



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

COMMONWEALTH EDISON COMPANY

DOCKET NO. 50-295

ZION NUCLEAR POWER STATION, UNIT 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 92  
License No. DPR-39

1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment by Commonwealth Edison Company (the licensee) dated August 19, 1985, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-39 is hereby amended to read as follows:

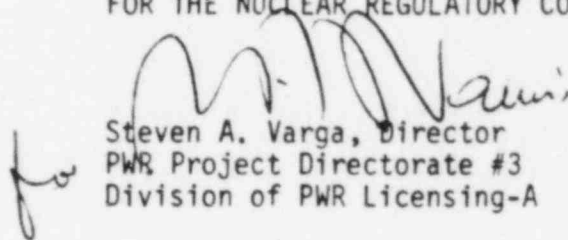
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(2) Technical Specifications

The Technical Specifications contained in Appendices - A and B, as revised through Amendment No. 92, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

  
Steven A. Varga, Director  
PWR Project Directorate #3  
Division of PWR Licensing-A

Attachment:  
Changes to the Technical  
Specifications

Date of Issuance: January 16, 1986



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

COMMONWEALTH EDISON COMPANY

DOCKET NO. 50-304

ZION NUCLEAR POWER STATION, UNIT 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 82  
License No. DPR-48

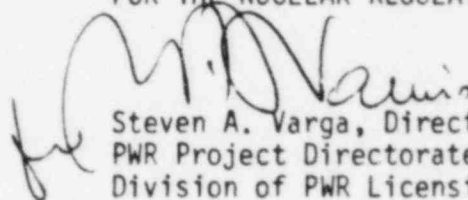
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  - A. The application for amendment by Commonwealth Edison Company (the licensee) dated August 19, 1985, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
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  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-48 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 82, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

  
Steven A. Varga, Director  
PWR Project Directorate #3  
Division of PWR Licensing-A

Attachment:  
Changes to the Technical  
Specifications

Date of Issuance: January 16, 1986

ATTACHMENT TO LICENSE AMENDMENTS

AMENDMENT NO. 92    FACILITY OPERATING LICENSE NO. DPR-39

AMENDMENT NO. 82    FACILITY OPERATING LICENSE NO. DPR-48

DOCKET NOS. 50-295 AND 50-304

Revise Appendix A as follows:

Remove Pages

105

119

Insert Pages

105

119

LIMITING CONDITION FOR OPERATION	SURVEILLANCE REQUIREMENT																								
3.3.4	<p data-bbox="1144 232 1868 297">4.3.4 D. Materials Irradiation Surveillance Specimen Inspection. (per unit)</p> <p data-bbox="1353 321 1942 524">Specimen capsules to be used in the reactor vessel material surveillance program shall be withdrawn during the refueling period either immediately preceeding or following the Effective Full Power Years (EFPY) of unit life as follows:</p> <p data-bbox="1353 557 1761 589"><u>CAPSULE WITHDRAWAL SCHEDULE</u></p> <p data-bbox="1527 621 1623 654"><u>UNIT 1</u></p> <table data-bbox="1353 678 1921 889"> <thead> <tr> <th>CAPSULE DESIGNATION</th><th>CAPSULE REMOVAL TIME (EFPY)</th></tr> </thead> <tbody> <tr> <td>T</td><td>REMOVED (1.16)</td></tr> <tr> <td>U</td><td>REMOVED (3.52)</td></tr> <tr> <td>X</td><td>REMOVED (5.17)</td></tr> <tr> <td>Y</td><td>8.5</td></tr> <tr> <td>W,S,V,Z</td><td>STAND BY</td></tr> </tbody> </table> <p data-bbox="1527 914 1623 946"><u>UNIT 2</u></p> <table data-bbox="1353 971 1921 1182"> <thead> <tr> <th>CAPSULE DESIGNATION</th><th>CAPSULE REMOVAL TIME (EFPY)</th></tr> </thead> <tbody> <tr> <td>U</td><td>REMOVED (1.27)</td></tr> <tr> <td>T</td><td>REMOVED (3.56)</td></tr> <tr> <td>Y</td><td>8.5</td></tr> <tr> <td>X</td><td>STAND BY</td></tr> <tr> <td>W,S,V,Z</td><td>STAND BY</td></tr> </tbody> </table>	CAPSULE DESIGNATION	CAPSULE REMOVAL TIME (EFPY)	T	REMOVED (1.16)	U	REMOVED (3.52)	X	REMOVED (5.17)	Y	8.5	W,S,V,Z	STAND BY	CAPSULE DESIGNATION	CAPSULE REMOVAL TIME (EFPY)	U	REMOVED (1.27)	T	REMOVED (3.56)	Y	8.5	X	STAND BY	W,S,V,Z	STAND BY
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## Basis

- 4.3.4 The surveillance inspection program has been developed to comply with Section XI of the ASME Boiler and Pressure Vessel Code and applicable addenda as required by 10 CFR 50, Section 50.55a(g), except where specific written relief has been granted by the NRC pursuant to 10 CFR 50, Section 50.55a(g)(6)(i). The design of the plant, state of non-destructive testing technology, and access to areas to be inspected require such relief.

The Reactor Vessel Material Surveillance Program is designed to evaluate the effects of radiation on the fracture toughness of reactor vessel steel based on the transition temperature approach and the fracture mechanics approach.

10 CFR 50, Appendix H, paragraph II B.1 requires that the reactor vessel material surveillance program shall meet the requirements of ASTM E185-82 such that the surveillance capsules represent end-of-life fluences at the reactor vessel 1/4 and wall thicknesses. Previous capsules were removed under Amendment Nos. 62 and 59.