AVERAGE DAILY UNIT POWER LEVEL

Docket No. 50-272
Unit Name Salem # 1
Date September 10, 1985
Telephone 609-935-6000
Extension 4455

Completed by J. P. Ronafalvy

Month August 1985

Day	Average Daily Power Level (MWe-NET)	Day	Average Daily Power Level (MWe-NET)
1	1068	17	1068
2	1069	18	1073
3	1065	19	1073
4	1044	20	1090
5	1075	21	1071
6	1077	22	1079
7	1070	23	1079
8	1084	24	1075
9	1085	25	1080
10	1064	26	1080
11	1060	27	1075
12	1084	28	1118
13	1074	29	1046
14	1075	30	1108
15	1078	31	1019
16	1073		

IE24
1/1

OPERATING DATA REPORT

Docket No. 50-272
Date September 10, 1985
Telephone 935-6000
Extension 4455

Completed by J. P. Ronafalvy

Operating Status

		Salem No. 1	Notes
1.	Unit Name		
2.	Reporting Period	August 1985	
3.	Licensed Thermal Power (MWt)	3338	
4.	Nameplate Rating (Gross MWe)	1170	
5.	Design Electrical Rating (Net MWe)	1090	
6.	Maximum Dependable Capacity (Gross MWe)	1124	
7.	Maximum Dependable Capacity (Net MWe)	1079	

8. If Changes Occur in Capacity Ratings (items 3 through 7) since Last Report, Give Reason N/A

9. Power Level to Which Restricted, if any (Net MWe) N/A

10. Reasons for Restrictions, if any N/A

	This Month	Year to Date	Cumulative
11. Hours in Reporting Period	744	5831	71640
12. No. of Hrs. Reactor was Critical	744	5813.6	41637.1
13. Reactor Reserve Shutdown Hrs.	0	0	3088.4
14. Hours Generator On-Line	744	5810.7	39969.3
15. Unit Reserve Shutdown Hours	0	0	0
16. Gross Thermal Energy Generated (MWH)	2490518	19345903	122113900
17. Gross Elec. Energy Generated (MWH)	831840	6572710	40488760
18. Net Elec. Energy Generated (MWH)	798663	6316931	38414913
19. Unit Service Factor	100	99.7	55.8
20. Unit Availability Factor	100	99.7	55.8
21. Unit Capacity Factor (using MDC Net)	99.5	100.4	49.7
22. Unit Capacity Factor (using DER Net)	98.5	99.4	49.2
23. Unit Forced Outage Rate	0	.3	29.8

24. Shutdowns scheduled over next 6 months (type, date and duration of each)

N/A

25. If shutdown at end of Report Period, Estimated Date of Startup:

N/A

26. Units in Test Status (Prior to Commercial Operation):

	Forecast	Achieved
Initial Criticality	9/30/76	12/11/76
Initial Electricity	11/1/76	12/25/76
Commercial Operation	12/20/76	6/30/77

UNIT SHUTDOWN AND POWER REDUCTIONS
REPORT MONTH August 1985

Docket No. 50-272
Unit Name Salem No.1
Date September 10,1985
Telephone 609-935-6000
Extension 4455

Completed by J.P. Ronafalvy

No.	Date	Type 1	Duration Hours	Reason 2	Method of Shutting Down Reactor	License Event Report	System Code 4	Component Code 5	Cause and Corrective Action to Prevent Recurrence

* There were no outages for the month of August resulting in a load reduction of greater than 20%.

1
F: Forced
S: Scheduled

2 Reason
A-Equipment Failure-explain
B-Maintenance or Test
C-Refueling
D-Regulatory Restriction
E-Operator Training & Licensing Exam
F-Administrative
G-Operational Error-explain
H-Other-explain

3 Method
1-Manual
2-Manual Scram.
3-Automatic Scram.
4-Continuation of Previous Outage
5-Load Reduction
9-Other

4 Exhibit G Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG 0161)

5 Exhibit 1 Salem as Source

MAJOR PLANT MODIFICATIONS
REPORT MONTH August 1985

DOCKET NO.: 50-272
UNIT NAME: Salem 1
DATE: September 10, 1985
COMPLETED BY: J. Ronafalvy
TELEPHONE: 609/339-4455

*DCR NO.	PRINCIPLE SYSTEM	SUBJECT
1EC-0675	Fuel Handling Cranes	Revise limit switch protection arrangement as necessary to comply with intent of No. 2 Unit Outstanding Item No. 047043.
1EC-1591	NPDES Thermal Monitoring System	Relocate circ. water inlet and outlet temperature monitors from their present location at the condenser inlet and outlet waterboxes to the circ. water intake structure for measuring inlet ambient river water temperature and to the six outlet pipes for measuring combined plant discharge temperature.
1EC-2025	Fire Protection-Doors	Addition of three fire rated doors. Two doors are located in the Aux Building and Mechanical/Electrical Penetration Areas, third door is located in the Service Water Intake Structure. In addition, door #379 is to be removed. All three doors rated for 3 hours (Class A).
1EC-2050A	Communication	New raceways for upgrading telephone and data transmission from Service Building. Conduit must be provided to house 300-pair cable of telephone twister copper cable.
1EC-2069	Radiation Monitoring	Install field modification kits for waterproofing the R13 radiation monitors.

* Design Change Request

MAJOR PLANT MODIFICATIONS
REPORT MONTH AUGUST 1985

DOCKET NO.: 50-272
UNIT NAME: Salem 1
DATE: September 10, 1985
COMPLETED BY: J. Ronafalvy
TELEPHONE: 609/339-4455

*DCR NO.	SAFETY EVALUATION 10 CFR 50.59
1EC-0675	The installation of the new upper limit switch and other hardware in the Fuel Handling Crane will not affect the safe shutdown of the reactor nor will it create any new fire hazard. No changes to the FSAR or the Tech. Specs. are involved. Implementation of this DCP will comply with NUREG-0612 requirements for upgrading the cranes at operating nuclear power plants. No unreviewed safety or environmental questions are involved.
1EC-1591	This DCP involves moving the location of the circulating water system temperature monitors and as such does not involve a change to the FSAR or the Tech. Specs. No unreviewed safety or environmental questions ae involved.
1EC-2025	The new fire doors are not safety related. Their presence will not impact the operation of existing safety related equipment nor will they reduce the margin of safety related equipment as outlined in the Tech. Specs. Since the fire doors are relatively passive and "inert", a failure of the doors will not result in any damage or influence the operation of safety related equipment. The new fire doors will reduce the risk of fire damage to safety related equipment. No unreviewed safety or environmental questions are involved.
1EC-2050A	This DCP involves the installation of telephone conduit between the Administration Building and the Service Building. No work is being performed in safety related areas of the plant. This DCP does not impinge or interfere with any safety related system. This package does not affect the basis for the Tech. Specs. or the FSAR from an electrical standpoint. No unreviewed safety or environmental questions are involved.
1EC-2069	This design change enhances the operation of the five R-13 radiation monitors which are not safety related. This modification will not alter, affect or compromise the safety related components and systems in the plant. No unreviewed safety or environmental questions are involved.

*DCR - Design Change Request

PSE&G SALEM GENERATING STATION
SAFETY RELATED WORK ORDER LOG

SALEM UNIT 1

WO NO	DEPT	UNIT	EQUIPMENT IDENTIFICATION	
8508180021				
	SIC	1	DORIC 577/P250 T2609A	
			FAILURE DESCRIPTION:	13 AUX. FEED PUMP TURBINE INBOARD BEARING TEMP. INDICATION IS FAILED.
			CORRECTIVE ACTION:	REMOVED FIELD LEADS FROM DORIC AND P-250. PLACED JUMPERS ON APPROPRIATE POINTS P-250 READ 8.0.7 AND DORIC READ 82 DEGREES F. RECONNECTED FIELD LEADS.
8508060572				
	SIC	1	13 S/G FWR FLO CH 2	
			FAILURE DESCRIPTION:	COMPARATOR TRIPPING OUT OF SPEC., REWORK AS NECESSARY, THEN COMPLETE CHANNEL FUNCTIONAL.
			CORRECTIVE ACTION:	COMP. 1FC531 #1B OUTPUT OUT OF SPEC FOUND CAPS LEAKING C2 & C3 REG/MED ALSO REGISTOR. INSTALLED DCR 1EC0575 R39.
8508060050				
	SIC	1	1 SW 308	
			FAILURE DESCRIPTION:	THE MOORE CONTROLLER FOR THIS VALVE CAUSES THE VALVE TO GO OPEN WHEN PUT IN THE SEAL POSITION.
			CORRECTIVE ACTION:	REMOVED POSITIONER AND DISCONNECTED AIR SUPPLY AND PROCESS IMPACT LINE TO 1SW308. 8/7/85 REINSTALLED POSITIONER. 8/7/85 VERIFIED CONTROLLER OPERATION.

WO NO	DEPT	UNIT	EQUIPMENT IDENTIFICATION	
8508030177	SMD	1	14AF920	
			FAILURE DESCRIPTION:	THE 14AF920 (CHECK) VALVE IS LEAKING THRU, ALLOWING FLOW BACK FROM 14 S/G TO THE AFST.
			CORRECTIVE ACTION:	REMOVED BONNET, CLEANED INTERNALS, REPLACED BONNET
8508010036	SIC	1	RX COOL FLO LOOP 12 CH1	
			FAILURE DESCRIPTION:	COMPARATOR IS TRIPPING OUT OF SPEC.
			CORRECTIVE ACTION:	REPLACED COMPARATOR SINAO-997 WITH SINAO-962. PERFORMED "AS LEFT" OF PAGE 9, STEPS 8.8.1.1. AND ILLUMINATED. I&C SAYS THE PROBLEM IS IN THE CONSOLE, NO COMPARATOR IS TRIPPED TO GENERATE THE ALARM.
8507301797	SMD	1	11GB4	
			FAILURE DESCRIPTION:	LEAKING AT VALVE BODY.
			CORRECTIVE ACTION:	TEMPERARILY REPAIRED VALVE BY FURMANITE. NEW WORK ORDER TO BE WRITTEN FOR PERMANENT REPAIR.
8507260357	SOD	1	2" RECIRC HEADER	
			FAILURE DESCRIPTION:	CRUD BUILDUP IN S.F. PIT DEMINERALIZER RECIRC. HEADER WHICH IS CAUSING A HIGH RAD AREA AND IS GETTING LARGER. WOULD LIKE 2" HEADER FLUSHED BY UTILIZING SF29 AND SF37 VIA REFUEL. WTR. PURIF PUMP. RECIRC LINE IN QUESTION HAS SF60 AND 61 FLOW GAGE TAPS.
			CORRECTIVE ACTION:	READINGS BEFORE STARTED 1.2R/HR SLUICED RESINS AND FLUSHED READINGS AFTER .2MR/HR RESTORED LINEUP TO NORMAL.

WO NO	DEPT	UNIT	EQUIPMENT IDENTIFICATION
8505130472	SMD	1	13CC197
			FAILURE DESCRIPTION: THE VALVE HANDWHEEL IS BROKEN.
			CORRECTIVE ACTION: REPLACED STRIPPED OUT HANDWHEEL.
0099178737	SMD	1	16 SERVICE WATER PUMP
			FAILURE DESCRIPTION: THE PUMP PACKING RING IS BLOWN.
			CORRECTIVE ACTION: TIGHTENED TUBE TENSION AND REPLACED SEAL RING UNDER LOCK NUT.
0099178079	SIC	1	11WHUT LEVEL INDICATION
			FAILURE DESCRIPTION: LEVEL INDICATION FOR 11 WHUT IS NOT WORKING PROPERLY. TANK OVERFLOWED WITH LEVEL INDICATING 82%. NO HIGH LEVEL ALARM WAS RECEIVED.
			CORRECTIVE ACTION: CAL. RECEIVER GAUGE AND ALARM HI/LO LIS 1008 CAL. XMTR 1LT 1008 AND SIMULATED INPUT TO TRANSMITTER AND VERIFIED CORRECTED READINGS AND ALARM POINTS ON RECEIVER GAUGE ON PNL 104 AND PNL 107 - ALSO BLEWDOWN INSTRUMENT LINE FROM TRANSMITTER TO #11 WHUT
0099178095	SIC	1	DORIC POINT 205
			FAILURE DESCRIPTION: THE DORIC POINT IS FAILED.
			CORRECTIVE ACTION: FOUND BROKEN LEAD TO TERMINAL 205 ON DORIC SATELLITE IN UNIT 1 RACK AREA. REPAIRED LEAD AND RECONNECTED TO TERMINAL BOARD. POINT NOW READING ON DORIC SCANNER.

WO NO	DEPT	UNIT	EQUIPMENT IDENTIFICATION
0099176637	SMD	1	11AF8
			FAILURE DESCRIPTION: THIS VALVE LEAKS THRU CAUSING THE AFWST TO OVERFLOW.
			CORRECTIVE ACTION: OPEN AND INSPECTED VALVE, CLEANED VALVE INTERNALS - REASSEMBLED W/NEW GASKETS.
0099165538	SIC	1	LOOP 12 CH I RX COOL FLOW
			FAILURE DESCRIPTION: CH I RX. COOL. FLOW IS INDICATING THREE % HIGHER THAN OTHER CHANNELS.
			CORRECTIVE ACTION: REPLACED TRANSMITTER, BENCH CALIBRATED NEW TRANSMITTER AND PERFORMED TIME RESPONSE TEST PROCEDURE 1PD-2.8.006.
0099165392	SIC	1	PLANT VENT RMS MONITOR
			FAILURE DESCRIPTION: PLEASE REPAIR "+" L.E.D. ON REMOTE DISPLAY MODULE. PRESENTLY NOT VISIBLE.
			CORRECTIVE ACTION: REPLACED L.E.D. I.C. FOR "+" INDICATION ON REMOTE DISPLAY 8/12/85.
0099165121	SIC	1	POWER RANGE CHAN.
			FAILURE DESCRIPTION: NEGATIVE RATE TRIP INOPERABLE. FOUND DURING CHANNEL FUNCT.
			CORRECTIVE ACTION: FOUND NEG. RATE BISTABLE CARD BAD AND AN UN-LANDED SHIELD WIRE. LANDED WIRE AND REPLACED BISTABLE CARD. FINISHED FUNCTIONAL SAT.

SALEM GENERATING STATION
MONTHLY OPERATING SUMMARY - UNIT NO. 1
AUGUST 1985

SALEM NO. 1

The Unit operated at full power for the entire period.

REFUELING INFORMATION

DOCKET NO.: 50-272
COMPLETED BY: J. Ronafalvy
UNIT NAME: Salem 1
DATE: September 10, 1985
TELEPHONE: 609/935-6000
EXTENSION: 4455

Month August 1985

1. Refueling information has changed from last month:
   YES _____ NO X

2. Scheduled date for next refueling: February 22, 1986

3. Scheduled date for restart following refueling: May 4,1986

4. A) Will Technical Specification changes or other license amendments be required?
   YES _____ NO _____
   NOT DETERMINED TO DATE _____

   B) Has the reload fuel design been reviewed by the Station Operating Review Committee?
   YES _____ NO X
   If no, when is it scheduled? January 1986

5. Scheduled date(s) for submitting proposed licensing action:
   January 1986 if required

6. Important licensing considerations associated with refueling:
   NONE

7. Number of Fuel Assemblies:
   A) Incore 193
   B) In Spent Fuel Storage 296

8. Present licensed spent fuel storage capacity: 1170

   Future spent fuel storage capacity: 1170

9. Date of last refueling that can be discharged to spent fuel pool assuming the present licensed capacity: September 2001

8-1-7.R4

PSE&G

Public Service Electric and Gas Company P.O. Box E Hancocks Bridge, New Jersey 08038

Salem Generating Station

September 10, 1985

Director, Office of Inspection and Enforcement
U.S. Nuclear Regulatory Commission
Washington, DC 20555

Dear Sir:

MONTHLY OPERATING REPORT
SALEM NO. 1
DOCKET NO. 50-272

In compliance with Section 6.9, Reporting Requirements for the Salem Technical Specifications, 10 copies of the following monthly operating reports for the month of August 1985 are being sent to you.

Average Daily Unit Power Level
Operating Data Report
Unit Shutdowns and Power Reductions
Major Plant Modification
Safety Related Work Orders
Operating Summary
Refueling Information

Sincerely yours,

J. M. Zupko, Jr.
General Manager - Salem Operations

JR:sbh
cc: Dr. Thomas E. Murley
Regional Administrator USNRC
Region I
631 Park Avenue
King of Prussia, PA 19406

Director, Office of Management
Information and Program Control
U.S. Nuclear Regulatory Commission
Washington, DC 20555

Enclosures
8-1-7.R4

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