

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1)
LaSalle County Station Unit 1

DOCKET NUMBER (2)

PAGE (3)

050003731 OF 04

TITLE (4)

Inadvertent Group II and IV Containment Isolation

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)		
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAME	DOCKET NUMBER(S)	
02	27	84	84	015	00	03	27	84	---	050003731	

OPERATING MODE (9)	THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR 5: (Check one or more of the following) (11)																																			
4	<table border="1"><tr><td>35.402(a)</td><td>35.405(a)</td><td>X</td><td>35.73(a)(1)(ii)</td><td>73.71(a)</td></tr><tr><td>35.405(a)(1)(ii)</td><td>35.38(a)(1)</td><td></td><td>35.73(a)(2)(iv)</td><td>73.71(b)</td></tr><tr><td>35.405(a)(1)(iii)</td><td>35.38(a)(2)</td><td></td><td>35.73(a)(2)(v)</td><td>OTHER (Specify in Abstract below and in Text, NRC Form 363A)</td></tr><tr><td>35.405(a)(1)(iv)</td><td>35.73(a)(2)(i)</td><td></td><td>35.73(a)(2)(vi)</td><td></td></tr><tr><td>35.405(a)(1)(v)</td><td>35.73(a)(2)(ii)</td><td></td><td>35.73(a)(2)(vii)</td><td></td></tr><tr><td>35.405(a)(1)(vi)</td><td>35.73(a)(2)(iii)</td><td></td><td>35.73(a)(2)(viii)</td><td></td></tr><tr><td>35.405(a)(1)(vii)</td><td>35.73(a)(2)(iv)</td><td></td><td>35.73(a)(2)(ix)</td><td></td></tr></table>	35.402(a)	35.405(a)	X	35.73(a)(1)(ii)	73.71(a)	35.405(a)(1)(ii)	35.38(a)(1)		35.73(a)(2)(iv)	73.71(b)	35.405(a)(1)(iii)	35.38(a)(2)		35.73(a)(2)(v)	OTHER (Specify in Abstract below and in Text, NRC Form 363A)	35.405(a)(1)(iv)	35.73(a)(2)(i)		35.73(a)(2)(vi)		35.405(a)(1)(v)	35.73(a)(2)(ii)		35.73(a)(2)(vii)		35.405(a)(1)(vi)	35.73(a)(2)(iii)		35.73(a)(2)(viii)		35.405(a)(1)(vii)	35.73(a)(2)(iv)		35.73(a)(2)(ix)	
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35.405(a)(1)(vii)	35.73(a)(2)(iv)		35.73(a)(2)(ix)																																	

LICENSEE CONTACT FOR THIS LER (12)

NAME	TELEPHONE NUMBER
Vincent Masterson, E.A., Extension 499	AREA CODE 815 357 6761

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRC	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRC
A	JM	011SIV	0101010	N					

SUPPLEMENTAL REPORT EXPECTED (14)

YES (If yes, complete EXPECTED SUBMISSION DATE)	NO	EXPECTED SUBMISSION DATE (15)	MONTH	DAY	YEAR
	X				

ABSTRACT (Limit to 1400 words, i.e., approximately fifteen printed lines) (16)

On February 27, 1984 with the Unit 1 Reactor in Cold Shutdown, a Group II and Group IV isolation occurred. The isolation was a result of the installation of modification M-1-1-83-045, the addition of Main Steam Isolation Valve (MSIV) Group I Automatic Closure Relay Alarm Logic. The alarm relay functions to provide an alarm when the MSIV automatic closure relay is de-energized. During the electrical connection of this alarm relay, the common connection for the 'B' logic isolation relays was lifted resulting in a Group II and Group IV isolation.

The modification was terminated until the cause for isolation could be found. The problem was corrected and the isolation logic subsequently reset. The isolation had no detrimental effect on the plant or the Decay Heat Removal System which was in operation.

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LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

APPROVED OMB NO. 3150-0104
EFFECTIVE 6/31/88

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TEXT of more than one is required, use additional LER Form 266A (17)

I. EVENT DESCRIPTION

On February 27, 1984 with the Unit 1 Reactor in Cold Shutdown, a Group II and IV Primary Containment Isolation (JM) occurred. (Refer to Technical Specification Table 3.6.3-1). At the time of the isolation, a modification was being installed which provided additional relay logic to the Main Steam Isolation Valves, which are the Group I Primary Containment Isolation Valves. The purpose of the Primary Containment Isolation System is to provide isolation of the penetrations passing through the primary containment in the event of an accident which may cause a release of radioactivity. The Group II and Group IV isolations were initiated as a result of work involving the installation of Modification M-1-1-83-045.

After receiving the Group II and Group IV isolations, the modification was immediately terminated. An attempt to reset the isolation was unsuccessful, and a later investigation revealed that the common connection to the neutral bus had been lifted. This lead was immediately landed and the Group II and Group IV isolation subsequently reset. The length of time of the isolation was approximately 5 minutes.

II. CAUSE

The Group II and Group IV Primary Containment Isolations occurred at 1353 on February 27, 1984. At the time of the occurrence, Unit 1 was in Condition 4 (Cold Shutdown) with the 'A' Residual Heat Removal Loop in Shutdown Cooling.

The isolation occurred during the installation of Modification M-1-1-83-045, which adds additional relays to provide an alarm if a Group I Main Steam (SB) Isolation Valve (MSIV) Automatic Closure Relay de-energizes. This was done to prevent unintended scrams when performing MSIV Isolation Channel Surveillances and Logic Tests.

The problem which occurred was that the common connection to the neutral bus for those relays causing a Group I, Group II, and Group IV Primary Containment Isolation was lifted in order to connect a separate common connection for the new alarm relay (KX7B) at that terminal. The K35B relay was momentarily de-energized when electrically connecting the alarm relay, and this resulted in the Group II and Group IV isolation. A Group I isolation did not occur as the Main Steam Isolation Valve Logic consists of 1 out of 2 taken twice. Although the 'B' logic had tripped when lifting the common at the neutral bus, the 'A' logic remained energized and therefore did not cause a Group I isolation.

The root cause for the occurrence appears to be personnel oversight on the part of those individuals involved in the design of the modification not fully realizing the resulting isolation when disconnecting the common lead at the neutral bus to electrically connect the alarm relay KX7B.

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TEXT IF more space is required, use additional NRC Form 306a's (17)

III. PROBABLE CONSEQUENCES OF THE OCCURRENCE

Upon inadvertent initiation of the Primary Containment Isolation Groups II and IV, the following resulted:

Group II Actions:

- A. Start Post LOCA Containment Monitoring Systems (IP)
- B. Trip Drywell Humidity Monitoring Systems (IJ)
- C. Secures Instrument Nitrogen to Drywell (LD) Pneumatic Valves
- D. Isolate Drywell Floor Drains Pump Discharge (WR)
- E. Isolate Drywell Equipment Drains Pump Suction (WR)
- F. Isolates Reactor Recirc Flow Control Hydraulics (AD)
- G. Isolates Testable Check Valves for the High Pressure Core Spray, Low Pressure Core Spray, Residual Heat Removal, and Reactor Core Isolation Cooling Systems
- H. Isolates Reactor Building Closed Cooling Water to the Reactor Recirc Pump Motor Coolers (AD)
- I. Isolates Primary Containment Chilled Water Coolers (KH)
- J. Air Loads the Outboard Feedwater Check Valves in the Closed Direction (SJ)

Group IV Actions:

- A. Trips Reactor Building Vent Fans (VA)
- B. Isolates Reactor Building Vent Dampers (VA)
- C. Isolates Primary Containment Vent and Purge Dampers (VA)
- D. Starts the Standby Gas Treatment System (BH)

At the time of the isolation the reactor was in Condition 4, Cold Shutdown. The Residual Heat Removal (RHR) 'A' loop was in the Shutdown Cooling mode. The Group II and Group IV isolation in no way impaired the ability of the RHR 'A' loop to provide removal of reactor decay heat.

LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

U.S. NUCLEAR REGULATORY COMMISSION

APPROVED OMB NO. 3150-0104

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YEAR SEQUENTIAL REVISION

NUMBER NUMBER NUMBER

8 4 - 0 1 5 - 0 0

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TEXT of report entered as required, use additional NRC Form 308A (1/77)

III. PROBABLE CONSEQUENCES OF THE OCCURRENCE (Continued)

Although the isolation was inadvertent and unintended from an operating standpoint, the Primary Containment Isolation System did perform and carry out fully its designed function. Containment monitoring was provided by the Post Loss of Coolant Accident (LOCA) Monitoring Systems. Reactor building closed cooling water to the Reactor Recirculation (RR) Pump Motor Coolers was isolated for a short period of time, and as the RR pumps were in slow speed had no effect on their reliable operation.

The end result of the Group II and Group IV isolation was the placement of the plant in a conservative and safe condition as designed.

IV. CORRECTIVE ACTION

Upon discovering the cause of the isolation and correcting the problem, the Group II and Group IV Isolation was subsequently reset and systems returned to their normal line-up as called for.

Training will be conducted on this for the Technical Staff Department.
(AIR 01-84-67049)

V. PREVIOUS EVENTS

None.

VI. NAME AND TELEPHONE NUMBER OF PREPARER

Vincent Masterson, 815-357-6761, extension 499.



Commonwealth Edison
LaSalle County Nuclear Station
Rural Route #1, Box 220
Marseilles, Illinois 61341
Telephone 815/357-6761

RAN
4-5-84

March 27, 1984

U. S. Nuclear Regulatory Commission
Document Control Desk
Washington, D.C. 20555

Dear Sir:

Reportable Occurrence Report #84-015-00, Docket #050-373 is being submitted to your office in accordance with 10 CFR 50.73.

GJD/MLD/kg

Enclosure

xc: NRC, Regional Director
INPO-Records Center
File/NRC

CE Dargatz

G. J. Diederich
Superintendent
LaSalle County Station

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LER

LER # 323-84015

EVENT DATE 840227

INPO RCVD DATE 840404 pcc

NSAC RCVD DATE

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