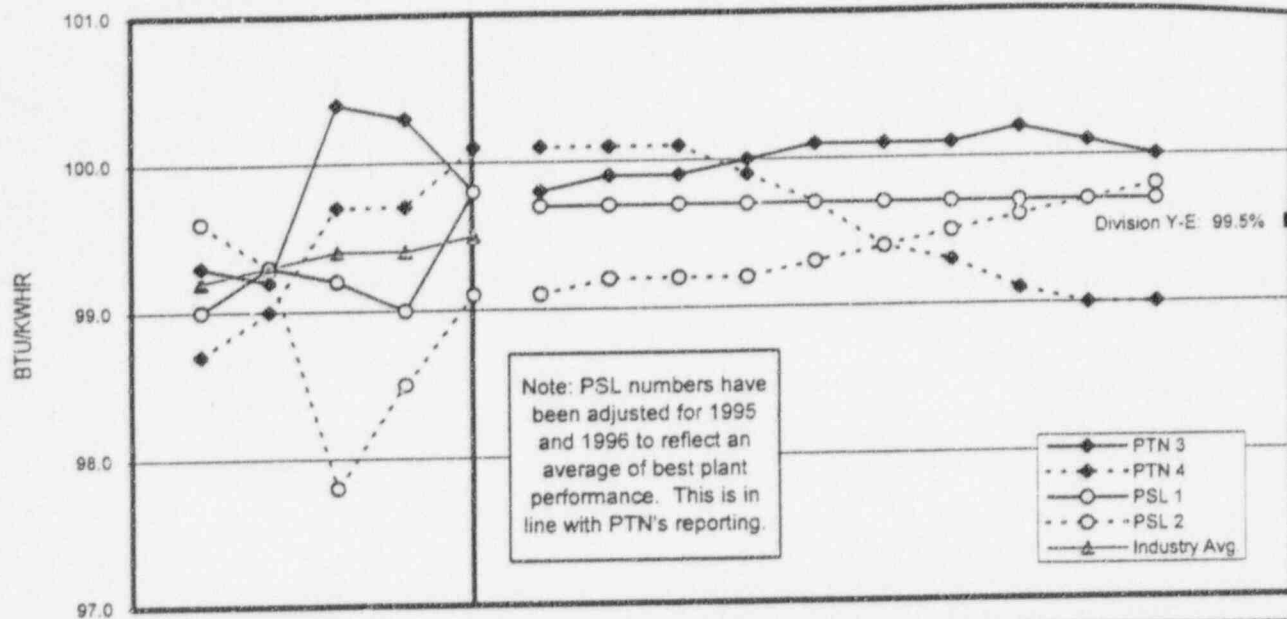
See Page ix for comments.

Data Providers: (PTN) Brian Dunn 246-6173/Jennifer Murphy 246-6827  
(PSL) Chuck Wood 467-7034/Roger Kulavich 467-7080



# THERMAL PERFORMANCE

(12-Month Running Average)



Unit	1991	1992	1993	1994	1995	1/96	2/96	3/96	4/96	5/96	6/96	7/96	8/96	9/96	10/96	11/96	12/96
PTN 3	99.3	99.2	100.4	100.3	99.8	99.8	99.9	99.9	100.0	100.1	100.1	100.1	100.2	100.1	100.0		
PTN 4	98.7	99.0	99.7	99.7	100.1	100.1	100.1	100.1	99.9	99.7	99.4	99.3	99.1	99.0	99.0		
PSL 1	99.0	99.3	99.2	99.0	99.8	99.7	99.7	99.7	99.7	99.7	99.7	99.7	99.7	99.7	99.7		
PSL 2	99.6	99.3	97.8	98.5	99.1	99.1	99.2	99.2	99.2	99.3	99.4	99.5	99.6	99.7	99.8		
Industry Avg.	99.2	99.3	99.4	99.4	99.5												

## DEFINITION

Thermal Performance is the ratio of the design gross heat rate (corrected) to the adjusted actual gross heat rate. Gross heat rate is defined as the ratio of total thermal energy produced by the reactor core to the total gross electrical energy produced by the generator during a given time period. Design gross heat rate (corrected) is the minimum theoretical heat rate that can be attained at design operating conditions for 100 percent power, expressed in British thermal units (BTUs) per kilowatt-hour(electric). Adjusted actual gross heat rate is the gross heat rate attained in the normal equipment lineup during one 24-hour period each month, expressed in BTUs per kilowatt-hour(electric) - power level should be greater than 80 percent.

## STATISTICAL SUMMARY

	Oct	12-Mo. Average
PTN 3	99.3%	100.0%
PTN 4	99.0%	99.0%
PSL 1	100.0%	99.7%
PSL 2	99.6%	99.8%

1996 Target (all units): 99.5%

## INDUSTRY PERFORMANCE

### WANO

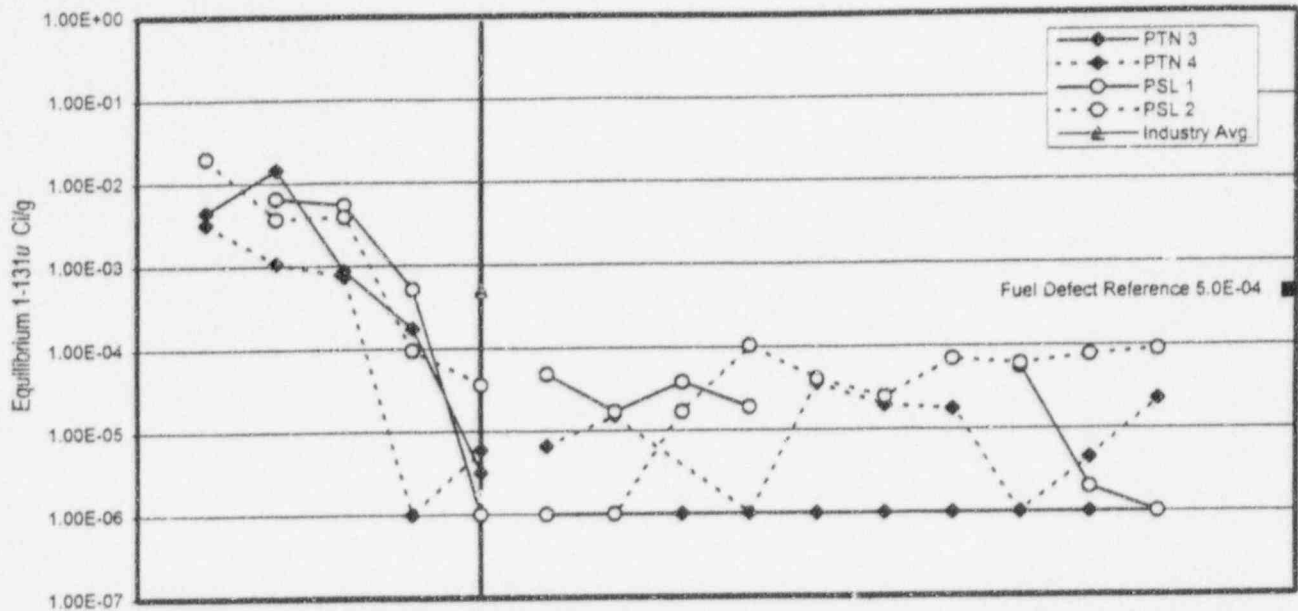
7/95 - 6/96 Median	99.6%
2000 Goal	99.5%

## PERFORMANCE SUMMARY

Data Providers: (PTN) Jose Donis 246-6008 and (PSL) Marty Smit 467-7079

## FUEL RELIABILITY

(Fuel Cycle/Monthly)



Unit	Previous	Cycles	1/96	2/96	3/96	4/96	5/96	6/96	7/96	8/96	9/96	10/96	11/96	12/96
PTN 3	4.48E-3	1.44E-2	8.70E-4	1.75E-4	3.16E-6	1.00E-6	1.00E-6	1.00E-6	1.00E-6	1.00E-6	1.00E-6	1.00E-6	1.00E-6	
PTN 4	3.24E-3	1.08E-3	7.43E-4	1.00E-6	5.98E-6	6.58E-6	1.51E-5	1.00E-6	3.84E-5	1.96E-5	1.75E-5	1.00E-6	4.48E-6	2.25E-5
PSL 1		6.54E-3	5.41E-3	5.17E-4	1.00E-6	4.82E-5	1.89E-5	3.81E-5	1.89E-5		5.81E-5	1.99E-6	1.00E-6	
PSL 2	2.01E-2	2.71E-3	3.82E-3	8.50E-5	3.62E-5	1.00E-6	1.00E-6	1.69E-5	1.04E-4	4.06E-5	2.45E-5	7.00E-5	8.08E-5	7.71E-5
Industry Avg.					5.00E-4									

### DEFINITION

This indicator is defined as the steady-state primary coolant iodine-131 activity (microcuries/gram) corrected for the tramp contribution and power level, and normalized to a common purification rate and average linear heat generation rate. The indicator value is calculated based on the average of the three monthly values for the most recent quarter of steady-state operation above 85 percent power. Steady state is defined as continuous operation for at least three days at a power level that does not vary more than  $\pm 5$  percent.

Note: If a calculated monthly value for a unit is less than 1.0E-6 microcuries per gram, the value is replaced by 1.0E-6 microcuries per gram.

### STATISTICAL SUMMARY

	Oct	Cycle No.
PTN 3	1.00E-06	15
PTN 4	2.25E-05	16
PSL 1	1.00E-06	14
PSL 2	8.90E-05	9

Division Target for each unit:  $\leq 4.50E-03$

### INDUSTRY PERFORMANCE

#### WANO

7/96 - 6/96 Fuel Defect Reference Threshold (PWR)	5.00 E-04
1994 Median	7.36 E-05
1994 Best Quartile	3.00 E-06

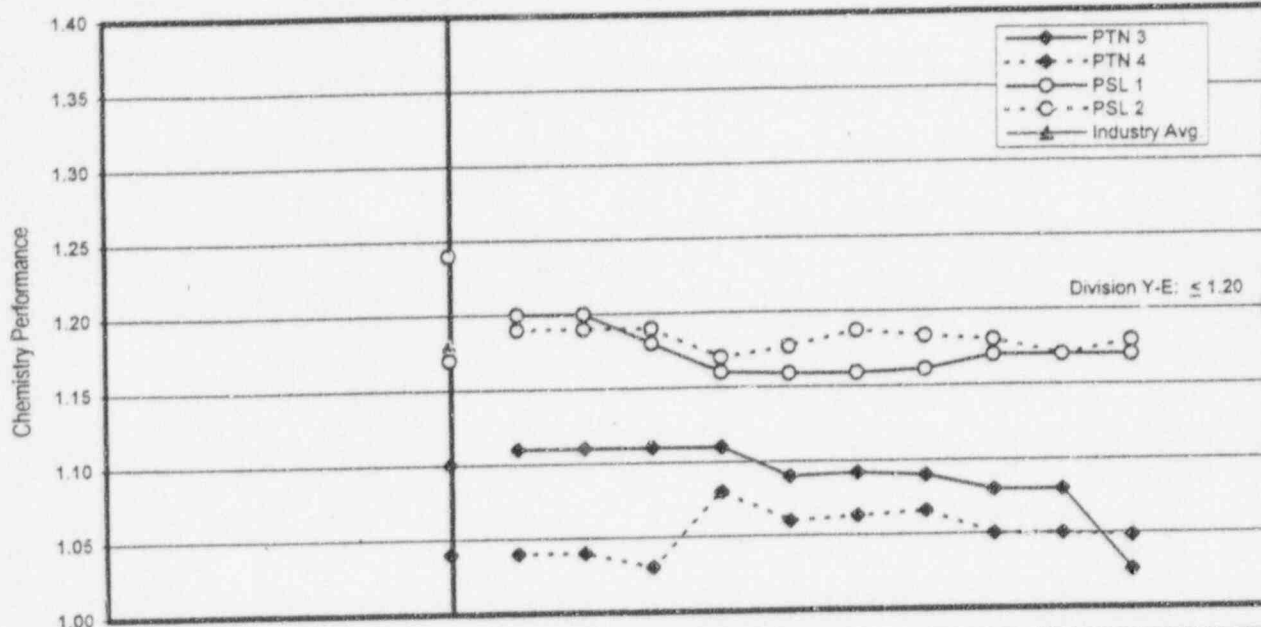
### PERFORMANCE SUMMARY

Preliminary review of the Reactor Coolant System radioisotopic data for all units indicates zero defects.

Data Provider: Modesto Jimenez 694-3323

# CHEMISTRY INDEX

(12-Months Ending Weighted Average)



Unit	1991	1992	1993	1994	1995	1/96	2/96	3/96	4/96	5/96	6/96	7/96	8/96	9/96	10/96	11/96	12/96
PTN 3					1.10	1.11	1.11	1.11	1.11	1.09	1.09	1.09	1.08	1.08	1.03		
PTN 4					1.04	1.04	1.04	1.03	1.08	1.06	1.06	1.07	1.05	1.05	1.05		
PSL 1					1.24	1.20	1.20	1.18	1.16	1.16	1.16	1.16	1.17	1.17	1.17		
PSL 2					1.17	1.19	1.19	1.19	1.17	1.18	1.19	1.18	1.18	1.17	1.18		
Industry Avg					1.18												

## DEFINITION

The Chemistry Index compares the concentration of selected impurities to the limiting values for those impurities. Each impurity value is divided by the limiting value for the impurity, and the sum of these ratios is normalized to 1.0. The limiting values are the "achievable values" defined by international industry-accepted values.

## STATISTICAL SUMMARY

	Oct	12-Mo. Ending Weighted Avg
PTN 3	1.00	1.03
PTN 4	1.01	1.05
PSL 1	1.19	1.17
PSL 2	1.25	1.18

1996 Target for each unit:  $\leq 1.20$

## INDUSTRY PERFORMANCE

### WANO

(PWR's with Recirculating Steam Generators without Molar Ratio Control)

7/95 - 6/96 Median	1.16
1995 Lowest Chemistry Index Value Attainable	1.00
Year 2000 Goal	1.10

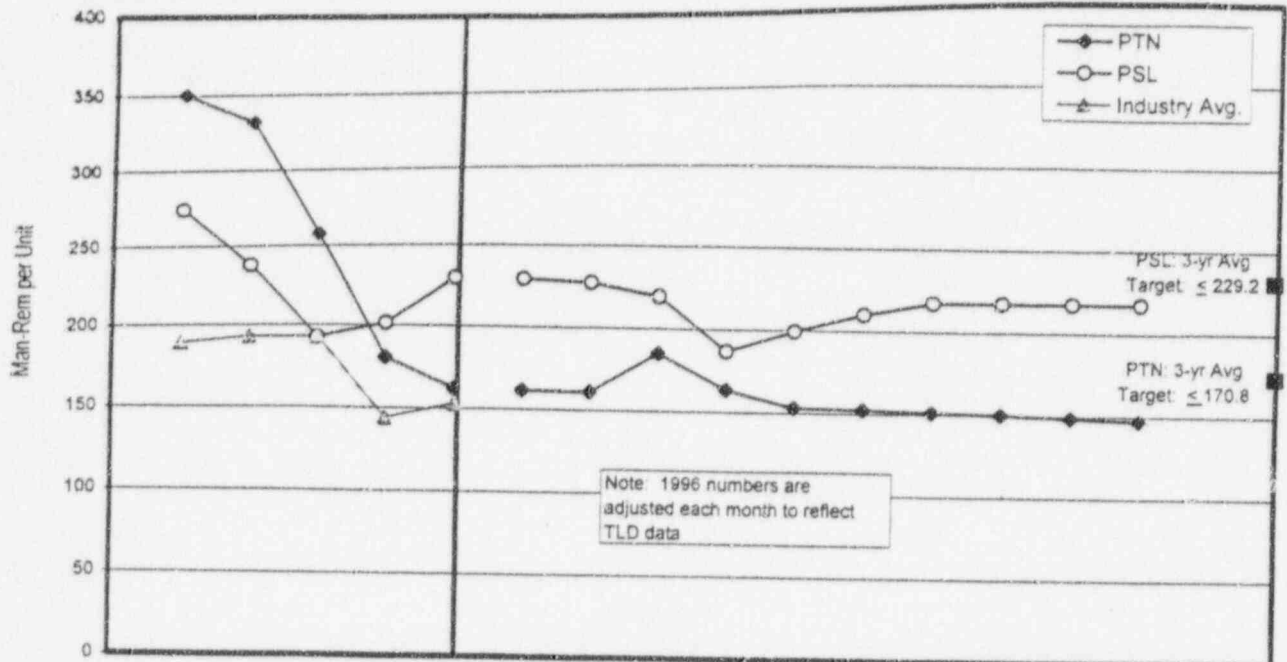
## PERFORMANCE SUMMARY

See Page x for comments.

Data Providers: (PTN) R. Steinke 246-6118/Jim Berg 246-6953  
(PSL) R. Frechette 465-3213/Dave Faulkner 465-3393

# RADIATION EXPOSURE

(3-year Running Average)



Plant	1991	1992	1993	1994	1995	1/96	2/96	3/96	4/96	5/96	6/96	7/96	8/96	9/96	10/96	11/96	12/96
PTN	350	332	257	181	162	162	161	185	184	153	152	151	150	149	147		
PSL	274	238	193	202	230	229	227	219	187	200	210	218	218	218	218		
Industry Avg.	189	193	193	144	153												

## DEFINITION

Collective Radiation Exposure is the total external whole-body dose received by all personnel (including contractors and visitors) coming on site during a time period, as measured by the primary dosimeter, thermoluminescent dosimeter (TLD) or film badge. Exposure measured by direct reading dosimeters should be included only for those periods or situations when more accurate data is not available to the utility from TLD's or film badges. In order to correlate this indicator with the new 10 CFR 20 reporting guidelines, U.S. utilities report deep dose equivalent (DDE) and the total effective dose equivalent (TEDE).

## STATISTICAL SUMMARY

3-yr Running per Unit

PTN 147.4\*

PSL 217.6\*

Division Targets: 3-yr Avg

PTN: Y-E thru 1996  $\leq 170.8$

PSL: Y-E thru 1996  $\leq 229.2$

## INDUSTRY PERFORMANCE

WANO

Man-Rem  
(per Unit)

(PWR's) Median 3-yr Distribution (7/93 - 6/96) 153

7/93 - 6/96 Median 136

2000 Goal 110

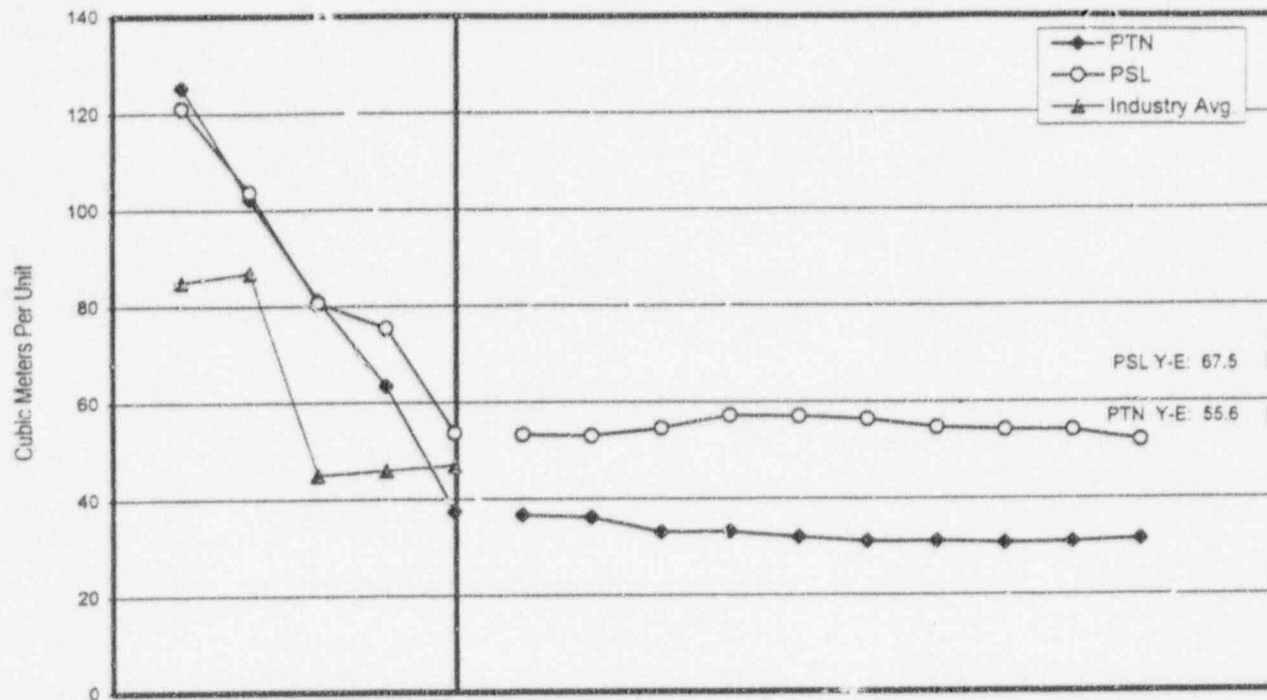
## PERFORMANCE SUMMARY

See Page x for comments.

\* Current month data is DRD; TLD was not available at time of publication.

# LOW-LEVEL SOLID RADIOACTIVE WASTE

(Cubic Meters per unit per year, 3-year Running Average)



Plant	1991	1992	1993	1994	1995	1/96	2/96	3/96	4/96	5/96	6/96	7/96	8/96	9/96	10/96	11/96	12/96
PTN	125.3	102.3	81.2	63.4	37.4	36.7	36.2	33.1	33.2	32.0	31.1	31.1	30.8	31.0	31.5		
PSL	121.1	103.8	80.7	75.4	53.7	53.3	53.0	54.6	57.2	57.0	56.4	54.7	54.2	54.1	52.0		
Industry Avg.	85.0	87.0	45.0	46.0	47.0												

## DEFINITION

This indicator is defined as the volume of low-level solid radioactive waste that has been processed and is in final form (for example, compacted or solidified) ready\* for disposal during a given period. It is calculated using the amount of waste in final form, including the container, actually shipped for disposal from both on-site and off-site facilities, plus the change in inventory of final-form waste in storage at both on-site and off site facilities.

\* in transit to or at disposal site for disposal

## STATISTICAL SUMMARY

3-yr Running thru Oct (per unit)

PTN	31.5
PSL	52.0
Division Targets:	
PTN 1996 Y-E	55.6 cu. mtr. per unit
PSL 1996 Y-E	67.5 cu. mtr. per unit

## INDUSTRY PERFORMANCE

WANO

Monthly Cu. Ft.  
(per unit)

3-yr Distribution Median (7/93 - 6/96)	46.0
7/95 - 6/96 Median	32.0
2000 Goal	45.0

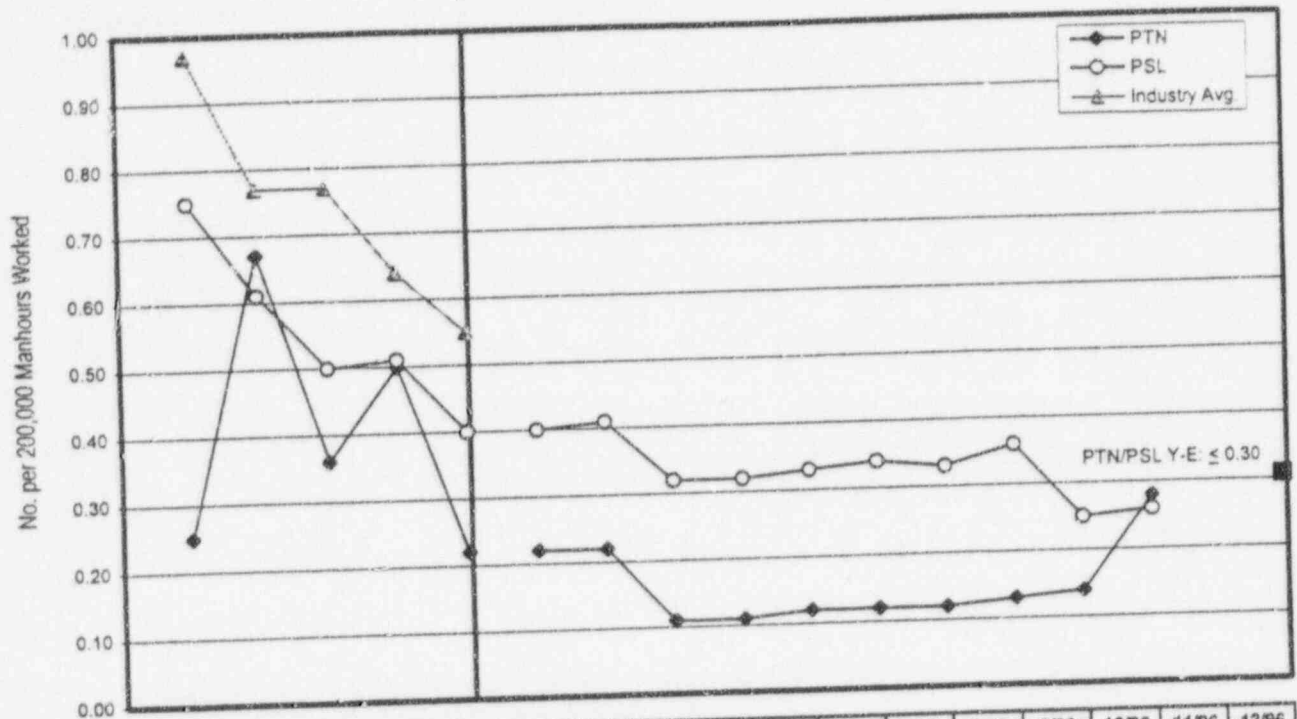
## PERFORMANCE SUMMARY

See Page x for comments.

Data Providers: (PTN) Bob Schuber 246-7220 and (PSL) Bruce Somers 467-7305



# INDUSTRIAL SAFETY ACCIDENT RATE (12-Month Running Average)



Plant	1991	1992	1993	1994	1995	1/96	2/96	3/96	4/96	5/96	6/96	7/96	8/96	9/96	10/96	11/96	12/96
PTN	0.25	0.67	0.36	0.50	0.22	0.22	0.22	0.11	0.11	0.12	0.12	0.12	0.13	0.14	0.28		
PSL	0.75	0.61	0.50	0.51	0.40	0.40	0.41	0.32	0.32	0.33	0.34	0.33	0.36	0.25	0.26		
Industry Avg.	0.97	0.77	0.77	0.64	0.55												

## DEFINITION

Industrial Safety Accident Rate is defined as the number of accidents per 200,000 man-hours worked for all utility personnel permanently assigned to the station that result in any of the following: (1) one or more days of restricted work (excluding the day of the accident); (2) one or more days away from work (excluding the day of the accident); and, (3) fatalities. Contractor personnel are not included for this indicator.

$$\text{Industrial Safety Accident Rate} = \frac{(\text{number of restricted-time} + \text{lost-time accidents} + \text{fatalities}) \times 200,000}{(\text{number of station man-hours worked})}$$

## STATISTICAL SUMMARY

12-Mo -Ending (Oct)

PTN	0.28
PSL	0.26
Division Targets:	
PTN 1996 Year-End	0.30
PSL 1996 Year-End	0.30

## INDUSTRY PERFORMANCE

WANO

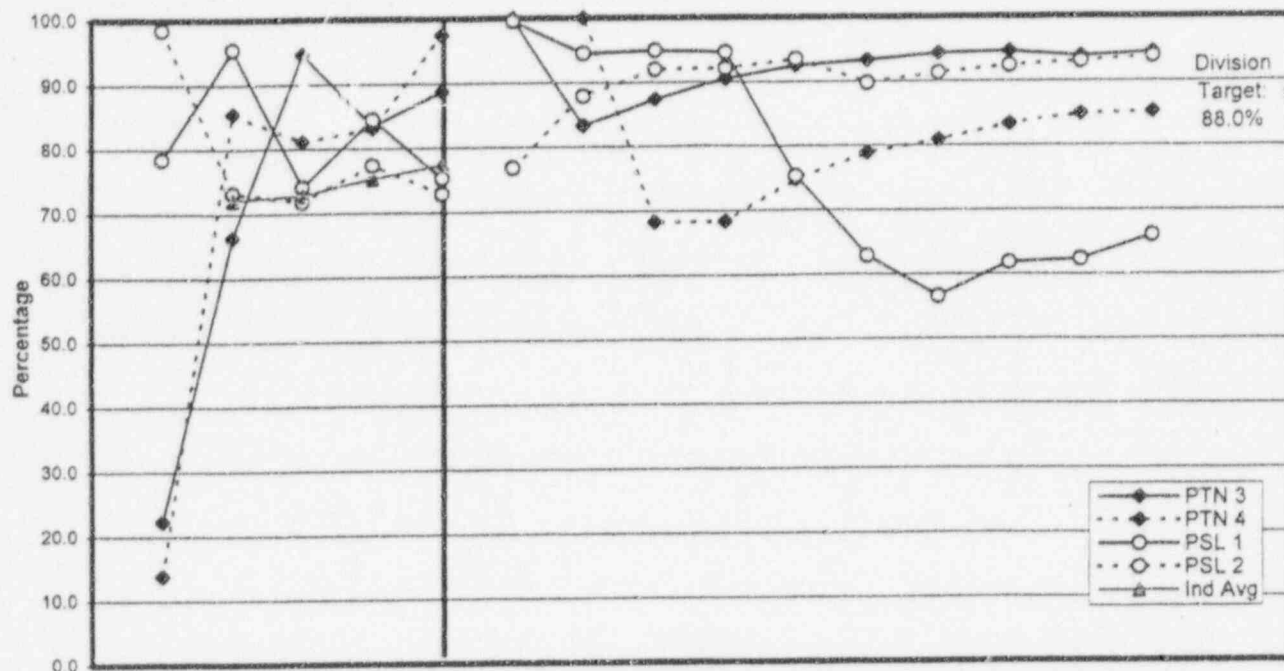
7/95 - 6/96 Median	0.44
2000 Goal	0.40

## PERFORMANCE SUMMARY

No Lost Time Injuries were reported in October.

# EQUIVALENT AVAILABILITY FACTOR

(Year-to-Date)



Unit	1991	1992	1993	1994	1995	1/96	2/96	3/96	4/96	5/96	6/96	7/96	8/96	9/96	10/96	11/96	12/96
PTN 3	22.3	66.3	94.6	83.5	88.7	99.9	83.3	87.4	90.5	92.5	93.3	94.3	94.6	93.8	94.4		
PTN 4	13.9	85.5	81.1	83.1	97.4	100.0	100.0	68.3	68.4	74.8	78.9	80.9	83.3	84.9	85.2		
PSL 1	78.5	95.4	74.0	84.5	75.3	99.6	94.5	94.9	94.6	75.3	62.9	56.6	61.8	62.4	66.1		
PSL 2	98.5	73.2	71.8	77.4	72.9	76.8	87.9	92.0	92.1	93.5	89.8	91.3	92.4	93.1	93.8		
Ind Avg		72.2	72.8	75.3	77.4												

## DEFINITION

Equivalent Availability Factor (EAF) is the ratio of the actual energy production capability to the energy that would be produced operating at full power for the same period expressed as a percent. Equivalent Availability provides an indication of the effectiveness of plant programs and practices in maximizing electrical generation.

$$\text{EAF\%} = \frac{\text{Available Hours} - (\text{Equivalent Unit Derated Hours})}{\text{Period Hours}} \times 100\%$$

## STATISTICAL SUMMARY

	Oct	Y-T-D	Y-T-D Target	3-yr Run. Avg
PTN 3	100.0%	94.4%	95.0%	89.2%
PTN 4	88.2%	85.2%	79.7%	89.4%
PSL 1	98.5%	66.1%	75.2%	77.0%
PSL 2	100.0%	93.8%	95.0%	81.5%
Division	96.7%	84.9%	86.2%	84.3%

1996 Division Y-E Target: 88.0%  
1996 Year-End Forecast: 86.5%

## INDUSTRY PERFORMANCE

### NERC/GADS

1995 (All types)	75.7%
1993-1995 (All types)	72.7%

### WANO

7/95 - 6/96 Median	82.3%
7/95 - 6/96 Average	77.3%

## PERFORMANCE SUMMARY

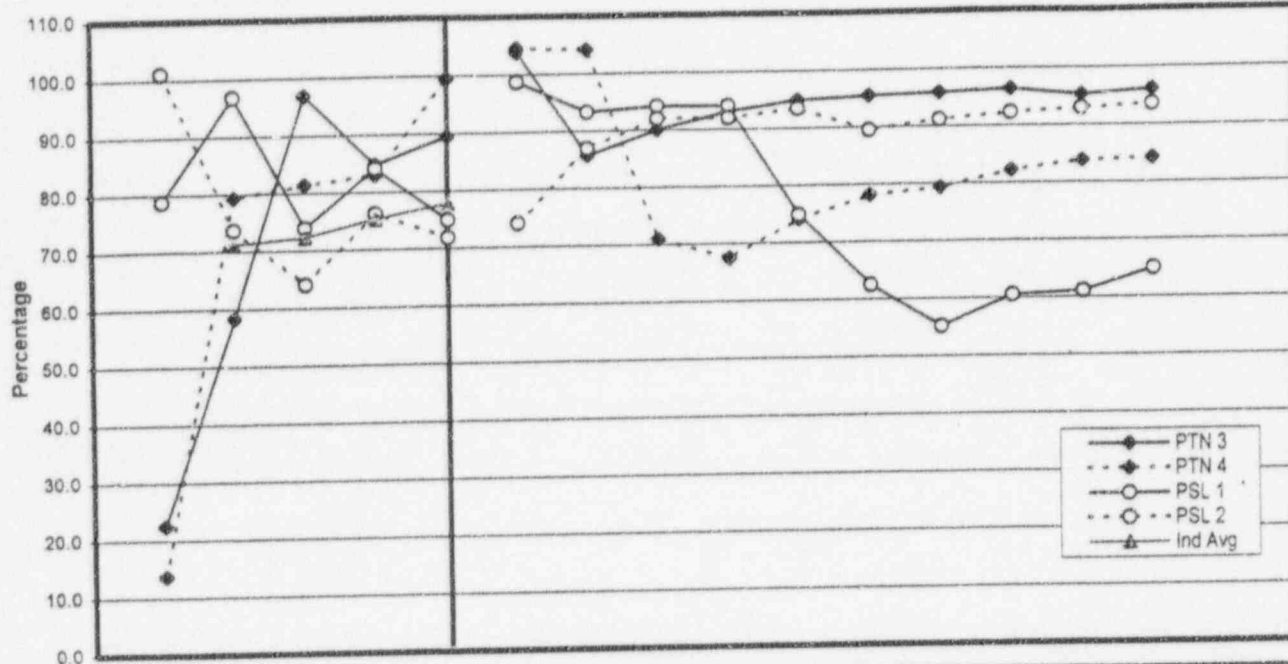
Equivalent Availability Factor performance in October was as follows:

- PTN Unit 3 operated at full power in October; EAF Y-T-D was 94.4% which is slightly below the Y-T-D target of 95.0%.
- PTN Unit 4 EAF for the month was 88.2%; power losses were attributed to RPI problems (0.2 hours) and repairs to the 5A Feedwater Heater Tube Side Relief Valve (11.2 hours), 4A SGFP Suction Relief Valve (2.5 hours), and 2A Low Pressure Feedwater Heater Tube (73.8 hours). EAF Y-T-D was 85.2% which is higher than the Y-T-D target of 79.7%.
- PSL Unit 1 EAF was 98.5%. Power losses were the result of leaks to the 4B LP Heater (0.6 hours) and Condenser Tube (10.8 hours). Y-T-D Performance of 66.1% was below the Y-T-D target of 75.2%.
- PSL Unit 2 operated at 100.0% for the month; Y-T-D EAF was 93.8% which is slightly below the Y-T-D target of 95.0%.

Data Provider: Sharon Bilger 694-4255

# CAPACITY FACTOR (MDC NET)

(Year-to-Date)



Unit	1991	1992	1993	1994	1995	1/96	2/96	3/96	4/96	5/96	6/96	7/96	8/96	9/96	10/96	11/96	12/96
PTN 3	22.5	58.4	97.0	84.8	89.5	103.9	85.7	90.0	93.1	94.9	95.5	95.9	96.4	95.3	96.2		
PTN 4	13.7	79.3	81.4	83.0	99.5	104.5	104.2	71.0	67.5	74.1	78.1	79.3	82.2	83.8	84.2		
PSL 1	78.8	96.9	73.9	84.1	75.0	98.8	93.3	94.2	94.1	74.9	62.5	55.2	60.5	61.1	65.0		
PSL 2	101.1	73.7	64.1	76.3	71.9	74.1	87.0	92.0	92.0	93.5	89.5	91.2	92.3	93.0	93.7		
Ind Avg		71.2	72.3	75.4	77.9												

## DEFINITION

Capacity Factor (CF) is the index of the actual electrical energy produced by the unit with respect to its potential.

$$\text{Capacity Factor} = \frac{\text{Net Electrical Generation}}{\text{Maximum Dependable Capacity (839 or 666) X Period Hours}} \times 100$$

## STATISTICAL SUMMARY

	Oct	Y-T-D	12-mo. ending
PTN 3	104.9%	96.2%	97.0%
PTN 4	87.2%	84.2%	87.0%
PSL 1	99.1%	65.0%	70.2%
PSL 2	99.8%	93.7%	78.0%
Division	97.7%	82.1%	83.0%

## INDUSTRY PERFORMANCE

<u>NERC/GADS</u>	
1995 All Types	75.2%
1991-1995 All Types	71.3%
<u>WANO</u>	
7/95 - 6/96 Median	81.1%
7/95 - 6/96 Average	77.9%

## PERFORMANCE SUMMARY

Capacity Factor performance in October was affected by:

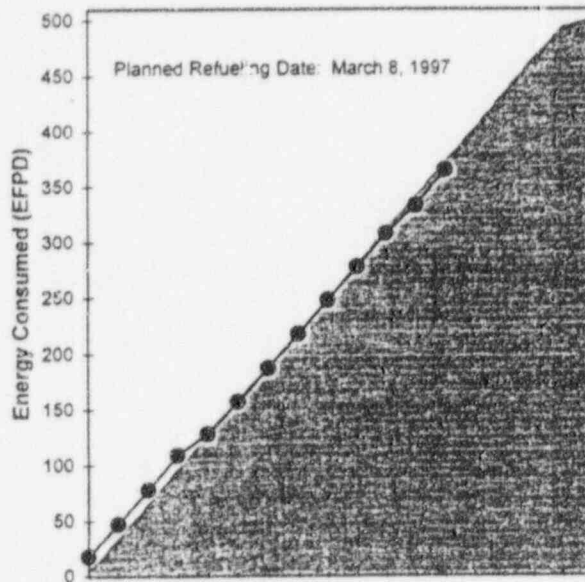
- PTN Unit 4 - RPI problems and repairs to the 5A Feedwater Heater Tube Side Relief Valve, 4A SGFP Suction Relief Valve, and 2A Low Pressure Feedwater Heater Tube.
- PSL Unit 1 - 4B LP Heater and Condenser Tube leaks.

Data Provider: Sharon Bilger 694-4255

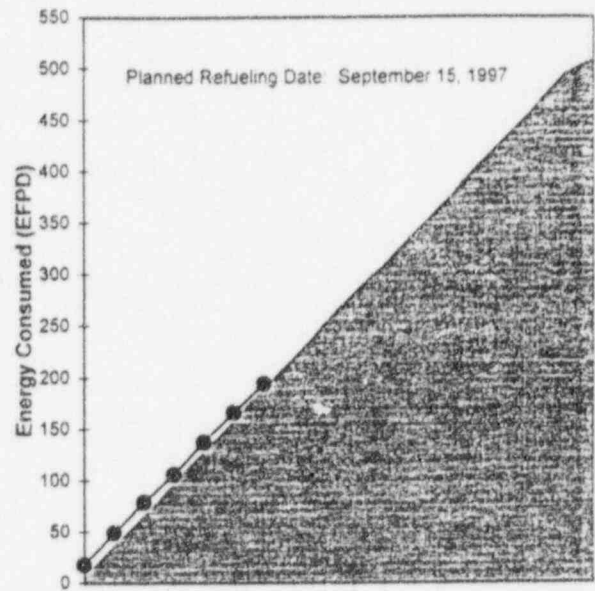


# PTN FUEL UTILIZATION FACTOR

Turkey Point Unit #3, Cycle 15



Turkey Point Unit #4, Cycle 16



PTN #3	Oct	Nov	Dec	Jan	Feb	Mar	Apr	May	Jun	Jul	Aug	Sep	Oct	Nov	Dec	Jan	Feb	Mar	Apr	May	Jun	Jul	Aug	Sep
Planned	5	35	66	98	127	158	188	219	249	281	312	342	373	403	435	466	494	501						
Actual	18	47	78	109	128	157	167	218	248	278	308	333	365											
Variance (+/-)	13	12	12	11	1	-1	-1	-1	-1	-3	-4	-9	-8											
PTN #4	Oct	Nov	Dec	Jan	Feb	Mar	Apr	May	Jun	Jul	Aug	Sep	Oct	Nov	Dec	Jan	Feb	Mar	Apr	May	Jun	Jul	Aug	Sep
Planned							6	37	67	98	129	159	190	220	251	282	310	341	371	402	432	463	494	509
Actual							18	49	79	106	137	166	194											
Variance (+/-)							12	12	12	8	8	7	4											

## DEFINITION

Fuel utilization plots the amount of nuclear energy used during the current fuel cycle. The amount of nuclear energy loaded into the core is expressed in effective full power days (EFPD). One EFPD is the equivalent of operating the reactor at maximum thermal rating (2200 at PTN or 2700 at PSL) for a 24 hour period. Planned energy is compared to actual energy used during the cycle. Fuel utilization is directly related to plant performance. The significance of variance EFPD(+/-) is the difference between planned and actual energy consumption. Fuel utilization can be used to project longer or shorter operating fuel cycles.

$$\text{Actual Energy} / \text{Calendar Days} = \text{Fuel Utilization Factor}$$

## Fuel Cycle Operating Assumptions

**PTN 3:** In accordance with the April 26, 1995 Approved Operating Schedule (AOS), Unit 3, Cycle 15 is scheduled to run 498 calendar days (October 27, 1995 to March 8, 1997) with design energy to run 501 (EFPD) days. NOTE: Due to the 34-day outage, startup began on October 7, 1995, 20 days earlier than planned.

**PTN 4:** In accordance with the February 8, 1996 AOS, Unit 4 Cycle 16 was scheduled to startup on April 25, 1996. Due to a 35-day refueling outage, the unit actually started up on April 8, 1996, 17 days earlier than planned. Note that the unit has a design energy of 507 EFPD.

## PERFORMANCE SUMMARY

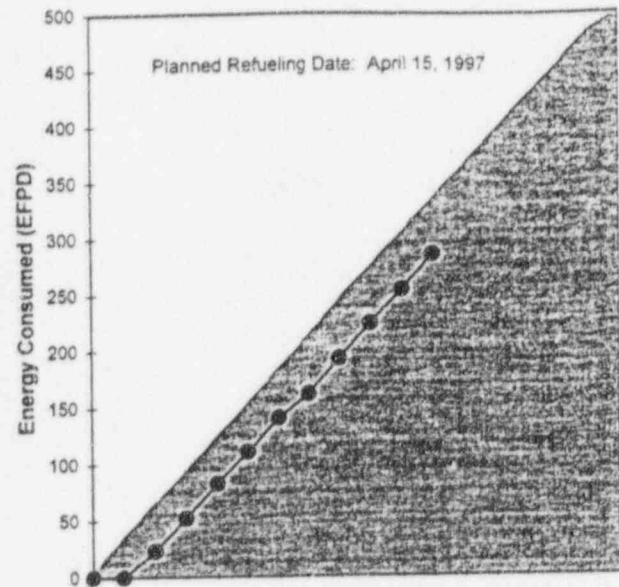
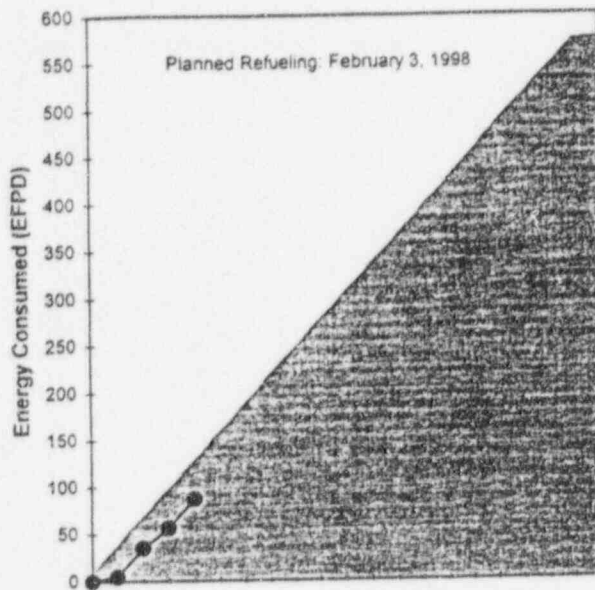
- Unit 3 achieved a Fuel Utilization Factor of 103.1% and operated for 767 effective full power hours in the month of October. For the period October 27, 1995 to October 31, 1996, the Fuel Utilization Factor was 99.4%. Power uprate occurred on 10/11/96.
- Unit 4 achieved a Fuel Utilization Factor of 90.6% and operated for 674 effective full power hours in the month of October. For the period April 8, 1996 to October 31, 1996, the Fuel Utilization Factor was 102.0%. Power uprate occurred on 10/28/96.

Data provider: Jimmie Perryman 694-3330

# PSL FUEL UTILIZATION FACTOR

St. Lucie Unit #1, Cycle 14

St. Lucie Unit #2, Cycle 9



PSL #1	Jun	Jul	Aug	Sep	Oct	Nov	Dec	Jan	Feb	Mar	Apr	May	Jun	Jul	Aug	Sep	Oct	Nov	Dec	Jan	Feb	Mar	Apr
Planned	9	39	69	98	129	158	188	218	246	276	305	335	365	395	425	454	485	514	544	573	575		
Actual	0	5	35	56	87																		
Variance (+/-)	-9	-34	-34	-42	-42																		
PSL #2																							
Planned						5	36	67	95	126	155	186	215	247	276	306	336	366	397	427	455	485	500
Actual						0	0	23	52	83	111	141	162	193	224	254	285						
Variance (+/-)						-5	-36	-44	-43	-43	-44	-45	-53	-54	-52	-52	-51						

## DEFINITION

Fuel utilization plots the amount of nuclear energy used during the current fuel cycle. The amount of nuclear energy loaded into the core is expressed in effective full power days (EFPD). One EFPD is the equivalent of operating the reactor at maximum thermal rating (2200 at PTN or 2700 at PSL) for a 24 hour period. Planned energy is compared to actual energy used during the cycle. Fuel utilization is directly related to plant performance. The significance of variance EFPD(+/-) is the difference between planned and actual energy consumption. Fuel utilization can be used to project longer or shorter operating fuel cycles.

$$\text{Actual Energy / Calendar Days} = \text{Fuel Utilization Factor}$$

## Fuel Cycle Operating Assumptions

**PSL 1:** In accordance with the February 8, 1996 Approved Operating Schedule (AOS), Unit 1, Cycle 14 was scheduled to begin operation June 22, 1996. This provided for a cycle of 591 calendar days with design energy to run 575 effective full power days (EFPD). Unit 1, Cycle 14 actually began operation July 25, 1996 and is currently scheduled to refuel February 3, 1998.

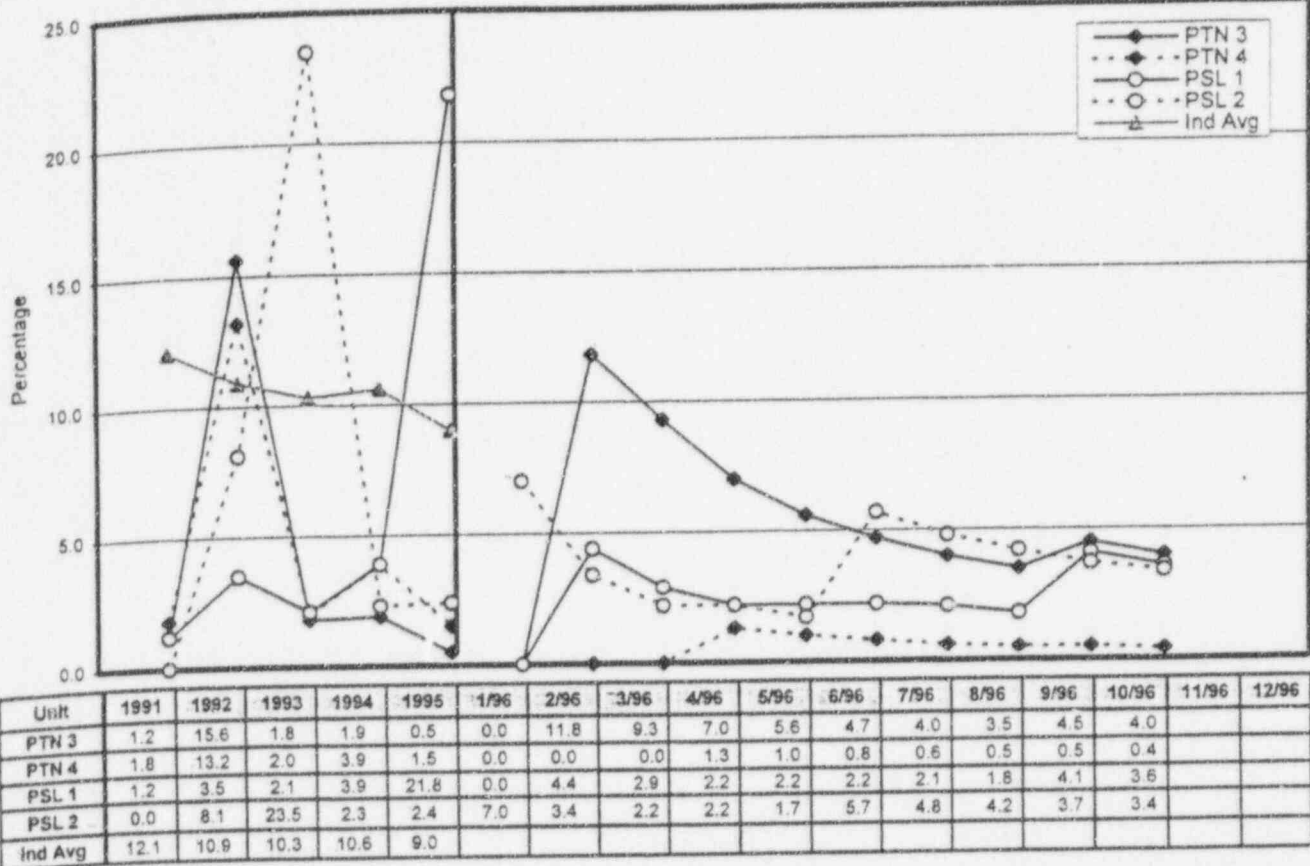
**PSL 2:** In accordance with the April 26, 1995 AOS, Unit 2, Cycle 9 was scheduled to begin operation November 24, 1995. This provided for a cycle of 508 calendar days with design energy to run 500 EFPD. Unit 2, Cycle 9 actually began operation January 5, 1996 and is currently scheduled to refuel April 15, 1997.

## PERFORMANCE SUMMARY

- Unit 1 achieved a Fuel Utilization Factor of 98.7% and operated at 734 effective full power hours in the month of October. For the period of June 22, 1996 through October 31, 1996, the Fuel Utilization Factor was 67.4%.
- Unit 2 achieved a Fuel Utilization Factor of 100.0% and operated at 744 effective full power hours in the month of October. For the period of November 24, 1995 through October 31, 1996, the Fuel Utilization Factor was 84.8%.

# FORCED OUTAGE RATE

(Year-to-Date)



## DEFINITION

Forced Outage Rate is the percentage of time that the unit was unavailable due to forced events compared to the time planned for full power operation. A forced outage exists when the unit requires immediate removal from service, i.e. the unit is not synchronized to the grid. This type of outage usually results from immediate mechanical/electrical/hydraulic controls systems and operator-initiated trips in response to unit alarms.

$$\text{Forced Outage Rate} = \frac{\text{Forced Outage Hours}}{\text{Forced Outage Hours} + \text{Service Hours}} \times 100\%$$

## STATISTICAL SUMMARY

	Oct	Y-T-D	12 Mo. Ending
PTN 3	0.0%	4.0%	3.3%
PTN 4	0.0%	0.4%	0.3%
PSL 1	0.0%	3.6%	3.2%
PSL 2	0.0%	3.4%	3.4%

## INDUSTRY PERFORMANCE

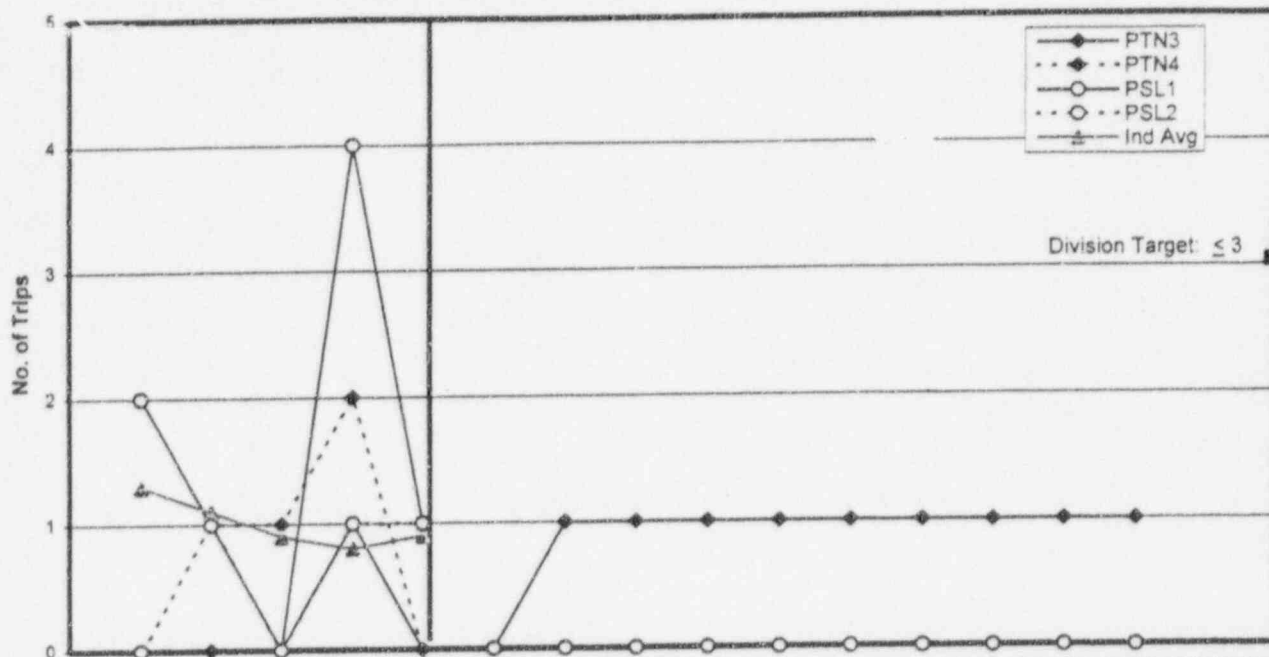
NERC/GADS	
1995 (All types)	9.0%
1991 - 1995 (All types)	10.6%

## PERFORMANCE SUMMARY

- No forced outages occurred in October.

# UNPLANNED AUTOMATIC TRIPS WHILE CRITICAL

(Year-to-Date)



Unit	1991	1992	1993	1994	1995	1/96	2/96	3/96	4/96	5/96	6/96	7/96	8/96	9/96	10/96	11/96	12/96
PTN3	0	0	0	1	0	0	1	1	1	1	1	1	1	1	1		
PTN4	0	1	1	2	0	0	0	0	0	0	0	0	0	0	0		
PSL1	2	1	0	4	1	0	0	0	0	0	0	0	0	0	0		
PSL2	0	1	0	1	1	0	0	0	0	0	0	0	0	0	0		
Ind Avg	1.3	1.1	0.9	0.8	0.9												

## DEFINITION

An Unplanned Automatic Scram is a non-manual actuation of the reactor protection system that results in a scram signal any time the unit is critical. Scrams that are planned as part of special evaluations or tests are not included in this definition. This indicator provides an indication of success in improving plant safety by reducing the number of undesirable and unplanned thermal-hydraulic and reactivity transients requiring reactor scrams.

## STATISTICAL SUMMARY

	Oct	Y-T-D	12-Month Ending
PTN 3	0	1	1
PTN 4	0	0	0
PSL 1	0	0	0
PSL 2	0	0	0
Division Total:	0	1	1

1996 Division Target:  $\leq 3$

## INDUSTRY PERFORMANCE

WANO	Trips per unit
Trips per 7000 Critical Hours	
3-yr Distribution Median (1993-95)	0.9
1995 Median	0.9
1995 Goal	1.0
NRC (1995 3rd Qtr Performance Indicator Rpt)	
Quarterly Trips Annualized	1.0
Quarterly Trips per 7000 Critical Hours	1.2

## PERFORMANCE SUMMARY

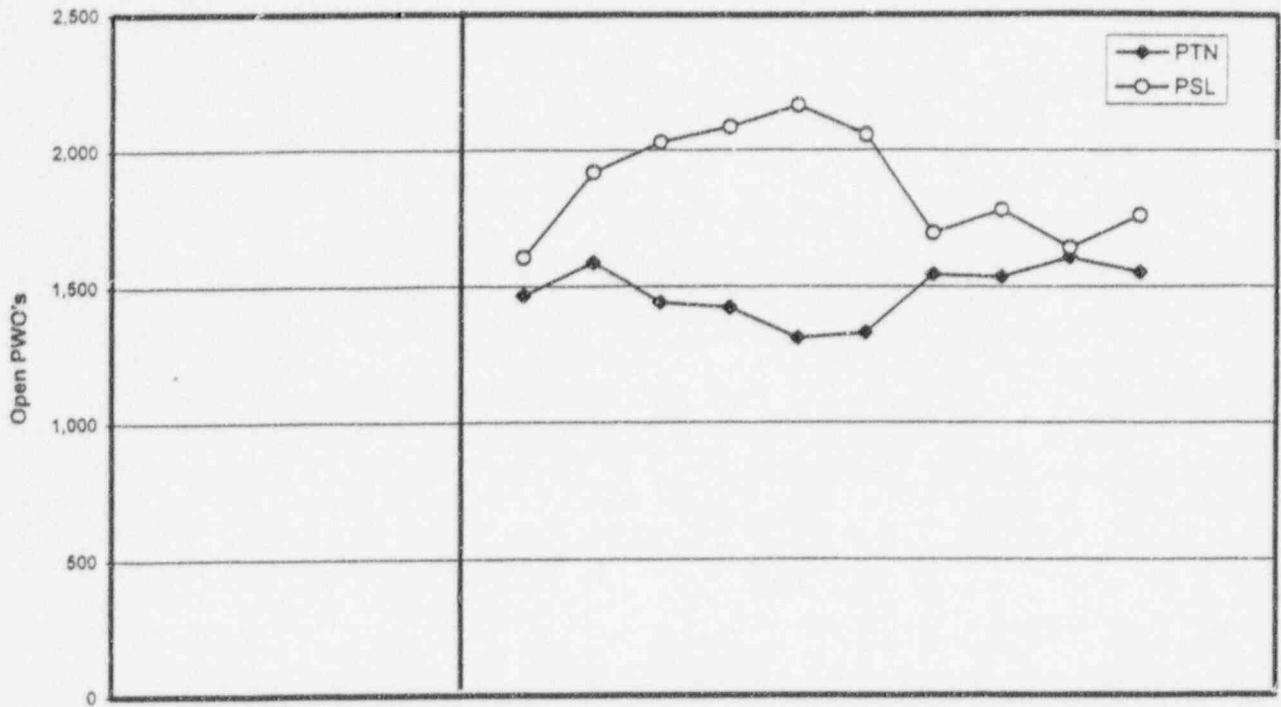
No automatic trips occurred in October 1996. Year-to-date, one automatic trip has been experienced:

- Turkey Point Unit 3 experienced an automatic trip on February 9th; the "B" Steam Generator Feed Pump was stopped to monitor its discharge check valve closing stroke which did not strike closed as expected. The resulting feed flow transient caused the "C" Steam Generator level to increase resulting in a turbine trip which tripped the reactor.

Data Providers: (PTN) J. Knorr 246-6757 (PSL) Kelly Korth 467-7054



## OPEN PWO'S



Unit	1991	1992	1993	1994	1995	1/96	2/96	3/96	4/96	5/96	6/96	7/96	8/96	9/96	10/96	11/96	12/96
PTN	*	*	*	*	*	1,466	1,589	1,441	1,422	1,310	1,330	1,544	1,536	1,608	1,553		
PSL	*	*	*	*	*	1,608	1,920	2,030	2,086	2,166	2,056	1,697	1,782	1,644	1,762		

### DEFINITION

This indicator includes Work Type 1 (Planned Miscellaneous), Work Type 3 (Projects), WorkType 5 (Trouble & Breakdown), Status 17 through 48, and all hold codes.

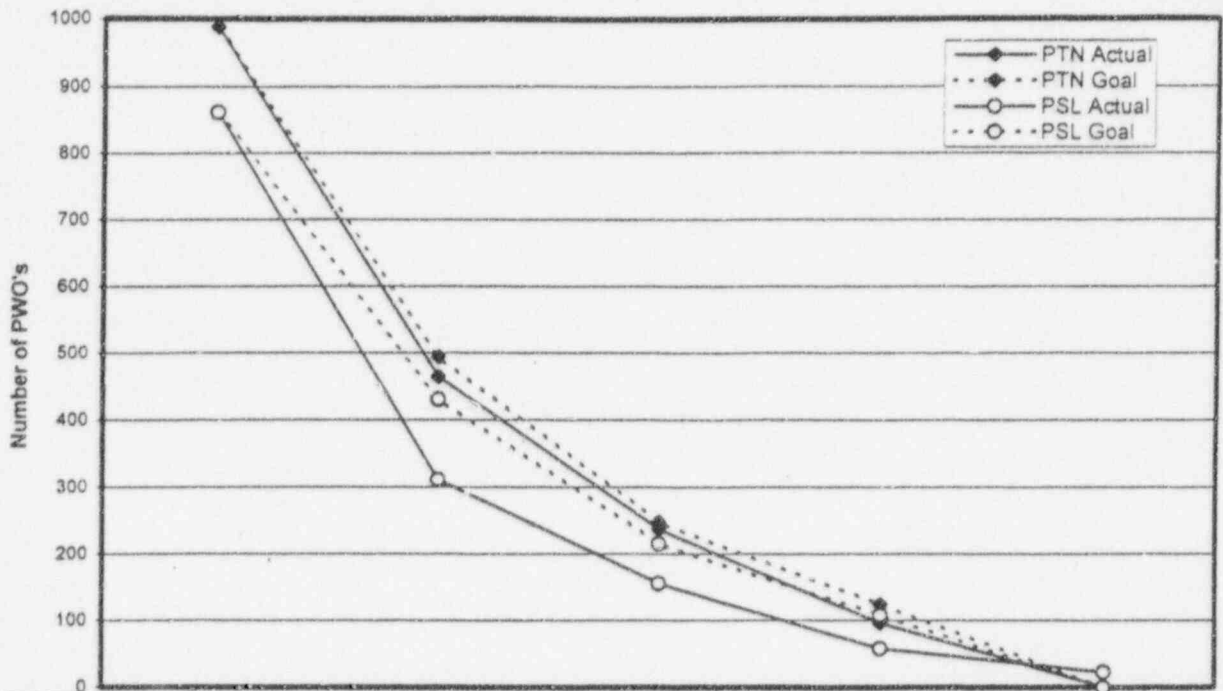
### STATISTICAL SUMMARY

### INDUSTRY PERFORMANCE

### PERFORMANCE SUMMARY

Data Providers: (PTN) Greg Heisterman 246-6796 and (PSL) Joe Marchese 467-7107

## PWO AGING CURVE



Unit	TOTALS	3 Mo. & greater	6 Mo. & greater	9 Mo. & greater	12 Mo. & greater
PTN Actual	988	464	237	96	1
PTN Goal	988	494	247	124	0
PSL Actual	862	311	156	58	23
PSL Goal	862	431	218	108	0

### DEFINITION

PWO Aging curves includes Work Type 1 (Planned Miscellaneous), Work Type 3 (Projects), WorkType 5 (Trouble & Breakdown), status 17 through 48, and excludes HC2 through HC6 and SNO. INPO definition is to be used for the graph.

Goal: To halve the backlog every three months.

### STATISTICAL SUMMARY

### INDUSTRY PERFORMANCE

### PERFORMANCE SUMMARY

Data Providers: (PTN) Greg Heisterman 246-6796 and (PSL) Joe Marchese 467-7107

## PWO'S GREATER THAN 12 MONTHS (GOLDEN OLDIES)

### TURKEY POINT

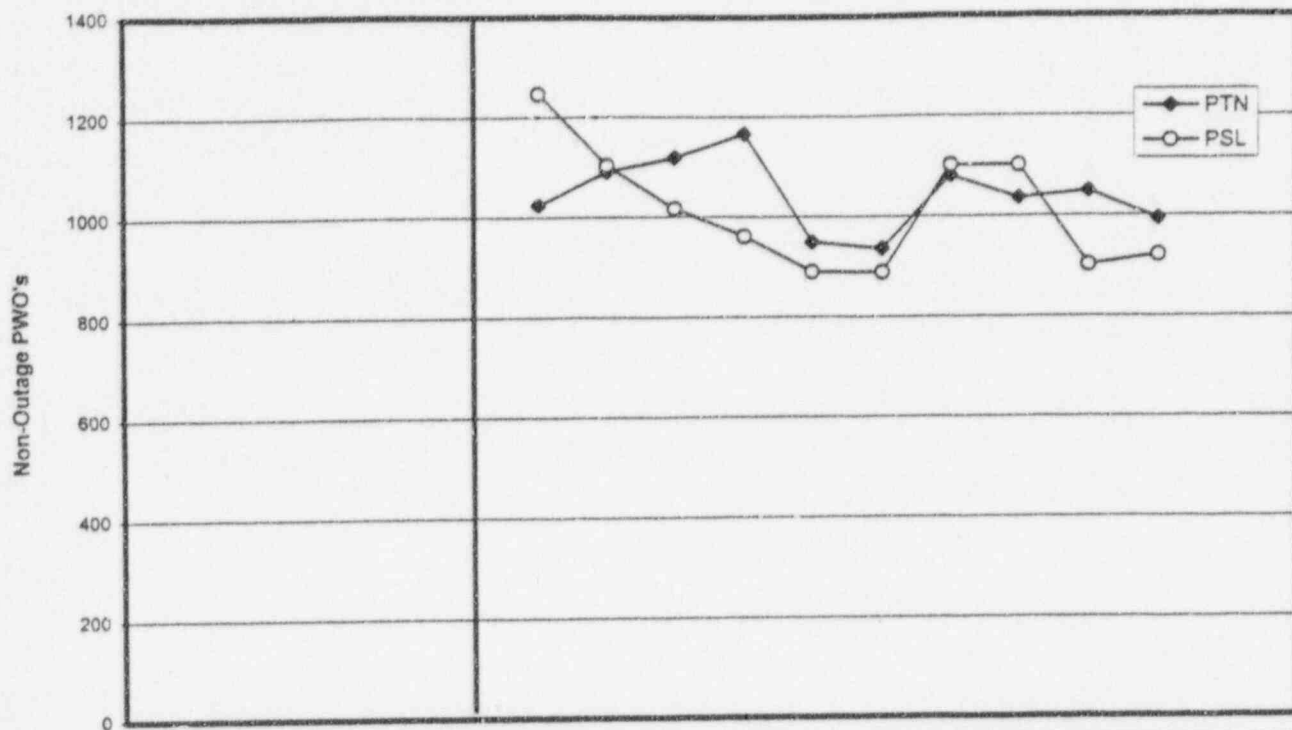
REF	PWO #	PWO TITLE	Orig. Date	Date on List
1	95030808	CONTROL BLDG ROOF TROLLEY BEAM, INSTALL TROLLEY BEAM	10/30/95	11/1/96

### ST. LUCIE

REF	PWO #	PWO TITLE	Orig. Date	Date on List
1	94017403	SUPPORT PCM-042-289 (H2 ISOL VLVS)	7/13/94	07/13/95
2	95003699	U-2 DSG 2A ANN 6.2 ALARM	2/5/95	02/05/96
3	95004362	VLV LEAKS BY SEAT	2/12/95	02/12/96
4	95005523	CHECK THE LIMITS ON SOLENOID	2/22/95	02/22/96
5	95013260	REPAIR REPLACE CONDUIT SUPPORTS	5/4/95	05/03/96
6	95013570	WILL NOT CALIBRATE, REPAIR	5/8/95	05/07/96
7	95014709	SWITCH CAUSES RELAYS TO CHATTER	5/19/95	05/18/96
8	95017293	MECH. SEAL LEAK AND FITTINGS LEAKING	6/14/95	06/13/96
9	95018290	ENG MOUNTED ELAPSED HOUR METER INCORREC	6/22/95	06/21/96
10	95020079	ALARMS WILL NOT ADJUST, REPAIR/REPLACE	7/13/95	07/12/96
11	95020533	IMPLEMENT PCM-152-994	7/25/95	07/24/96
12	95022001	REPLACE FH-1A AND FH-1 DOORS	8/9/95	08/08/96
13	95024572	REPLACE TEMP CONNECTIONS TO CALGON TIME	8/9/95	08/08/96
14	95024538	WATER LEAK	8/31/95	08/30/96
15	95030685	U/2 OVERHAUL SPARE HVE-21B MOTOR	9/29/95	09/28/96
16	95029229	NEED TO BE CALIBRATED FOR LOCAL READING	10/16/95	10/15/96
17	95029289	OSCILATING AND NOT CONTROLLING	10/16/95	10/15/96
18	95029430	PLEASE REPAIR VLV, LEAKS BY SEAT	10/17/95	10/16/96
19	95030905	U/2 4KV BKR 2AB-1 PARTS REPLACE	10/22/95	10/21/96
20	95031853	PIUMP CASING DEGRADED-REPAIR OF REPLACE	10/23/95	10/22/96
21	95030597	REPAIR CRACKS ON A/C BASE NE CORNER	10/23/95	10/22/96
22	95030690	U/2 REPLACE ARC CHUTE RETAINER	10/28/95	10/27/96
23	95030795	REPAIR AS REQUIRED	10/30/95	10/29/96

PWO Greater than 12 month's, includes Work Type 1 (Planned Miscellaneous), Work Type 3 (Projects), WorkType 5 (Trouble & Breakdown), status 17 through 48, and excludes HC2 through HC6 and SNO.

## NON-OUTAGE PWO'S



### DEFINITION

Non-Outage PWO's includes Work Type 1 (Planned Miscellaneous), Work Type 3 (Projects), WorkType 5 (Trouble & Breakdown), status 17 through 48, and excludes HC2 through HC6 and SNO.

### STATISTICAL SUMMARY

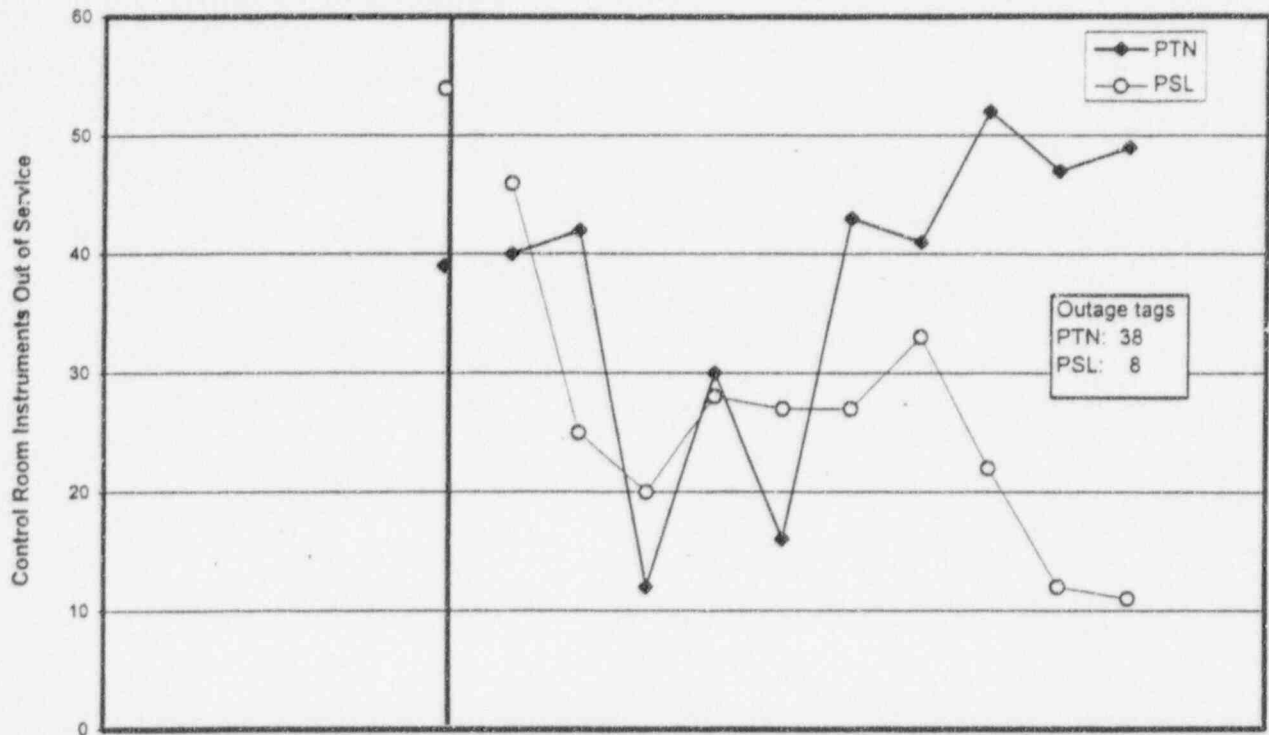
### INDUSTRY PERFORMANCE

### PERFORMANCE SUMMARY

Data Providers: (PTN) Greg Heisterman 246-6796 and (PSL) Joe Marchese 467-7107



## CONTROL ROOM INSTRUMENTS OUT-OF-SERVICE



Unit	1991	1992	1993	1994	1995	1/96	2/96	3/96	4/96	5/96	6/96	7/96	8/96	9/96	10/96	11/96	12/96
PTN					39	40	42	12	30	16	43	41	52	47	49		
PSL					54	46	25	20	28	27	27	33	22	12	11		

### DEFINITION

This indicator defines the number of control room instruments for each unit that cannot perform their design function, regardless of the reason. Instruments on the control room back panels are readily available for use by the control room crews and are included; however, instruments in adjoining areas where operators are not normally stationed (such as computer rooms) are not included. Count deficiency tags that are in status 05 to 48.

### STATISTICAL SUMMARY

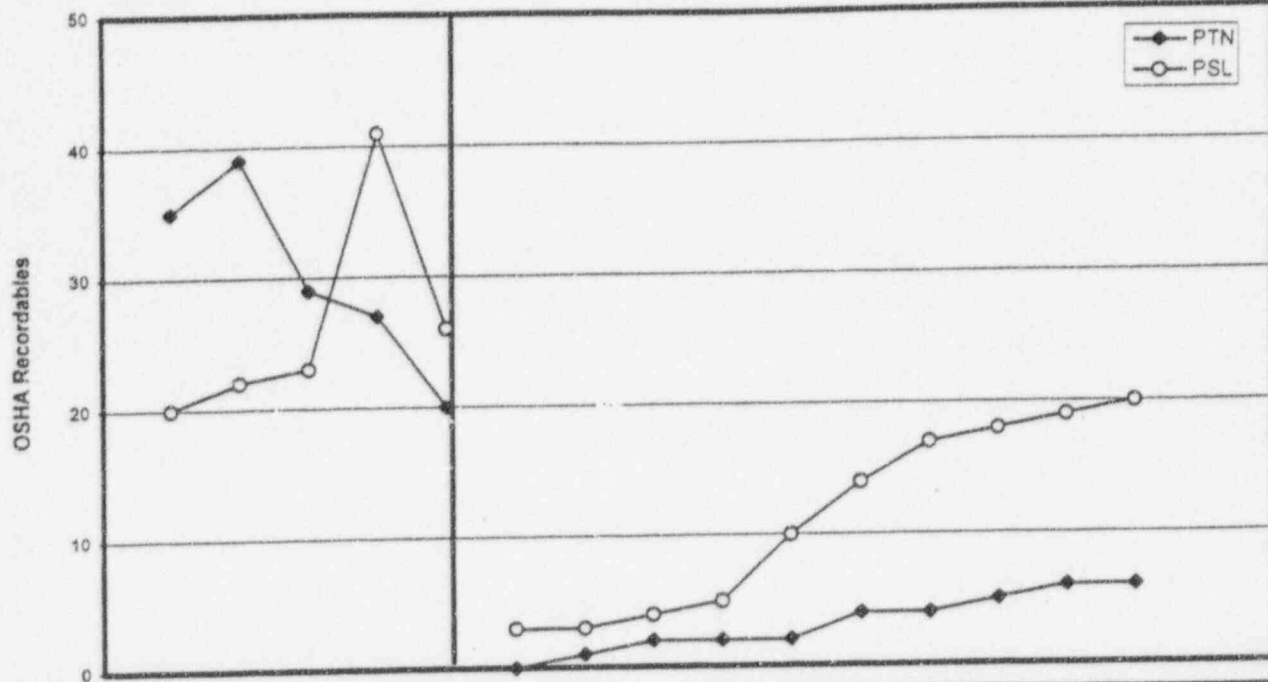
### INDUSTRY PERFORMANCE

### PERFORMANCE SUMMARY

Data Providers: (PTN) Greg Heisterman 246-6796 and (PSL) Joe Marchese 467-7107

# OSHA RECORDABLES

(Year-to-Date)



Unit	1991	1992	1993	1994	1995	1/96	2/96	3/96	4/96	5/96	6/96	7/96	8/96	9/96	10/96	11/96	12/96
PTN	35	39	29	27	20	0	1	2	2	2	4	4	5	6	6		
PSL	20	22	23	41	26	3	3	4	5	10	14	17	18	19	20		

## DEFINITION

The definition by Occupational Safety Health Administration (OSHA) is an injury occurring on the job that requires medical treatment beyond first aid as defined by 29 CFR 1904. This graph shows OSHA Recordables reported at the sites.

## STATISTICAL SUMMARY

Month Y-T-D

PTN 0 6  
PSL 0 20

Nuclear Division Y-E Target: 1.75 per 200,000 Hours Worked  
Corporate Y-E Target: 3.75 per 200,000 Hours Worked

## INDUSTRY PERFORMANCE

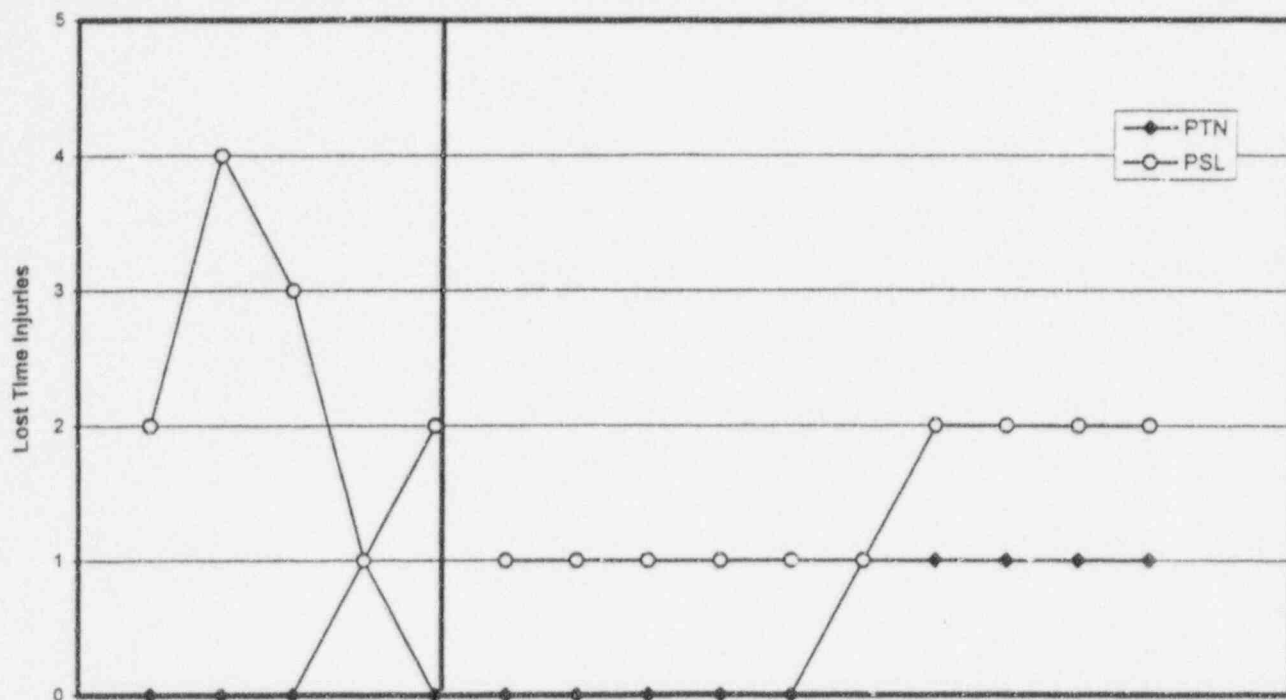
## PERFORMANCE SUMMARY

St. Lucie reported one OSHA Recordable Injury in October as a result of an employee treated for carpal tunnel syndrome.

A total of 26 OSHA Recordables have been received year-to-date.

# LOST TIME INJURIES

(Year-to-Date)



Unit	1991	1992	1993	1994	1995	1/96	2/96	3/96	4/96	5/96	6/96	7/96	8/96	9/96	10/96	11/96	12/96
PTN	0	0	0	1	0	0	0	0	0	0	1	1	1	1	1		
PSL	2	4	3	1	2	1	1	1	1	1	1	2	2	2	2		

## DEFINITION

A Lost Time Injury as defined by Occupational Safety Health Administration (OSHA) is an occupational injury that requires an employee to miss a full day (8 hour shift) beyond the day of injury.

## STATISTICAL SUMMARY

	Month	Y-T-D
PTN	0	1
PSL	0	2

## INDUSTRY PERFORMANCE

## PERFORMANCE SUMMARY

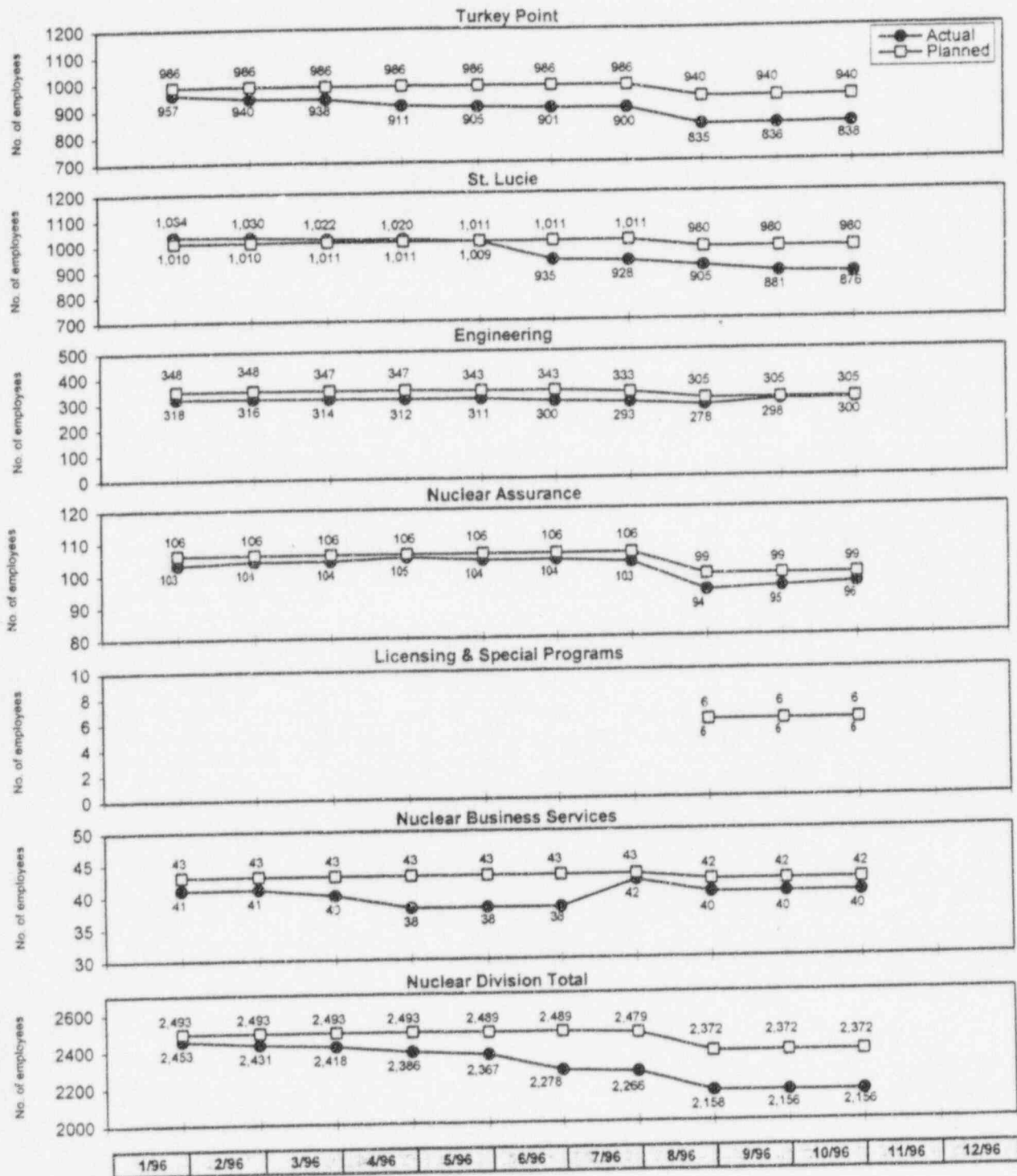
No Lost Time Injuries were reported in October.

Data providers: J. Sambito (PTN) 246-7372 and T. Moser (PSL) 467-7171

# REGULAR STAFFING

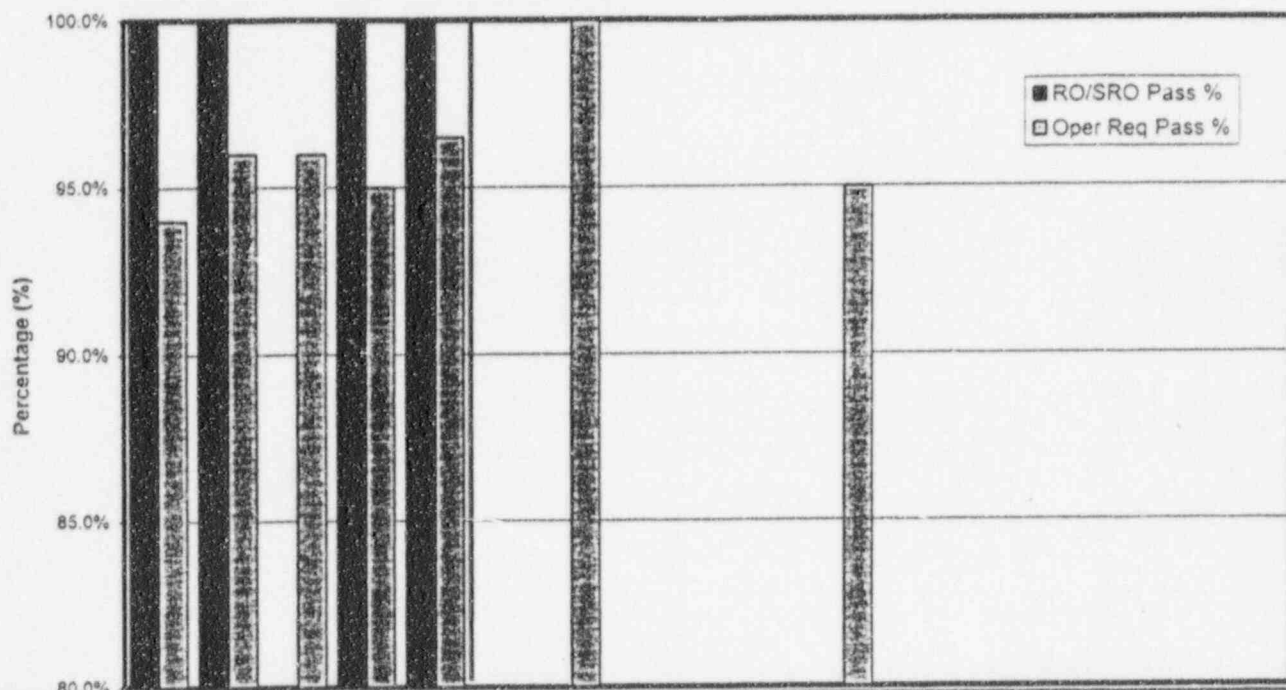
## FPL Employees and Long-Term Contractors

### by Department



## OPERATOR EXAMINATION PERFORMANCE

### Turkey Point Units 3 & 4



Exams:	1991	1992	1993	1994	1995	1/96	2/96	3/96	4/96	5/96	6/96	7/96	8/96	9/96	10/96	11/96	12/96
RO/SRO Taken	39	5	-	8	13	-	-	-	-	-	-	-	-	-	-	-	-
Passed	39	5	-	8	13	-	-	-	-	-	-	-	-	-	-	-	-
RO/SRO Pass %	100.0%	100.0%	-	100.0%	100.0%	-	-	-	-	-	-	-	-	-	-	-	-
Oper Req Taken	47	78	69	63	58	-	64	-	-	-	19	-	-	-	-	-	-
Passed	44	75	66	60	56	-	64	-	-	-	18	-	-	-	-	-	-
Oper Req Pass %	94.0%	96.0%	96.0%	95.0%	96.5%	-	100.0%	-	-	-	95.0%	-	-	-	-	-	-

### DEFINITION

Initial License Examination (RO/SRO) results are reported for all candidates taking an Initial License Exam as conducted by the NRC.

Operator Requalification Examination results are reported for both RO's and SRO's. This examination is administered annually by the utility and may be jointly administered by the NRC. Retests of operators who failed examinations are not included.

### STATISTICAL SUMMARY

Initial RO/SRO License Exams			
	No. Taken	No. Passed	Pass Rate %
YTD 1996	0	0	0.0%
YTD 1995	13	13	100.0%
Operator Requal Exams			
	No. Taken	No. Passed	Pass Rate %
YTD 1996	83	82	98.8%
YTD 1995	58	56	96.5%

### INDUSTRY PERFORMANCE

The NRC at their last Regional Training Managers Meeting (for fiscal year 1994) provided the following data:

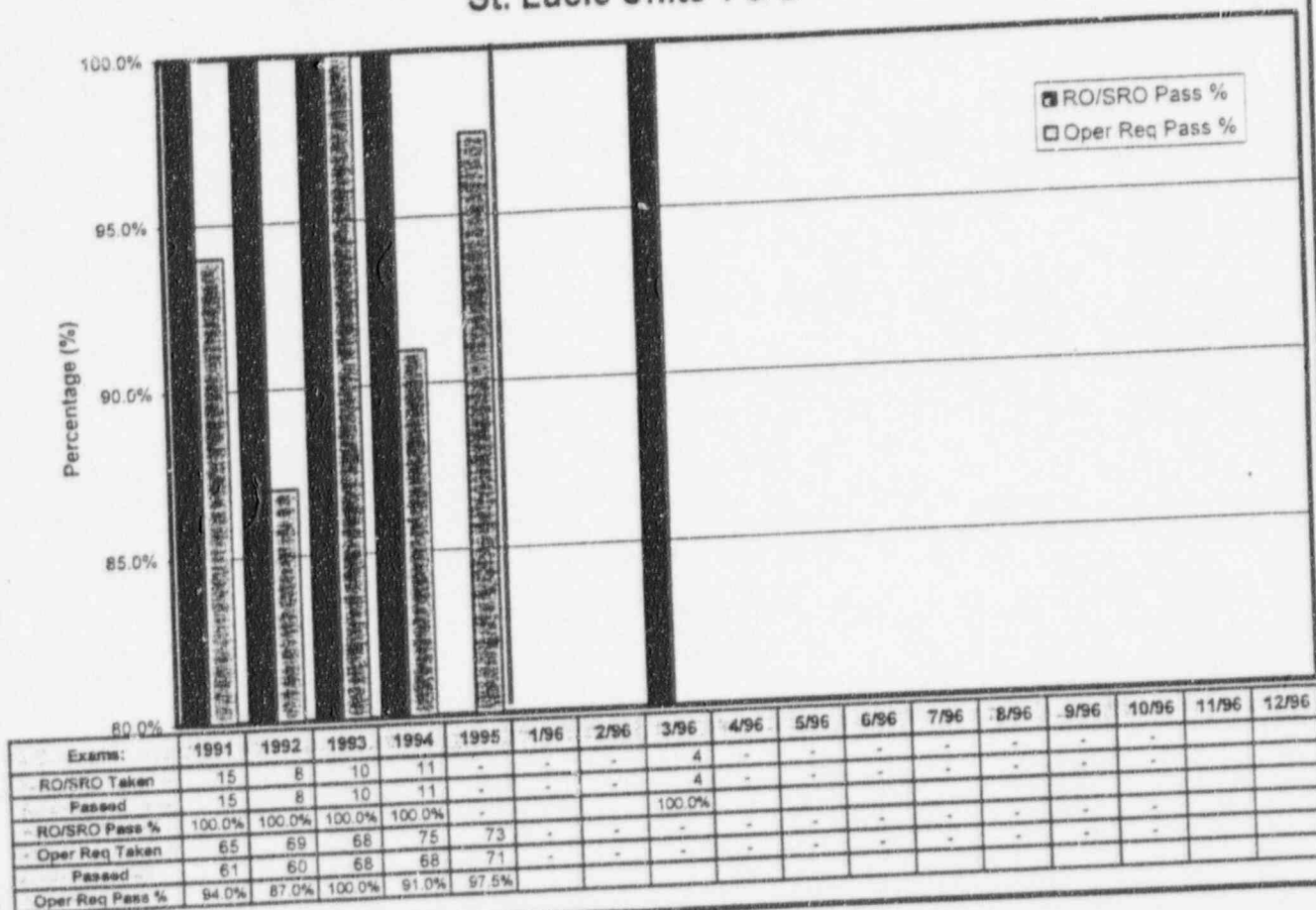
Initial NRC Exams		NRC Requal Exams	
RO's Pass Rate	94.6%	RO's Pass Rate	91.0%
Instant SRO's Pass Rate	94.4%	SRO's Pass Rate	85.0%
Upgrade SRO's Pass Rate	94.7%	Average Overall	88.0%
Average Overall	94.6%		

### PERFORMANCE SUMMARY

No exams were given at Turkey Point in October.



# **OPERATOR EXAMINATION PERFORMANCE** **St. Lucie Units 1 & 2**



Exams:	1991	1992	1993	1994	1995	1/96	2/96	3/96	4/96	5/96	6/96	7/96	8/96	9/96	10/96	11/96	12/96
RO/SRO Taken	15	8	10	11	-	-	-	4	-	-	-	-	-	-	-	-	-
Passed	15	8	10	11	-	-	-	4	-	-	-	-	-	-	-	-	-
RO/SRO Pass %	100.0%	100.0%	100.0%	100.0%	-	-	-	100.0%	-	-	-	-	-	-	-	-	-
Oper Req Taken	65	69	68	75	71	-	-	-	-	-	-	-	-	-	-	-	-
Passed	61	60	68	68	71	-	-	-	-	-	-	-	-	-	-	-	-
Oper Req Pass %	94.0%	87.0%	100.0%	91.0%	97.5%	-	-	-	-	-	-	-	-	-	-	-	-

## **DEFINITION**

Initial License Examination (RO/SRO) results are reported for all candidates taking an Initial License Exam as conducted by the NRC.

Operator Regualification Examination results are reported for both RO's and SRO's. This examination is administered annually by the utility and may be jointly administered by the NRC. Retests of operators who failed examinations are not included.

## **STATISTICAL SUMMARY**

Initial RO/SRO License Exams			
	No. Taken	No. Passed	Pass Rate %
YTD 1996	4	4	100.0%
YTD 1995	0	0	0.0%

Operator Regual Exams			
	No. Taken	No. Passed	Pass Rate %
YTD 1996	0	0	0.0%
YTD 1995	73	71	97.5%

## **INDUSTRY PERFORMANCE**

The NRC at their last Regional Training Managers Meeting (for fiscal year 1994) provided the following data:

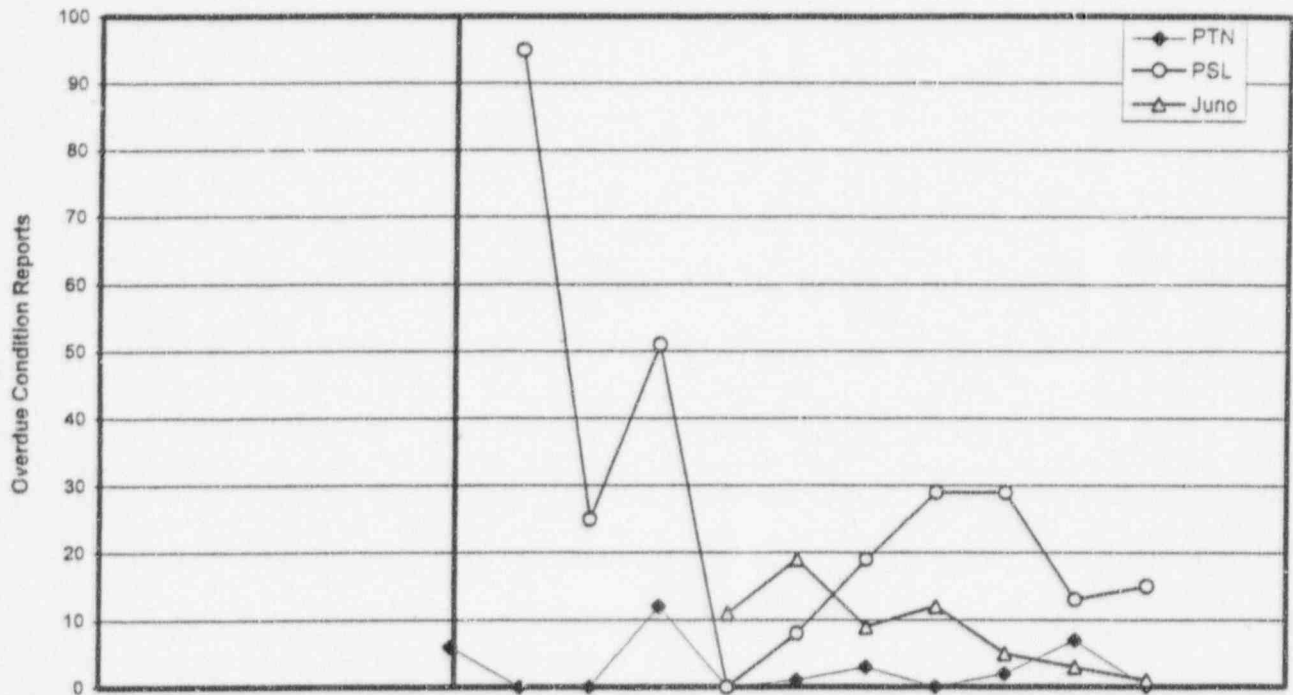
Initial NRC Exams		NRC Regual Exams	
RO's Pass Rate	94.6%	RO's Pass Rate	91.0%
Instant SRO's Pass Rate	94.4%	SRO's Pass Rate	85.0%
Upgrade SRO's Pass Rate	94.7%	Average Overall	88.0%
Average Overall	94.6%		

## **PERFORMANCE SUMMARY**

No exams were given in October at St. Lucie.

Data Provider: Mike Allen (PSL) 467-4161

## OVERDUE CONDITION REPORTS



Unit	1991	1992	1993	1994	1995	1/96	2/96	3/96	4/96	5/96	6/96	7/96	8/96	9/96	10/96	11/96	12/96
PTN					6	0	0	12	0	1	3	0	2	7	0		
PSL						95	25	51	0	8	19	29	29	13	15		
Juno									11	19	9	12	5	3	1		

### DEFINITION

This graph shows Condition Reports that exceed assigned priority timeframe. This indicator takes a snapshot the last day of each month. Record the number of overdue condition reports the last day of each month.

Severity Levels are as follows: A = 3 working days B = 10 calendar days C = 30 calendar days

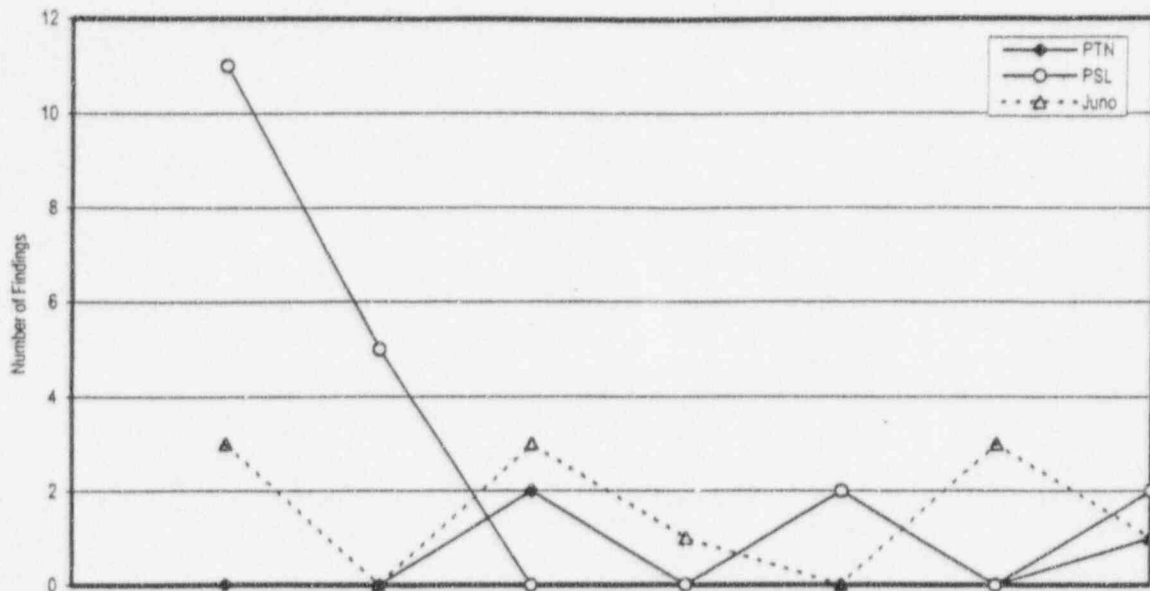
### STATISTICAL SUMMARY

### INDUSTRY PERFORMANCE

### PERFORMANCE SUMMARY

Data Providers: (PTN) Ed Lyons 246-6967, (PSL) J. Voorhees 467-7587, and Bob Symes (Juno) 694-4287

## QA FINDINGS



Unit	0-30 days	31-60 days	61-90 days	91-120 days	121-150 days	151-180 days	> 180 days
PTN	0	0	2	0	0	0	1
PSL	11	5	0	0	2	0	2
Juno	3	0	3	1	0	3	1

### DEFINITION

The indicator represents the age of audit findings open as of the last day of the month. The clock starts on the day of the audit report transmittal and continues until Quality Assurance (QA) verifies that implementation has occurred.

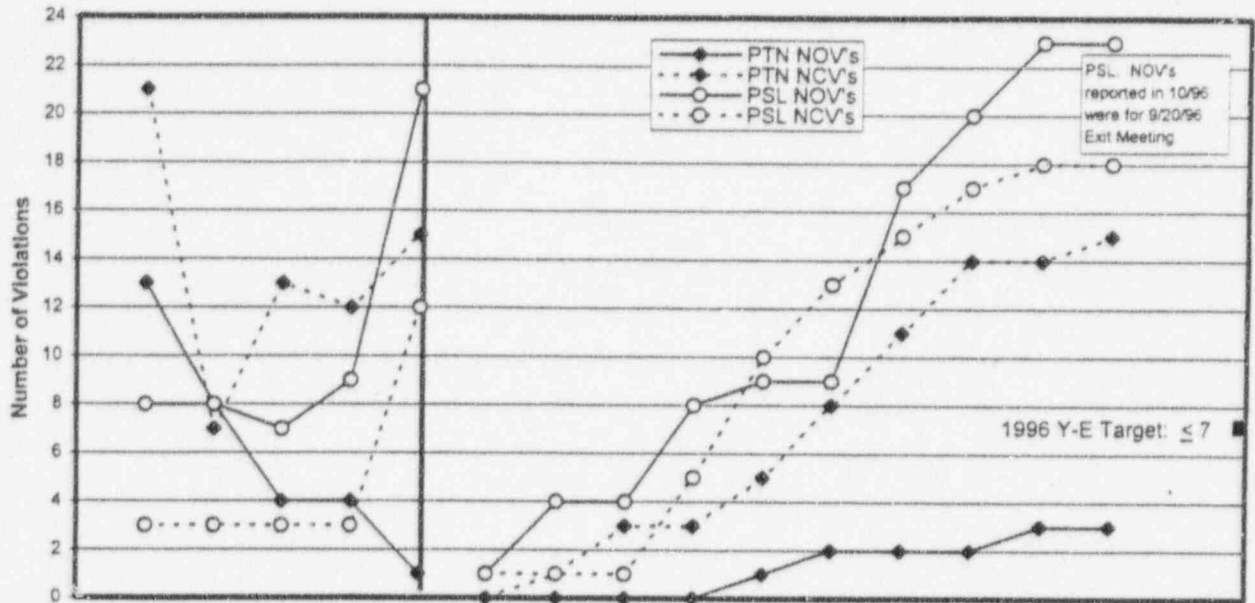
### STATISTICAL SUMMARY

### INDUSTRY PERFORMANCE

### PERFORMANCE SUMMARY

## NRC VIOLATIONS Cited and Non-Cited

(Year-to-Date)



Unit	1991	1992	1993	1994	1995	1/96	2/96	3/96	4/96	5/96	6/96	7/96	8/96	9/96	10/96	11/96	12/96
PTN NOV's	13	8	4	4	1	0	0	0	0	1	2	2	2	3	3		
PTN NCV's	21	7	13	12	15	0	1	3	3	5	8	11	14	14	15		
PSL NOV's	8	8	7	9	21	1	4	4	8	9	9	17	20	23	23		
PSL NCV's	3	3	3	3	12	1	1	1	5	10	13	15	17	18	18		

### DEFINITION

Notice of Violations (NOV's) are categorized in terms of five levels of severity to show their relative importance. Severity Levels I and II are violations that involve actual or high potential impact on the public. Severity Level III Violations are cause for significant concern. Severity Level IV violations are less serious, but are of more than minor concern; i.e., if left uncorrected, could lead to a more serious concern. Violations are counted on the date of the inspection exit meeting. Violations are now counted with respect to the date of occurrence (using the date of the inspection exit meeting) instead of the date of the inspection report, as was done in the past.

Non-cited Violations (NCV's) are violations of NRC regulations for which no citation will be issued.

#### STATISTICAL SUMMARY

	Oct	YTD	Year End Target
PTN Cited	0	3	7
PTN Non-Cited	0	15	
PSL Cited	0	23	7
PSL Non-Cited	0	18	

#### INDUSTRY PERFORMANCE

<u>NRC Violations</u>	
1995 IBG Group Mean	9.4
1995 Region II Mean	10.5
1995 IBG Top Quartile Mean	3.8
1995 IBG Top Quartile Entry	6.0
1995 Region II Top Quartile Entry	7.0

### PERFORMANCE SUMMARY

Turkey Point reported one Notice of Violation (NOV) in October for September. Year-to-date, three NOV's have been reported.

St. Lucie reported three NOV's in October for September; eleven potential NOV's are pending investigation. Year-to-date, the total confirmed is 23.

See next page for details.

Data Providers: (PTN) Gary Hollinger 246-6078 and (PSL) Ed Weinkam 467-7162

## NOTICE OF VIOLATIONS - 1996

### TURKEY POINT:

- 1 #96-06-02 Preconditioning of EDG fuel system prior to surveillance test. Exit Mtg date: 6/14/96.
- 2 #96-04-02 Failure to follow Chemical Volume Control System Operating Procedure during a Unit 4 blender flushing evolution. Exit Mtg date: 5/15/96.
- 3 #96-11-03 Fire Zones 64 and 143 not covered by exemption to Appendix R. Exit Mtg date: 9/27/96.

### ST. LUCIE:

- 1 #96-01-01 Temporary Changes to Procedures Improperly Changed Intent of Procedures. Exit Mtg Date: 1/26/96.
- 2 #96-03-01 Operators failed to follow procedures with four examples. Exit Mtg Date: 2/8/96.
- 3 #96-03-02 Inadequate design control of RCS boron dilution procedure. Exit Mtg Date: 2/8/96.
- 4 #96-03-03 A 10 CFR 50.59 safety evaluation was inadequate. Exit Mtg Date: 2/8/96.
- 5 #96-04-01 A containment gaseous/particulate/iodine monitor was rendered inoperable due to a failure to follow procedures, combined with a lack of proper follow through on the part of non-licensed operators taking logs. Exit Mtg Date: 4/3/96.
- 6 #96-04-02 Failures to make required log entries for reactivity manipulations and a main generator hydrogen addition. Exit Mtg Date: 4/3/96.
- 7 #96-04-03 An EDG was rendered inoperable due to a failure to follow procedures while placing the fuel oil tank on recirculation. Exit Mtg Date: 4/3/96.
- 8 #96-04-04 Reviews of historical data for CEA maintenance revealed that post-modification testing acceptance criteria for Unit 1 CEA power cables were not applied to post-modification test data. Exit Mtg Date: 4/3/96.
- 9 #96-06-05 An audit of fire brigade member qualifications resulted in a violation for failure to maintain current physicals. Exit Mtg date: 5/17/96.
- 10 #96-09-01 Failure to control overtime. Exit Mtg date: 7/8/96.
- 11 #96-09-02 Inadequate procedure for managing overtime. Exit Mtg date: 7/8/96.
- 12 #96-09-03 Failure to test the Reactor Cavity Pressure Relief Dampers in Accordance with the Vendor's Technical Manual. Exit Mtg date: 7/8/96.
- 13 #96-09-04 Failure to perform adequate testing on the Reactor Cavity Pressure Relief Dampers. Exit Mtg date: 7/8/96.
- 14 #96-09-06 Failure to control contaminated tools in accordance with licensee procedures. Exit Mtg date: 7/8/96.
- 15 #96-11-04 Preconditioning of valves prior to surveillance. Exit Mtg date: 8/6/96.
- 16 #96-12-02 Failure to perform safety evaluation as required by 10CFR 50.59 for failure to recognize an unreviewed safety question in the development of a safety evaluation for an EDG fuel oil transfer line valve lineup change (Level III). Exit Mtg date: 7/12/96.
- 17 #96-12-03 Failure to coordinate design changes with the necessary changes to operating procedures as evidenced by three examples. Exit Mtg date: 7/12/96.
- 18 #96-12-04 Failure to assure that the design of the Circulating and Intake Cooling Water System was correctly translated into plant drawings. Exit Mtg date: 7/12/96.
- 19 #96-16-01 Failure to make a 1 hour security event notification to the NRC in accordance with 10 CFR 73.71 and plant procedure 0006125 for the July 26, 1996, tampering event. Exit Mtg date: 8/23/96.
- 20 #96-16-04 Failure to follow procedures for key control for the Unit 1 PORV key lock switches. Exit Mtg date: 8/23/96.
- 21 #96-13-01 Failure to include some SSC's within the scope of the Rule in the Plant's Maintenance Rule Program (PASS, Communications, Service Air, Radiation Monitors). Exit Mtg date: 9/20/96.
- 22 #96-13-02 Failure to follow procedures in actions required by ADM 17.08 (multiple examples). Exit Mtg date: 9/20/96.
- 23 #96-13-03 Improper use of MPFFs as indicator of reliability for safety significant systems. Exit Mtg date: 9/20/96.

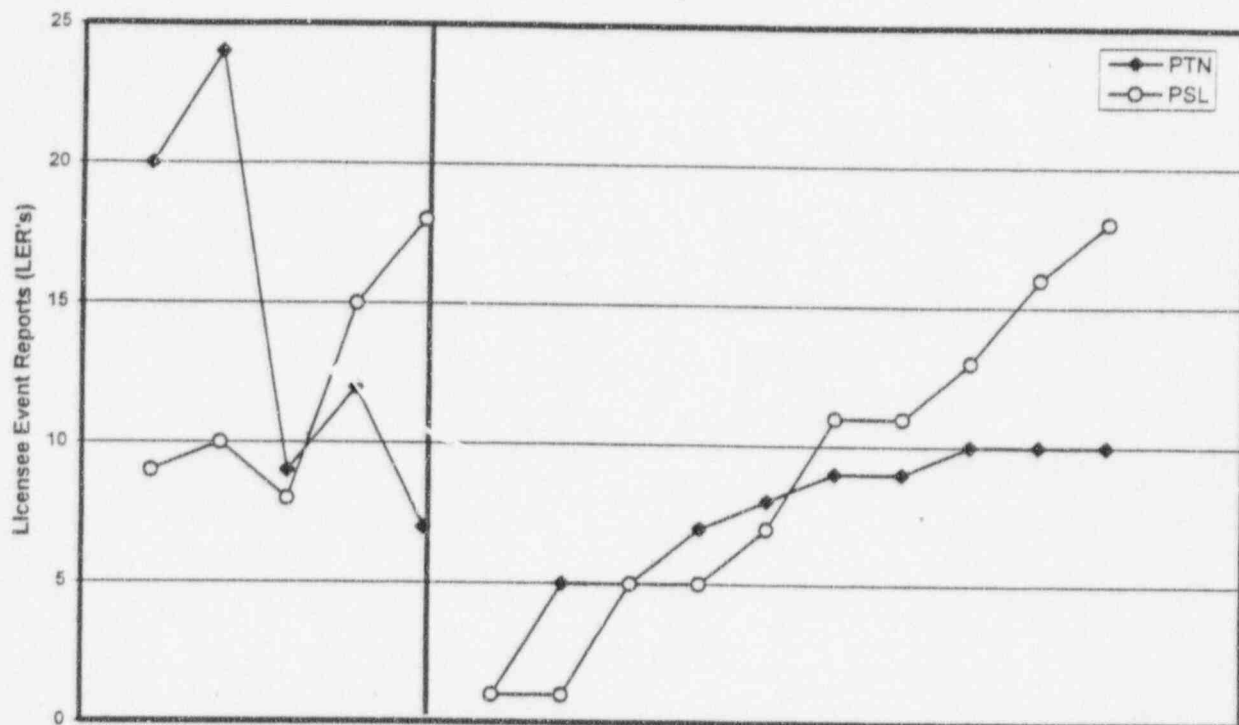
Potential Notice of Violations at St. Lucie for October were reported as follows:

- 1 #96-15-04 Failure to follow procedure for the use of M&TE during NI system work. Exit Mtg date: 10/12/96.
- 2 #96-15-08 Failure to use or follow procedure resulting in the operability of the containment PIG radiation monitor (repeat violation). Exit Mtg date: 10/12/96.
- 3 #96-17-01 Violation of 10CFR Part 50, Appendix B related to implementation of PC/M's for nuclear instrumentation and BEACON (3 examples). Exit Mtg date: 10/13/96.
- 4 #96-17-02 Failure of I&C personnel to write a Condition Report to address a discrepant condition related to implementation of a nuclear instrumentation of a nuclear instrumentation replacement PC/M. Exit Mtg date: 10/25/96.
- 5 #96-18-01 Failure to maintain adequate provisions to implement staff augmentation measures for the emergency response organization. Exit Mtg date: 10/28/96.
- 6 #96-18-02 Failure to meet 10 CFR Appendix E in providing adequate training to identified members of the emergency response organization. Exit Mtg date: 10/28/96.
- 7 #96-18-03 Lack of instruction in EIPs for relocating to an alternate OSC, as identified in the E Plan. Exit Mtg date: 10/28/96.
- 8 #96-18-04 Failure to effect timely corrective actions for Hurricane Erin. Exit Mtg date: 10/28/96.
- 9 #96-19-01 Unauthorized access to the protected area and potential access to vital areas by a terminated employee. Exit Mtg date: 10/25/96.
- 10 #96-19-02 Events resulting in unauthorized access to the plant were not reported in accordance with 10 CFR 73.71/10 CFR Part 73 Appendix G. Exit Mtg date: 10/25/96.
- 11 #96-19-03 Procedures were inadequate in that unauthorized individuals may have gained access to the plant. Exit Mtg date: 10/25/96.



# LICENSEE EVENT REPORTS

(Year-to-Date)



Unit	1991	1992	1993	1994	1995	1/96	2/96	3/96	4/96	5/96	6/96	7/96	8/96	9/96	10/96	11/96	12/96
PTN	20	24	9	12	7	1	5	5	7	8	9	9	10	10	10		
PSL	9	10	8	15	18	1	1	5	5	7	11	11	13	16	18		

## DEFINITION

License Event Reports (LER) are submitted to the NRC by the licensee to report unusual occurrences prescribed by 10CFR50.73.

## STATISTICAL SUMMARY

	Oct	YTD
PTN	0	10
PSL	2	18

## INDUSTRY PERFORMANCE

## PERFORMANCE SUMMARY

Data Providers: (PTN) Gary Hollinger 246-6076 and (PSL) Ed Weinkam 467-7162

## INPO ASSESSMENT RATINGS



Unit	1986	1987	1988	1989	1990	1991	1992	1993	1994	1995	1996
PTN	4.00	4.00	3.00	4.00	3.00	-	2.00	2.00	-	1.00	-
PSL	-	-	2.00	-	1.00	-	1.00	-	-	1.00	-

### INPO ASSESSMENT PROGRAM DESCRIPTION

The Institute of Nuclear Power Operations (INPO) conducts periodic evaluations of site activities to make an overall determination of plant safety, to evaluate management systems and controls, and to identify areas needing improvement. Information is assembled from discussions, interviews, observations, and reviews of documentation.

**Evaluation Frequency:** INPO's goal is to visit each plant on an average of every 18 months. However, this frequency may vary depending upon the previous assessment ratings. For instance, if a plant is rated a "1" or "2", the interval between assessments is usually 20-24 months; if a plant is rated a "4" or a "5", the assessment interval is  $\leq 18$  months.

#### Performance Category Ratings:

- Category 1: Overall performance is excellent. Industry standards of excellence are met in most areas. No significant weaknesses noted.
- Category 2: Overall performance is exemplary. Industry standards of excellence are met in most areas. No significant weaknesses noted.
- Category 3: Overall performance is generally in keeping with the high standards required in nuclear power. However, improvements are needed in a number of areas. A few significant weaknesses may exist.
- Category 4: Overall performance is acceptable, but improvements are needed in a wide range of areas. Significant weaknesses are noted in several areas.
- Category 5: Overall performance does not meet the industry standard of acceptable performance. The margin of nuclear safety is measurably reduced. Strong and immediate management action to correct deficiencies is required. Special attention, assistance, and follow-up are required.

**NOTE:** If a plant is found to be operating without an adequate margin of nuclear safety, INPO will request that the plant be shutdown, or not started up.

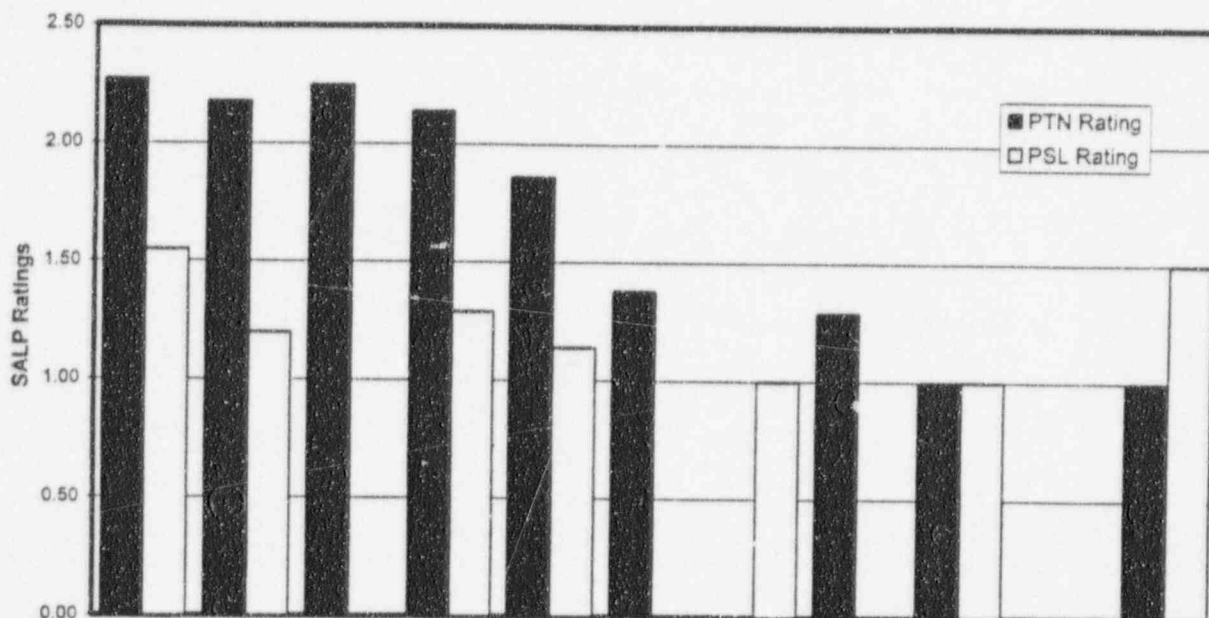
### PERFORMANCE SUMMARY

Turkey Point and St. Lucie received an INPO category rating of "1" in 1995.

The next evaluations for Turkey Point and St. Lucie are expected in the last quarter of 1996 and first quarter of 1997, respectively.

Data Providers: (PTN) Gary Hollinger 246-6078 and (PSL) Ed Weinkam 467-7162

## NRC SALP CATEGORY RATINGS



	1986	1987	1988	1989	1990	1991	1992	1993	1994	1995	1996
PTN Rating	2.27	2.18	2.25	2.14	1.86	1.38	-	1.29	1.00	-	1.00
Period Ending	4/30/86	5/31/87	6/30/88	7/31/89	7/31/90	9/28/91	-	1/30/93	8/27/94	-	8/17/96
PSL Rating	1.55	1.20	-	1.29	1.14	-	1.00	-	1.00	-	1.50
Period Ending	4/30/86	10/31/87	-	4/30/89	10/31/90	-	5/2/92	-	1/1/94	-	1/6/96

### SALP PROGRAM DESCRIPTION

It is the policy of the NRC to use the Systematic Assessment of Licensee Performance (SALP) process to articulate the agency's observations and insights on licensee safety performance. The SALP report communicates those observations and insights.

**Evaluation Frequency:** The NRC will normally review and evaluate each power reactor licensee that possesses an operating license at least every 18 months. When the NRC determines that the performance warrants a more frequent evaluation, the normal SALP frequency may be increased. The assessment period may be extended to a maximum of 24 months when a plant receives a Category 1 rating in all four functional areas.

**Functional Areas:** Performance is generally evaluated in four (4) functional areas:

- Plant Operations.** This functional area consists chiefly of the control and execution of activities directly related to operating a plant. It includes activities such as plant startup, power operations, plant shutdown, and system lineups. It also includes initial and requalification training of licensed operators.
- Maintenance.** This functional area includes all activities associated with either diagnostic, predictive, preventive, or corrective maintenance of plant structures, systems, and components, or maintenance of the physical condition of the plant.
- Engineering.** This functional area addresses the adequacy of technical and engineering support for all plant activities. It includes all licensee activities associated with design control; the design, installation, and testing of plant modifications; engineering and technical support for operations, outages, maintenance, testing, surveillance, and procurement activities; configuration management; design-basis information and its retrieval; and support for licensing activities.
- Plant Support.** This functional area covers all activities related to plant support functions, including radiological controls, emergency preparedness, security, chemistry, and fire protection. Housekeeping controls are also included in this area.

**Performance Category Ratings:** Licensee performance in each functional area is assessed by assigning a category rating as discussed below:

Category 1. Licensee attention and involvement have been properly focused on safety and resulted in a superior level of safety performance.

Category 2. Licensee attention and involvement are normally well focused and resulted in a good level of safety performance.

Category 3. Licensee attention and involvement have resulted in an acceptable level of performance. However, performance may exhibit one or more of the following characteristics: ineffective programs and significant issues, lack of corrective action thoroughness, and deficiencies in root cause analysis. Because the margin to unacceptable performance in important aspects is small, increased NRC and licensee attention is required.

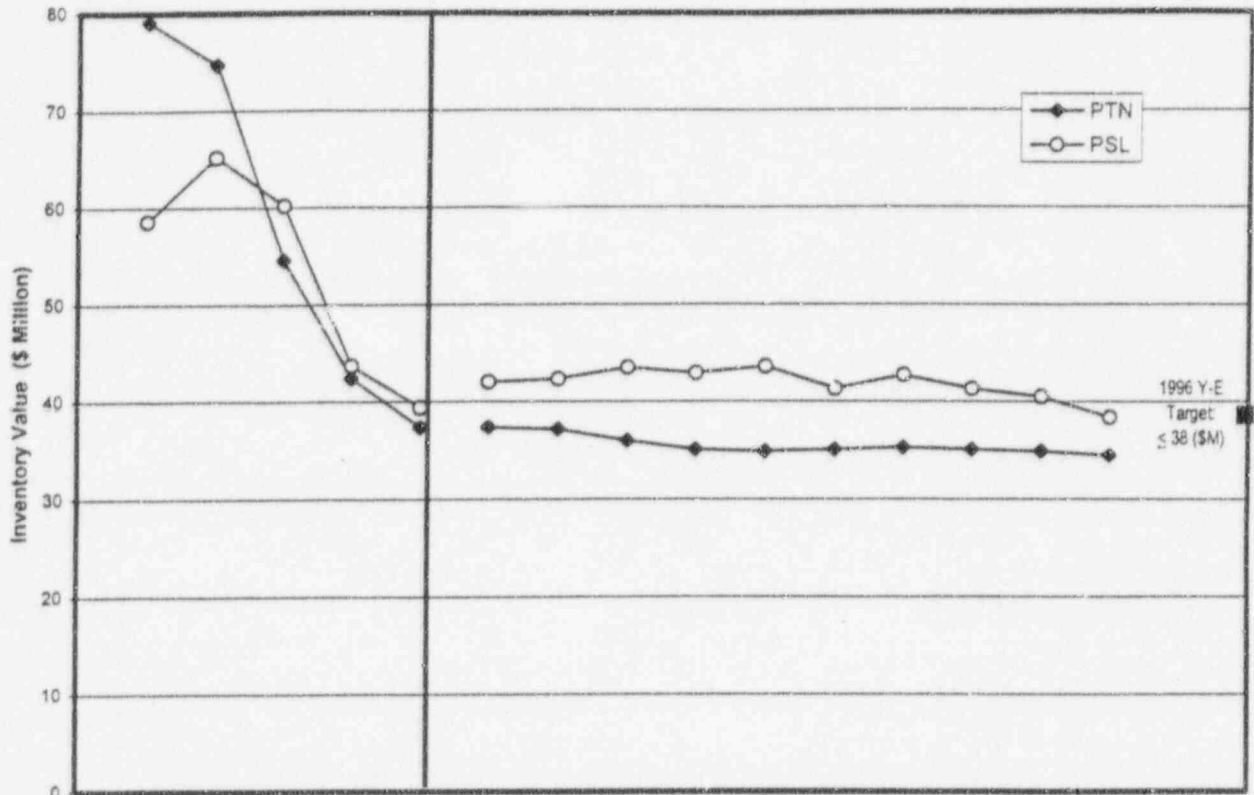
### PERFORMANCE SUMMARY

The Nuclear Regulatory Commission issued its Systematic Assessment of Licensee Performance (SALP) report for the Turkey Point plant for August 28, 1994 through August 17, 1996. For the second consecutive time, the plant received Category 1 (superior) ratings in all four functional areas rated by the NRC.

Functional Area:	Turkey Point		St. Lucie	
	Prior	Most Recent	Prior	Most Recent
Plant Operations	1	1	1	2
Maintenance	1	1	1	2
Engineering	1	1	1	1
Plant Support	1	1	1	1
Overall:	1.00	1.00	1.00	1.50

Data Providers: (PTN) Gary Hollinger 246-6078 and (PSL) Ed Weinkam 467-7162

## INVENTORY VALUE ACCOUNT 154.300



Unit	1991	1992	1993	1994	1995	1/96	2/96	3/96	4/96	5/96	6/96	7/96	8/96	9/96	10/96	11/96	12/96
PTN	79.1	74.7	54.6	42.4	37.4	37.4	37.2	36.0	35.1	34.9	35.1	35.3	35.1	34.9	34.5		
PSL	58.6	65.2	60.2	43.7	39.4	42.0	42.3	43.5	42.9	43.6	41.3	42.7	41.3	40.5	38.4		

### DEFINITION

This indicator reflects the value of Account 154.300. This account reflects materials needed to keep operational the physical equipment and facilities of the plant (e.g., spare parts, consumables, commodities, tools). The information is pulled from SAR Report #G0009R72-501. The PassPort system utilizes SAR for system data reporting.

### STATISTICAL SUMMARY

		Start	End	% Change	YE Target
PTN	Monthly	34.9	34.5	-1.1%	
	Y-T-D	37.4	34.5	-7.8%	≤38M
PSL	Monthly	40.5	38.4	-5.2%	
	Y-T-D	39.4	38.4	-2.5%	≤38M

### INDUSTRY PERFORMANCE

IBG (Year 1995)	*Total
Average	\$39.2 Million
Top Quartile Entry	\$35.0 Million
Top Quartile Average	\$29.0 Million

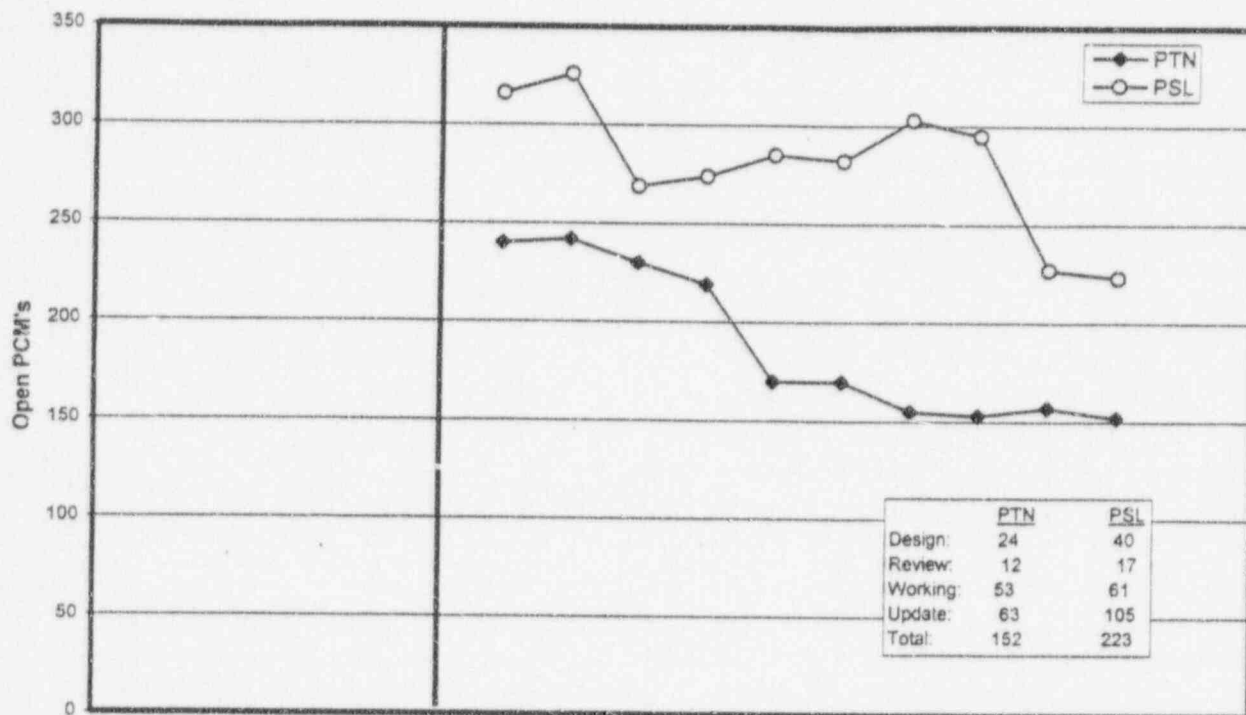
\*Value does not include Capital as defined by PRUC

### PERFORMANCE SUMMARY

- Turkey Point Regular Inventory decreased by \$0.4M in October.
- St. Lucie's Regular inventory decreased by \$2.1M for the month.

Data Providers: (PTN) Dick Rose 246-6692 and (PSL) Gil Boissy 467-7582

## OPEN PLANT CHANGES / MODIFICATIONS (PC/Ms)



Unit	1991	1992	1993	1994	1995	1/96	2/96	3/96	4/96	5/96	6/96	7/96	8/96	9/96	10/96	11/96	12/96
PTN						240	242	230	219	170	170	155	153	157	152		
PSL						316	326	269	274	285	282	303	295	227	223		

### DEFINITION

This indicator tracks the total number of PC/M's in the modification cycle. The purpose is to provide management a snapshot of the number of PC/M's. Include all PC/M's in design (not issued to the plants) review (issued to the plants for review), and implementation or drawing update phases. The PC/M is considered open until drawing update is complete.

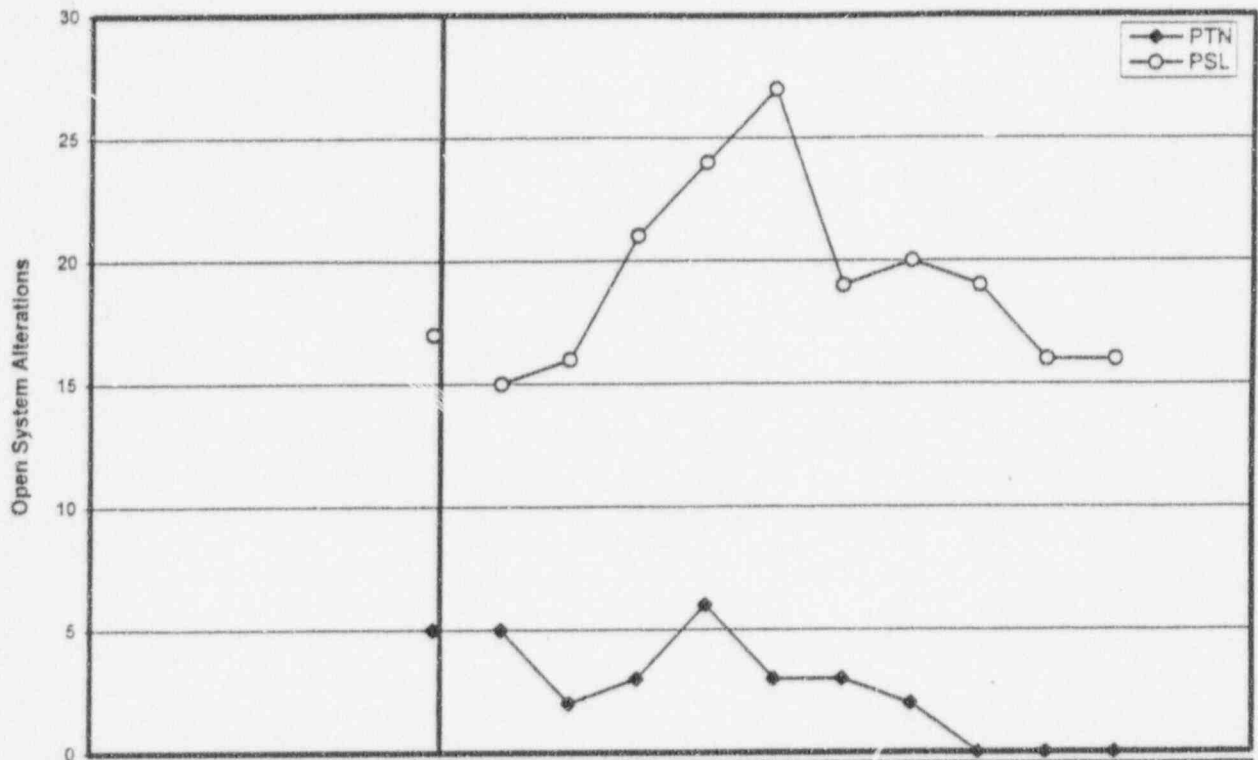
### STATISTICAL SUMMARY

### INDUSTRY PERFORMANCE

### PERFORMANCE SUMMARY



## OPEN TEMPORARY SYSTEM ALTERATIONS



Unit	1991	1992	1993	1994	1995	1/96	2/96	3/96	4/96	5/96	6/96	7/96	8/96	9/96	10/96	11/96	12/96
PTN					5	5	2	3	6	3	3	2	0	0	0		
PSL					17	15	16	21	24	27	19	20	19	16	16		

### DEFINITION

A temporary system alteration is a modification made to plant equipment, components, or systems that does not conform with approved drawings or other design documents; a modification that is necessary for continued safe plant operation; a modification that will remove a nuisance or distraction to the Plant Operators; a modification necessary to enable the plant to start up in a safe manner.

### STATISTICAL SUMMARY

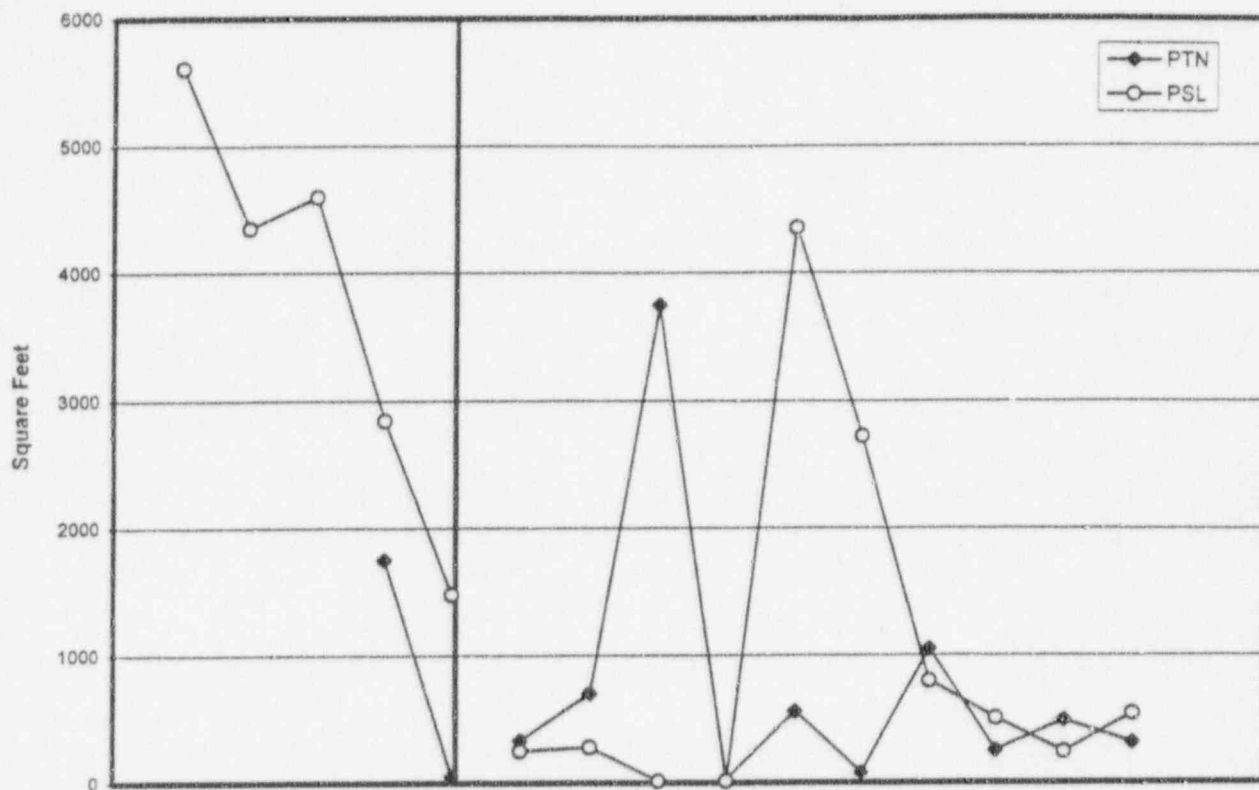
### INDUSTRY PERFORMANCE

	<u>Oct</u>	<u>Year End Goal</u>
PTN	0	5
PSL	16	10

### PERFORMANCE SUMMARY

Data Providers: (PTN) Julio Balaguero 246-6971 and (PSL) Kris Mohindroo 467-7482

## CONTAMINATED FLOOR SPACE



Unit	1991	1992	1993	1994	1995	1/96	2/96	3/96	4/96	5/96	6/96	7/96	8/96	9/96	10/96	11/96	12/96
PTN				1750	50	330	700	3748	32	559	77	1048	250	489	312		
PSL	5610	4351	4600	2848	1480	250	280	12	12	4354	2725	800	505	242	540		

### DEFINITION

This indicator, designed to measure contaminated floor space with removable activity  $\geq 1000$  dpm/100cm sq. beta/gamma or  $\geq 20$  dpm/100cm sq. alpha, is counted against the base. Areas that can be specifically exempted from the base include: reactor containment building, chemical volume control system demineralizer room and long term process areas such as the decontamination facility. Contaminated components such as charging pumps, evaporators, etc. are not included as part of "recoverable" floor space (i.e. not considered floor area you can walk or step on).

PTN: Total Base(117,746 sq. ft.) Exempted Area (6,110 sq. ft.)

PSL: Total Base(112,422 sq. ft.) Exempted Area (7,722 sq. ft.)

### STATISTICAL SUMMARY

### INDUSTRY PERFORMANCE

	<u>Oct</u>	<u>Target</u>
PTN	312	50
PSL	540	50

### PERFORMANCE SUMMARY

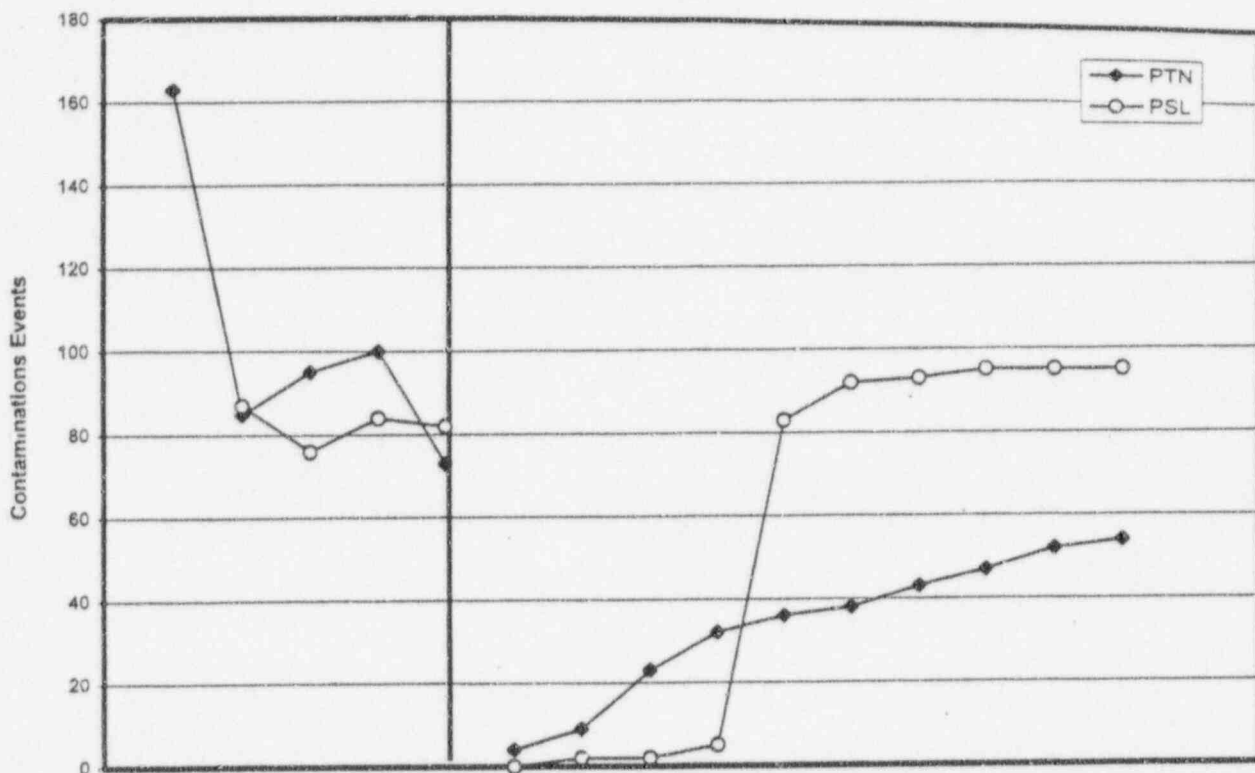
Turkey Point and St. Lucie's Contaminated Floor Space was higher than the year-end target.

Data Providers: (PTN) John Lindsay 246-6548 and (PSL) Hank Buchanan 467-7300



# PERSONNEL CONTAMINATION EVENTS

(Year-to-Date)



Unit	1991	1992	1993	1994	1995	1/96	2/96	3/96	4/96	5/96	6/96	7/96	8/96	9/96	10/96	11/96	12/96
PTN	163	85	95	100	73	4	9	23	32	36	38	43	47	52	54		
PSL		87	76	84	82	0	2	2	5	83	92	93	95	95	95		

## DEFINITION

This indicator is designed to monitor personnel contamination. A personnel contamination exists when 5000 dpm per 100cm<sup>2</sup> on skin or personal clothing as detected by Personal Contamination Monitor and  $\geq 100$  counts per minute (net) using the Frisker is observed.

## STATISTICAL SUMMARY

	<u>Oct</u>	<u>Y-T-D</u>
PTN	2	54
PSL	0	95

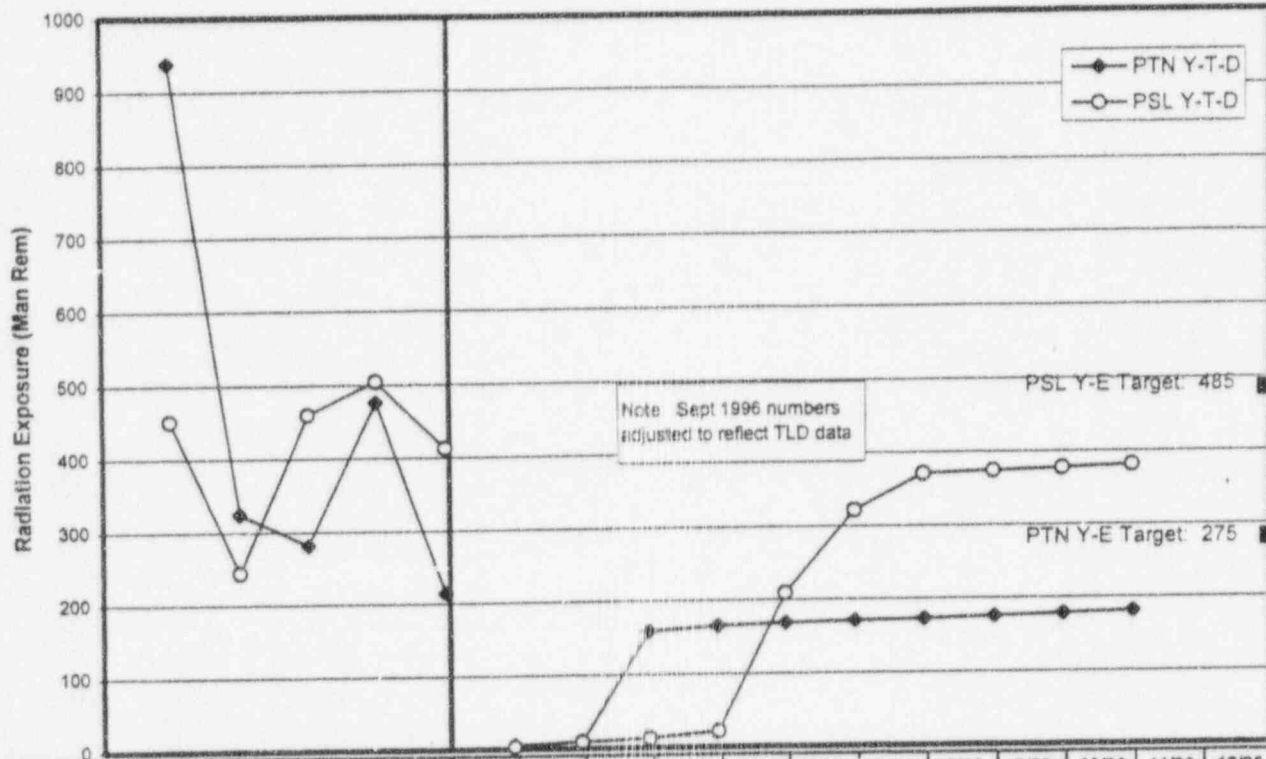
## INDUSTRY PERFORMANCE

## PERFORMANCE SUMMARY

Data Providers: (PTN) John Lindsay 246-6548 and (PSL) Hank Buchanan 467-7300

# RADIATION EXPOSURE

(Year-to-Date)



Unit	1991	1992	1993	1994	1995	1/96	2/96	3/96	4/96	5/96	6/96	7/96	8/96	9/96	10/96	11/96	12/96
PTN Y-T-D	938.0	324.9	281.8	476.2	214.6	5.3	9.5	159.2	165.5	168.7	171.0	172.8	175.4	178.6	181.4		
PSL Y-T-D	451.3	244.5	459.9	504.7	412.8	3.0	9.0	13.0	21.9	209.2	321.9	371.3	374.6	377.4	381.5		

## DEFINITION

Collective Radiation Exposure is the total effective dose equivalent received by all on-site personnel (including contractors and visitors), it includes external deep dose as measured by the thermoluminescent dosimeters (TLD's) plus internal dose. It is reported in man-rem for the station. Current month readings may be taken from the direct reading dosimeters (DRD's).

### STATISTICAL SUMMARY

	Oct	YTD	YTD Target
PTN	3.4*	181.4	265.0
PSL	4.0*	381.5	468.0
Year-End Targets:			
PTN: 275.0			
PSL: 485.0			

### INDUSTRY PERFORMANCE

	Man-Rem (two units)
WANO (PWR's)	
1995 Median	306
1995 Goal	370

### PERFORMANCE SUMMARY

- Turkey Point and St. Lucie's Y-T-D Collective Radiation Exposure totals were below Y-T-D targets.

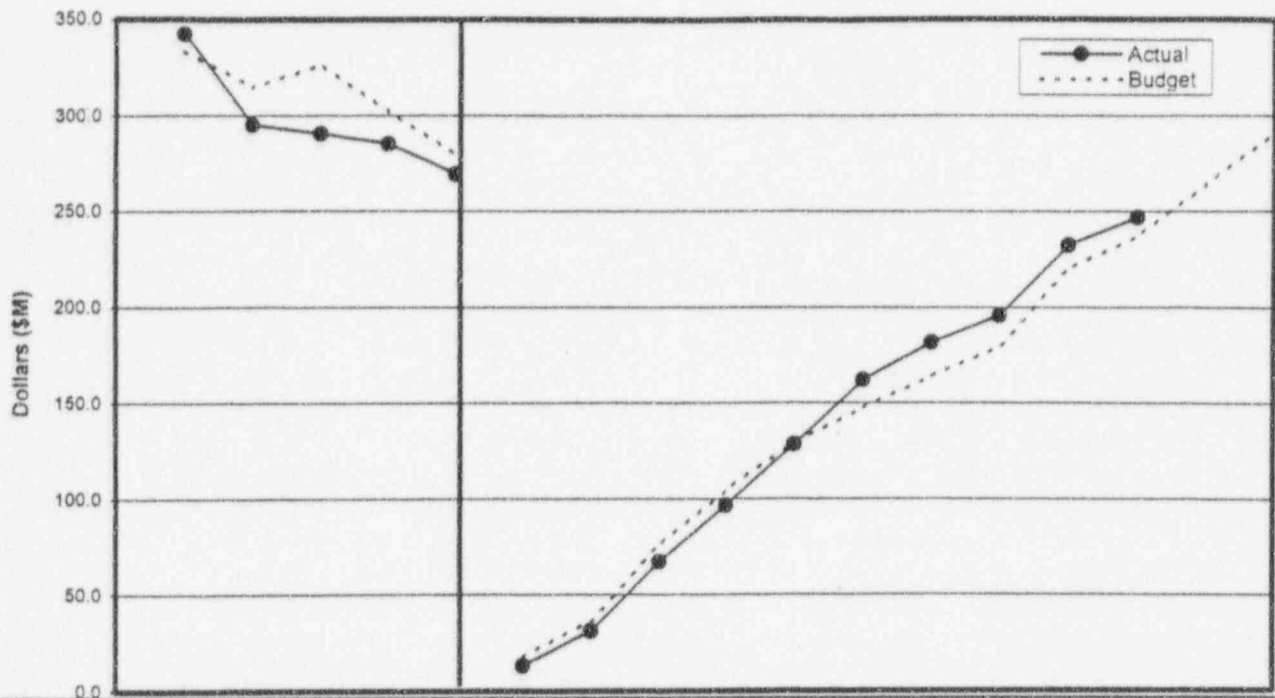
Data Providers: (PTN) John Lindsay 246-6548 and (PSL) Hank Buchanan 467-7300

\* Includes DRD count



## O&M BUDGET - DIVISION

(Year-to-Date)



	1991	1992	1993	1994	1995	1/96	2/96	3/96	4/96	5/96	6/96	7/96	8/96	9/96	10/96	11/96	12/96
Actual	342.4	295.2	290.6	285.2	269.5	13.4	31.2	67.4	96.7	128.5	162.2	181.9	195.8	232.5	246.9		
Budget	333.3	314.6	326.4	302.0	279.2	17.8	36.7	75.8	104.8	129.6	147.4	164.0	179.6	220.1	237.0	264.0	291.1
Variance (%)	2.7	-6.2	-11.0	-5.6	-3.5	-24.9	-14.9	-11.1	-7.8	-0.9	10.1	11.1	9.0	5.7	4.2		

### DEFINITION

Operating and Maintenance Expenditures include Nuclear Division operation and maintenance expenses associated with direct employees, contractors and consultants, equipment, tools, design, engineering and other items/activities required to sustain the electrical generation of the plants and to provide required support. Fuel costs, corporate administrative and general expenses, and charges from other departments outside the Nuclear Division are excluded.

$$\frac{\text{Y-T-D Actual Expenses} - \text{Y-T-D Budgeted Expenses}}{\text{Y-T-D Budgeted Expenses}} \times 100\% = \text{O\&M Variance \%}$$

### STATISTICAL SUMMARY

	Oct Y-T-D Actual (\$M)	Y-T-D Variance (\$M)	%
1996	246.9 (\$M)	9.9 (\$M)	4.2%
1995	211.3 (\$M)	-16.1 (\$M)	-7.1%
1996 Y-E Budget: 291.1 (\$M)			

### INDUSTRY PERFORMANCE

	O&M Budget
IBG Actual 1995 Average	\$334.1M
IBG Top Quartile Entry	\$277.6M
IBG Top Quartile Average	\$234.2M

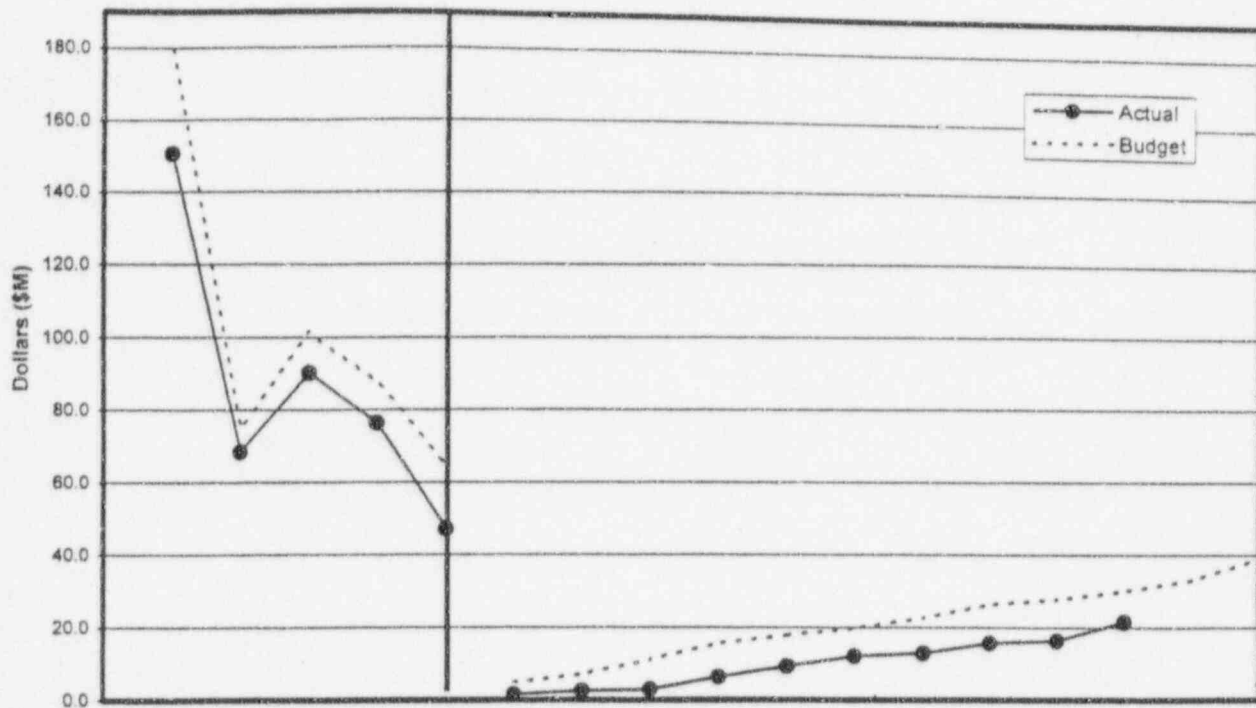
### PERFORMANCE SUMMARY

O&M Expenditures through October 1996 were \$246.9 million which represented a budget overrun of \$9.9 million (or 4.2%). Significant contributors to the variance include: St. Lucie Unit 1 outage scope changes partially offset by transfer of the Turkey Point Termal Uprate Project from O&M and by project and material purchase cancellations.

Data provider: Laune Hertzlin 694-4631

# CAPITAL BUDGET - DIVISION

(Year-to-Date)



	1991	1992	1993	1994	1995	1/96	2/96	3/96	4/96	5/96	6/96	7/96	8/96	9/96	10/96	11/96	12/96
Actual	150.7	68.4	90.3	76.5	47.2	1.5	2.5	2.9	6.3	9.3	12.1	12.9	15.7	16.4	21.5		
Budget	179.6	75.1	101.4	87.7	64.7	4.8	7.1	11.1	15.7	17.9	19.8	22.6	26.3	27.8	30.1	33.1	39.2
Variance (%)	-16.1	-9.0	-11.0	-12.7	-27.0	-67.6	-65.0	-73.7	-59.7	-47.7	-38.6	-42.8	-40.4	-40.9	-28.3		

## DEFINITION

Capital Expenditures are those directly incurred/budgeted by the Nuclear Division for the construction of new utility plant additions and improvements made to increase efficiency, reliability or safety. Capital fuel costs are excluded.

$$\frac{\text{Y-T-D Actual Expenses} - \text{Y-T-D Budgeted Expenses}}{\text{Y-T-D Budgeted Expenses}} \times 100\% = \text{Capital Variance \%}$$

## STATISTICAL SUMMARY

	Oct Y-T-D Actual (\$M)	Y-T-D Variance (\$M)	%
1996	21.5 (\$M)	-8.5 (\$M)	-28.3%
1995	31.4 (\$M)	-19.7 (\$M)	-38.6%
1996 Y-E Budget:	39.2 (\$M)		

## INDUSTRY PERFORMANCE

	Capital Budget
IBG Actual 1995 Average	\$75.8M
IBG Top Quartile Entry	\$42.6M
IBG Top Quartile Average	\$27.2M

## PERFORMANCE SUMMARY

Capital Expenditures through October 1996 were \$21.5 million which represented a budget underrun of \$8.5 million (or 28.3%). Significant contributors to the variance include: Plant projects cancelled/deferred/or reclassified as O&M; and, underrun in the Steam Generator Replacement Project (SGRP).

Data provider: Laurie Hertzlin 694-4631