

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1): LaSalle County Station Unit 2										DOCKET NUMBER (2): 0 5 0 0 0 3 7 4										PAGE (3) 1 OF 93	
TITLE (4) Actuation of E SRV/SRV N Actuation																					
EVENT DATE (5)			LER NUMBER (6)				REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)											
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES			DOCKET NUMBER(S)									
07	23	85	85	035		00	08	14				0 5 0 0 0									
OPERATING MODE (9): 1		THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR 43 (Check one or more of the following) (11)																			
POWER LEVEL (10): 0.213		20.402(a)		20.408(a)		X		20.73(a)(2)(iv)		20.73(a)(2)(iv)		20.73(a)(2)(iv)									
		20.408(a)(1)(i)		20.36(a)(1)				20.73(a)(2)(v)		20.73(a)(2)(v)		20.73(a)(2)(v)									
		20.408(a)(1)(ii)		20.36(a)(2)				20.73(a)(2)(vi)		20.73(a)(2)(vi)		20.73(a)(2)(vi)									
		20.408(a)(1)(iii)		20.73(a)(2)(i)				20.73(a)(2)(vii)(A)		20.73(a)(2)(vii)(A)		20.73(a)(2)(vii)(A)									
		20.408(a)(1)(iv)		X 20.73(a)(2)(ii)				20.73(a)(2)(viii)(B)		20.73(a)(2)(viii)(B)		20.73(a)(2)(viii)(B)									
		20.408(a)(1)(v)		20.73(a)(2)(iii)				20.73(a)(2)(ix)		20.73(a)(2)(ix)		20.73(a)(2)(ix)									
LICENSEE CONTACT FOR THIS LER (12):																					
NAME John Ullrich, extension 571										TELEPHONE NUMBER AREA CODE 8 1 4 5 3 5 7 1 7 6 7 6 1											
COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13):																					
CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRC	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRC												
D	SIB	1 RIV	CL71110	N																	
SUPPLEMENTAL REPORT EXPECTED (14):																					
YES (If yes, complete EXPECTED SUBMISSION DATE):										X NO											
EXPECTED SUBMISSION DATE (15):										MONTH DAY YEAR											

ABSTRACT (Limit to 1600 spaces, i.e., approximately fifteen single space typewritten lines) (16):

On July 23, 1985, during the performance of LaSalle Special Test (LST) 85-103, Set Pressure Verification Device (SPVD) Operational Test, Safety Relief Valve (SRV-SB) 2B21-F013E was inadvertently opened twice and Safety Relief Valve 2B21-F013N was inadvertently opened once.

At the time of this event LaSalle Unit 2 was ~~at~~ operating Condition 1 at 23% power. The cause of the inadvertent actuations was ~~performed~~ the SPVD Operational Test at an excessive pressure (955 psig). The SPVD is designed to test the SRV setpoints at a system pressure of 300 psig below the nameplate set pressure of the SRV. This results in a test pressure range of 850-905 psig reactor pressure.

This event was of minimal significance because both SRV's reclosed as indicated by the tailpipe temperatures returning to their prelift values.

The SRV's were verified to be closed by observing tailpipe temperatures. The SPVD Operational Test was terminated to preclude any additional SRV actuations. Future testing of the Set Pressure Verification Device will only be performed at a reactor pressure between 850-905 psig.

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LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

US NUCLEAR REGULATORY COMMISSION

APPROVED OMB NO 3150-0104

EXPIRES 8/31/85

FACILITY NAME (1)	DOCKET NUMBER (2)	LER NUMBER (5)			PAGE (3)		
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER			
LaSalle County Station Unit 2	05000374	85	035	00	02	OF	03

TEXT (If more space is required, use additional NRC Form 366A 2/117)

I. EVENT DESCRIPTION

On July 23, 1985, during the performance of LaSalle Special Test (LST) 85-103, Set Pressure Verification Device (SPVD) Operational Test, Safety Relief Valve 2B21-F013E was inadvertently opened twice and Safety Relief Valve 2B21-F013N was inadvertently opened once.

II. CAUSE

At the time of this event, LaSalle Unit 2 was in Operating Condition 1 at approximately 23% power. At the start of testing, Station personnel were not aware of any maximum pressure restrictions for SPVD testing, i.e., none are listed in the SPVD technical manual. The cause of these inadvertent Safety Relief Valve (SRV-SB) actuations was performance of the SPVD operational test at an excessive system pressure (955 psig). Following these two valve actuations, a Crosby Valve and Gage Company representative informed Station personnel that the SPVD was designed to test the set pressure of the SRV's at a system pressure of 300 psig below the nameplate set pressure of the SRV. This results in a pressure range of 850-905 psig, reactor pressure, in which this test can be performed. Due to the excessive pressure, the force on the under side of the valve disc is sufficient to cause the SRV to fully open when nitrogen is admitted to the SPVD bellows assembly, mounted on the top of the SRV, during the test. When the nitrogen pressure was vented from the bellows, the SRV's reclosed. These valves are 6 X R X 10 style HB-65-BP Dual Function Safety Relief Valves, manufactured by Crosby Valve and Gage Company.

III. PROBABLE CONSEQUENCES OF THE OCCURRENCE

This event was of minimal significance because the SRV's both reclosed as indicated by the tailpipe temperature returning to their prelift values.

IV. CORRECTIVE ACTION

The SRV's were verified to be closed by verifying the SRV tailpipe temperatures returned to the prelift value. The Set Pressure Verification Device Operational Test, LST-85-103, was terminated to preclude any additional SRV actuations. Future testing of the Set Pressure Verification Device, on the Unit 2 SRV's will only be performed at a reactor pressure between 850-905 psig. Action Item Record 374-200-85-00128 was written to track the development of a surveillance procedure for the operation of the SPVD.

V. PREVIOUS OCCURRENCES

None.

LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

FACILITY NAME (1): LaSalle County Station Unit 2	DOCKET NUMBER (2): 0 5 0 0 0 3 7 4	LER NUMBER (8)			PAGE (3)		
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER			
		8 5	— 0 3 5	— 0 0	0 3	OF	0 3

TEXT (If more space is required, use additional NRC Form 366A's) (17)

VI. NAME AND TELEPHONE NUMBER OF PREPARER

John Ullrich, 815/357-6761, extension 571.



Commonwealth Edison
LaSalle County Nuclear Station
Rural Route #1, Box 220
Marseilles, Illinois 61341
Telephone 815/357-6761

August 14, 1985

U.S. Nuclear Regulatory Commission
Document Control Desk
Washington, D.C. 20555

Dear Sir:

Reportable Occurrence Report #85-035-00, Docket #050-374 is being submitted to your office in accordance with 10CFR 50.73.

for *R.D. Bishop*
G. J. Diederich
Station Manager
LaSalle County Station

GJD/DRR/kg

Enclosure

xc: NRC, Regional Director
INPO-Records Center
File/NRC

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