

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1) Washington Nuclear Plant - Unit 2						DOCKET NUMBER (2) 0 5 0 0 0 3 9 7 1 OF 0 3						PAGE (3)																																											
TITLE (4) Improper RPS APRM Flow Biased Trip Setpoint Change During Single Recirc Loop Operation																																																							
EVENT DATE (8)				LER NUMBER (8)				REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)																																												
MONTH	DAY	YEAR		SEQUENTIAL NUMBER		REVISION NUMBER		MONTH	DAY	YEAR	FACILITY NAMES				DOCKET NUMBER(S)																																								
0	7	2		2	8	5		0	5	1		0	0	0		8	2	1		8	5							0 5 0 0 0																											
OPERATING MODE (9)				THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: (Check one or more of the following) (11)																																																			
POWER LEVEL (10)				20.402(b)				20.406(e)				50.73(a)(2)(iv)				50.73(a)(2)(v)				50.73(a)(2)(vi)				50.73(a)(2)(vii)				50.73(a)(2)(viii)(A)				50.73(a)(2)(viii)(B)				50.73(a)(2)(ix)				50.72(b)(2)iii(D)															
0				20.406(a)(1)(i)				50.38(c)(1)				50.38(c)(2)				50.73(a)(2)(i)				50.73(a)(2)(ii)				50.73(a)(2)(iii)				50.73(a)(2)(iv)				50.73(a)(2)(v)				50.73(a)(2)(vi)				50.73(a)(2)(vii)				50.73(a)(2)(viii)				50.73(a)(2)(ix)				50.72(b)(2)iii(D)			
1				20.406(a)(1)(ii)				50.38(c)(1)				50.38(c)(2)				50.73(a)(2)(i)				50.73(a)(2)(ii)				50.73(a)(2)(iii)				50.73(a)(2)(iv)				50.73(a)(2)(v)				50.73(a)(2)(vi)				50.73(a)(2)(vii)				50.73(a)(2)(viii)				50.73(a)(2)(ix)				50.72(b)(2)iii(D)			
1				20.406(a)(1)(iii)				50.38(c)(1)				50.38(c)(2)				50.73(a)(2)(i)				50.73(a)(2)(ii)				50.73(a)(2)(iii)				50.73(a)(2)(iv)				50.73(a)(2)(v)				50.73(a)(2)(vi)				50.73(a)(2)(vii)				50.73(a)(2)(viii)				50.73(a)(2)(ix)				50.72(b)(2)iii(D)			
1				20.406(a)(1)(iv)				50.38(c)(1)				50.38(c)(2)				50.73(a)(2)(i)				50.73(a)(2)(ii)				50.73(a)(2)(iii)				50.73(a)(2)(iv)				50.73(a)(2)(v)				50.73(a)(2)(vi)				50.73(a)(2)(vii)				50.73(a)(2)(viii)				50.73(a)(2)(ix)				50.72(b)(2)iii(D)			
1				20.406(a)(1)(v)				50.38(c)(1)				50.38(c)(2)				50.73(a)(2)(i)				50.73(a)(2)(ii)				50.73(a)(2)(iii)				50.73(a)(2)(iv)				50.73(a)(2)(v)				50.73(a)(2)(vi)				50.73(a)(2)(vii)				50.73(a)(2)(viii)				50.73(a)(2)(ix)				50.72(b)(2)iii(D)			
LICENSEE CONTACT FOR THIS LER (12)														TELEPHONE NUMBER																																									
NAME														AREA CODE																																									
W. S. Davison, Compliance Engineer														5 0 9 3 7 7 - 1 2 5 0 1																																									
COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)														Ext. 2726																																									
CAUSE	SYSTEM	COMPONENT	MANUF- TURER	REPORTABLE TO NPROS		CAUSE	SYSTEM	COMPONENT	MANUF- TURER	REPORTABLE TO NPROS		CAUSE	SYSTEM	COMPONENT	MANUF- TURER	REPORTABLE TO NPROS																																							
SUPPLEMENTAL REPORT EXPECTED (14)														EXPECTED SUBMISSION DATE (15)																																									
YES (If yes, complete EXPECTED SUBMISSION DATE)														X NO																																									

ABSTRACT (Limit to 1400 spaces, i.e. approximately fifteen single-space typewritten lines) (16)

The plant was shifted to single loop 60 Hertz recirculation system operation on 7/19/85. On 7/22/85 the Average Power Range Monitor (APRM), Reactor Protection System (RPS) thermal power trip calibration procedure was performed. Since the procedure contained only values for two recirculation loop operation, incorrect values for single loop operation were used. When the problem was discovered, the APRM, RPS flow biased thermal trip values were reset to single loop values.

The requirement for minimum operable APRM channels per RPS trip system was not met for a period of approximately 4 hours.

8508300540 850821
PDR ADOCK 05000397
S PDR

LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

APPROVED OMB NO. 3150-0104

EXPIRES: 8/31/85

FACILITY NAME (1) Washington Nuclear Plant - Unit 2	DOCKET NUMBER (2) 0 5 0 0 0 3 9 7 8 5 - 0 5 1 - 0 0 0 2 OF 0 3	LER NUMBER (3)			PAGE (3)		
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER			

TEXT (If more space is required, use additional NRC Form 365A's) (17)

Plant Conditions

- a) Power Level - 60.5%
- b) Reactor Mode - 1

Event

At approximately 1530 on 7/19/85, a decision was made by the Plant Operating Committee to shift to single recirculation loop operation.

At 0110 on 7/20/85, the Technical Department procedure for Average Power Range Monitor (APRM) Flow Bias Setpoint was performed to adjust the APRM, flow biased thermal power setpoints. At approximately 1400 on 7/20/85 procedure deviations were approved for the Instrument Department APRM Thermal Trip weekly calibration procedure but were not distributed. At 0030 on 7/22/85 the weekly procedure for the APRM Thermal Trip Calibration which contained only values for two loop operation was performed. Instrumentation and Control (I&C) technicians coming in on day shift, 7/22/85, discovered the error and informed the Shift Manager at 0800 hours. Division One Channels A, C and E were subsequently recalibrated to the single recirculation pump loop values.

Immediate Corrective Action

- o At 0920 on 7/22/85, the APRM, RPS flow biased thermal power trips for Channels A, C and E, were reset to single recirculation loop values.

Further Corrective Action

The Instrumentation Department procedure was changed to incorporate the single recirculation loop values. The root cause of this event was the failure to perform the weekly APRM Thermal Trip setpoint calibration with the latest approved procedure. This is considered to be an isolated event. Individuals involved will be counselled and provided additional training. A contributing cause of this event was the failure of the original Instrumentation procedure to address single recirculation loop operation. To preclude recurrence of this event a new plant procedure will be written to identify all actions to be taken prior to commencing single recirculation loop operation including all procedures which require deviation.

Safety Significance

The requirement for minimum operable channels per RPS trip system was exceeded for the APRM, RPS flow biased simulated thermal power trip for a period of approximately 4 hours.

There were no safety consequences as no actual initiating conditions were present during the period of the event.

This event did not affect the safety of the public or Plant personnel.

LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

U.S. NUCLEAR REGULATORY COMMISSION

APPROVED OMB NO. 3150-0104

EXPIRES: 8/31/85

FACILITY NAME (1)

DOCKET NUMBER (2)

LER NUMBER (3)

PAGE (3)

Washington Nuclear Plant - Unit 2

0 5 0 0 0 3 9 7 8 5 - 0 5 1 - 0 0 0 3 OF 0 3

TEXT (If more space is required, use additional NRC Form 388A's) (17)

Similar Events

None

EIIS InformationEIIS Reference

Text Reference

System

Component

APRM

IG

CHA

Washington Public Power Supply System

P.O. Box 968 3000 George Washington Way Richland, Washington 99352 (509) 372-5000

Docket No. 50-397

August 21, 1985

Document Control Desk
U.S. Nuclear Regulatory Commission
Washington, D.C. 20555

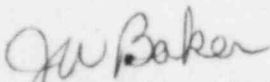
Subject: NUCLEAR PLANT NO. 2
LICENSEE EVENT REPORT NO. 85-051

Dear Sir:

Transmitted herewith is Licensee Event Report No. 85-051 for WNP-2 Plant. This report is submitted in response to the report requirements of 10CFR50.73 and discusses the item of reportability, corrective action taken, and action taken to preclude recurrence.

This is the follow-up report to the verbal notification given at 1940 hours on July 22, 1985.

Very truly yours,



C.M. Powers (M/D 927M)
WNP-2 Plant Manager

CMP:1a

Enclosure:

Licensee Event Report No. 85-051

cc: Mr. John B. Martin, NRC - Region V
Mr. A. D. Toth, NRC - Site (901A)
Ms. Dottie Sherman, ANI
INPO Records Center - Atlanta, GA
Mr. W. S. Chin, BPA

IE22
1/1