

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1) Salem Generating Station - Unit 2										DOCKET NUMBER (2) 0 5 0 0 0 3 1 1 1				PAGE (3) 1 OF 4									
TITLE (4) Reactor Coolant System Unidentified Leakage Greater Than T/S Limit																							
EVENT DATE (5)			LER NUMBER (6)				REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)													
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES				DOCKET NUMBER(S)										
0	7	2	0	8	5	8	5	0	1	5	0	0	0	8	1	9	8	5	0	5	0	0	0
OPERATING MODE (9) 1		THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: (Check one or more of the following) (11)																					
POWER LEVEL (10) 11010		20.402(b)				20.406(c)				50.73(a)(2)(iv)				73.71(b)									
		20.406(a)(1)(i)				50.36(c)(1)				50.73(a)(2)(v)				73.71(c)									
		20.406(a)(1)(ii)				50.36(c)(2)				50.73(a)(2)(vii)				OTHER (Specify in Abstract below and in Text, NRC Form 366A)									
		20.406(a)(1)(iii)				50.73(a)(2)(i)				50.73(a)(2)(viii)(A)													
		20.406(a)(1)(iv)				50.73(a)(2)(ii)				50.73(a)(2)(viii)(B)													
		20.406(a)(1)(v)				50.73(a)(2)(iii)				50.73(a)(2)(ix)													
LICENSEE CONTACT FOR THIS LER (12)																							
NAME J. L. Rupp-Operations Licensing Engineer										TELEPHONE NUMBER 6109 31391-4309													
COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)																							
CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPDs		CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPDs													
X	AIB	TITV	R344	N																			
X	AIB	TIV	R344	N																			
SUPPLEMENTAL REPORT EXPECTED (14)																EXPECTED SUBMISSION DATE (15)		MONTH	DAY	YEAR			
YES (If yes, complete EXPECTED SUBMISSION DATE)																NO							

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

On July 20, 1985, following the completion of a routine Reactor Coolant System (RCS) water inventory balance surveillance, RCS unidentified leakage was determined to be 1.001 GPM; the maximum allowed by Technical Specifications is 1.0 GPM. Action Statement 3.4.7.2.b was entered, and a controlled shutdown was initiated in accordance with the action requirements. Investigation revealed the packing glands on 2PR9 (Pressurizer Safety Valve Loop Seal Drain Valve) and 2PS8 (Pressurizer Instrumentation Tap) to be the source of the leakage. The leaks were terminated by adjustment of the packing on both valves. Following the completion of a RCS water inventory balance, which verified that RCS unidentified leakage was less than 1.0 GPM, Action Statement 3.4.7.2.b was terminated. The leak, which was discovered by the performance of routine surveillance, was identified early; i.e., when unidentified leakage just exceeded the allowable limit of 1.0 GPM. This event involved no undue risk to the health or safety of the public. However, because of Technical Specification requirements, a shutdown was initiated, and because that shutdown was completed, the event is reportable in accordance with the requirements of 10CFR 50.73(a)(2)(i)(A).

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PLANT AND SYSTEM IDENTIFICATION:

Westinghouse - Pressurized Water Reactor

Energy Industry Identification System (EIIS) codes are identified in the text as [XX].

IDENTIFICATION OF OCCURRENCE:

Reactor Coolant System [AB] Unidentified Leakage Exceeded Technical Specification Limit

Event Date: 07/20/85

Report Date: 08/19/85

This report was initiated by Incident Report No. 85-180

CONDITIONS PRIOR TO OCCURRENCE:

Mode 1 - Rx Power 100 % - Unit Load 1130 MWe

DESCRIPTION OF OCCURRENCE:

Technical Specification 3.4.7.2 requires that Reactor Coolant System (RCS) leakage be limited to:

- a. no pressure boundary leakage,
- b. one (1) GPM unidentified leakage,
- c. one (1) GPM total primary-to-secondary leakage through all steam generators and five-hundred (500) gallons per day through any one steam generator,
- d. ten (10) GPM identified leakage from the RCS,
- e. forty (40) GPM controlled leakage at a RCS pressure of 2230 (+-20) psig, and
- f. one (1) GPM leakage at a RCS pressure of 2230 (+-20) psig from any RCS pressure isolation valve specified in Table 3.4-1 of the Technical Specifications.

Technical Specification Action Statement 3.4.7.2.b states:

With any RCS leakage greater than any one of the above limits, excluding pressure boundary leakage and leakage from RCS pressure isolation valves, reduce the leakage rate to within limits within four (4) hours or be in at least hot standby (Mode 3) within the next six (6) hours and in cold shutdown (Mode 5) within the following thirty (30) hours.

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DESCRIPTION OF OCCURRENCE: (cont'd)

At 0410 hours, July 20, 1985, following the completion of a RCS water inventory balance in accordance with Surveillance Procedure SP(O) 4.7.2.d, RCS unidentified leakage was determined to be 1.001 GPM. Action Statement 3.4.7.2.b was entered at that time, and a controlled shutdown was initiated in accordance with the action requirements. At 0449 hours, in accordance with the requirements of the Code of Federal Regulations, 10CFR 50.72(b)(1)(i)(A), the Commission was notified of the initiation of the Unit shutdown. At 1203 hours, the Unit was stabilized in Mode 3.

APPARENT CAUSE OF OCCURRENCE:

Investigation revealed the packing glands on 2PR9 (Pressurizer Safety Valve Loop Seal Drain Valve) and 2PS8 (Pressurizer Instrumentation Tap) to be the source of the leakage.

ANALYSIS OF OCCURRENCE:

The RCS leakage limits are based on ensuring the ability to detect leakage from the reactor coolant boundary. The one (1) GPM value is sufficiently low to ensure early detection of additional leakage; the ten (10) GPM identified leakage limitation provides allowance for a limited amount of leakage from known sources whose presence will not interfere with the detection of unidentified leakage by the leak detection systems; and, pressure boundary leakage of any magnitude is unacceptable, since it may be indicative of an impending gross failure of the pressure boundary.

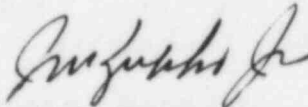
The leak, which was discovered by the performance of routine surveillance, was identified early; i.e., when unidentified leakage just exceeded the allowable limit of one (1) GPM. Operation subsequent to the discovery was in accordance with the action requirements. The leak was identified, determined not to be from the pressure boundary, and unidentified leakage was reduced to less than one (1) GPM. This event therefore involved no undue risk to the health or safety of the public. However, because of Technical Specification requirements, a shutdown was initiated, and because that shutdown was completed, the event is reportable in accordance with the requirements of the Code of Federal Regulations, 10CFR 50.73(a)(2)(i)(A).

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CORRECTIVE ACTION:

The leaks were terminated by adjustment of the packing on both 2PR9 and 2PS8. A RCS water inventory balance was then performed in accordance with Surveillance Procedure SP(O) 4.7.2.d. The surveillance was completed at 0746 hours, July 21, 1985. The results verified RCS unidentified leakage to be less than 1.0 GPM and, at 0804 hours, Action Statement 3.4.7.2.b was terminated.


General Manager-
Salem Operations

JLR:tns

SORC Mtg 85-120



Public Service Electric and Gas Company P.O. Box E Hancocks Bridge, New Jersey 08038

Salem Generating Station

August 19, 1985

U. S. Nuclear Regulatory Commission
Document Control Desk
Washington, DC 20555

Dear Sir:

SALEM GENERATING STATION
LICENSE NO. DPR-75
DOCKET NO. 50-311
UNIT NO. 2
LICENSEE EVENT REPORT 85-015-00

This Licensee Event Report is being submitted pursuant to the requirements of 10CFR 50.73(a)(2)(i)(A). This report is required within thirty days of discovery.

Sincerely yours,

A handwritten signature in dark ink, appearing to read "J. M. Zupko, Jr.", is written above the typed name.

J. M. Zupko, Jr.
General Manager-
Salem Operations

JLR:pc

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