



SACRAMENTO MUNICIPAL UTILITY DISTRICT □ 6201 S Street, P.O. Box 15830, Sacramento CA 95852-1830. (916) 452-3211
AN ELECTRIC SYSTEM SERVING THE HEART OF CALIFORNIA

RJR 85-563

November 25, 1985

DIRECTOR OF NUCLEAR REACTOR REGULATION
ATTENTION HUGH L THOMPSON JR DIRECTOR
DIVISION OF LICENSING
U S NUCLEAR REGULATORY COMMISSION
WASHINGTON DC 20555

DOCKET NO. 50-312
LICENSE NO DPR-54
PROPOSED AMENDMENT NO. 100, REVISION 3

In accordance with 10 CFR 50.59, the Sacramento Municipal Utility District proposes to amend its Operating License DPR-54 for Rancho Seco Nuclear Generating Station Unit 1. Enclosed is Proposed Amendment No 100, Revision 3. Attachment I to this submittal is the No Significant Hazards Evaluation in accordance with 10 CFR 50.92.

This proposal is in response to Enclosure 1 and Enclosure 3 of Generic Letter 83-37. Therefore, the District wishes to rescind Proposed Amendment No 83, Supplement 2.

Attachment II to this submittal is a description of the proposed changes for Amendment 100, Revision 3. Attachment III contains a safety analysis justifying this set of changes.

Pursuant to 10 CFR 50.91(a), we have provided a copy of this letter, the Proposed Change in Technical Specifications, the Safety Analysis and our analyses of Significant Hazards Considerations to Joseph Ward, the designated representative of the State of California.

Since this is a revision to Proposed Amendment No. 100, no additional license fees are required.

If you have any questions concerning this proposal, please contact Mr. Ron Colombo at Rancho Seco Nuclear Generating Station.

William K. Latham

WILLIAM K. LATHAM
ACTING GENERAL MANAGER

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PDR ADOCK 05000312
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Attachments (3)

Subscribed and sworn to before me
this 25th day of November, 1985.

Patricia K. Geisler
Notary Public



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ATTACHMENT I

NO SIGNIFICANT HAZARDS CONSIDERATIONS

These proposed changes provide additional operational and surveillance requirements for a number of NUREG-0737 items. Generic Letter No. 83-37 provides guidance on acceptable changes for these items in the form of Model Standard Technical Specifications. This submittal meets the standards of the generic letter as discussed in Attachment III. Therefore, operation of Rancho Seco in accordance with this Proposed Amendment:

1. does not involve a significant increase in the probability or consequences of an accident previously evaluated.
2. does not create the possibility of a new or different kind of accident from any accident previously evaluated, and
3. does not involve a significant reduction in a margin of safety.

Therefore, significant safety hazards are not associated with this Proposed Amendment.

IMPLEMENTATION

It is requested that the amendment authorizing this change become effective 120 days after receipt, to allow for the necessary procedural revisions to be put in place.

ATTACHMENT II

DESCRIPTION OF PROPOSED CHANGES

1. Page iv, Table of Contents, Section 3.5.5; This is a new section for Accident Monitoring Instrumentation.
2. Page viii, Table of Contents, Section 6.18; This is a new section for Post Accident Sampling.
3. Page ix, List of Tables, Table 3.5.5-1; This is a new table for Accident Monitoring Instrumentation Operability Requirements.
4. Page 3-2, Specification 3.1.1.6; Operability requirements have been established for the Reactor Coolant System High Point Vents.
5. Page 3-2a, 4th paragraph of the Bases; The word "safety" was added for clarification.
6. Page 3-2a, last paragraph of the Bases; A description of the purpose and operating status of the high point vents is provided.
7. Pages 3-38a through 3-38c, Specification 3.5.5; Accident Monitoring Instrumentation Limiting Conditions for Operation are listed.
8. Page 4-7c, Table 4.1-1, items 58 through 66; Accident Monitoring Instrument Surveillance Requirements are listed.
9. Pages 4-8 and 4-8a, Table 4.1-2, item 14; Surveillance Standards are provided for the RCS High Point Vents.
10. Pages 4-39 and 4-39a, Specification 4.8; Auxiliary feedwater pump periodic testing specifications have been revised so that they are in agreement with B&W Standard Technical Specifications.
11. Page 6-12f, Specification 6.9.5D; A special report will be required when accident monitoring instrumentation is inoperable.
12. Page 6-22, Specification 6.18; Post accident sampling administrative controls have been established.