

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1) SAN ONOFRE NUCLEAR GENERATING STATION, UNIT 2										DOCKET NUMBER (5) 0 5 0 0 0 3 6 1				PAGE (3) 1 OF 0 2		
TITLE (4) REACTOR POWER SURGE																
EVENT DATE (8)			LER NUMBER (6)				REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)						
MONTH	DAY	YEAR	YEAR	SEC. NUMBER	REV. NUMBER	MONTH	DAY	YEAR	FACILITY NAMES				DOCKET NUMBER(S)			
0 2	2 3	8 4	8 4	0 1 0	0 0	0 3	2 6	8 4					0 5 0 0 0 0 0 0			
OPERATING MODE (9)		THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: (Check one or more of the following) (11)														
1		20.402(b)				20.405(c)				50.73(a)(2)(iv)				73.71(b)		
POWER LEVEL (10)		1 0 0				20.405(a)(1)(i)				50.73(a)(2)(v)				73.71(c)		
		20.405(a)(1)(ii)				50.36(c)(1)				50.73(a)(2)(vii)				<input checked="" type="checkbox"/> OTHER (Specify in Abstract below and in Text, NRC Form 366A)		
		20.405(a)(1)(iii)				50.36(c)(2)				50.73(a)(2)(viii)(A)				Voluntary Report		
		20.405(a)(1)(iv)				50.73(a)(2)(i)				50.73(a)(2)(viii)(B)						
		20.405(a)(1)(v)				50.73(a)(2)(ii)				50.73(a)(2)(ix)						
						50.73(a)(2)(iii)										
LICENSEE CONTACT FOR THIS LER (12)																
NAME J. G. HAYNES, STATION MANAGER										TELEPHONE NUMBER 7 1 4 4 9 2 - 7 7 C 0						
COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)																
CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRC		CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRC						
SUPPLEMENTAL REPORT EXPECTED (14)																
<input type="checkbox"/> YES (If yes, complete EXPECTED SUBMISSION DATE)										<input checked="" type="checkbox"/> NO		EXPECTED SUBMISSION DATE (15)		MONTH	DAY	YEAR

Abstract (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

This submittal provides an informational Licensee Event Report describing an occurrence of a partial loss of extraction steam feedwater heating which resulted in a reactor power increase of 3%. On February 23, 1984, at approximately 2215, with Unit 2 in Mode 1 at 100% power, an adjustment to first point heater E036 was in progress. During this adjustment, the first point level controller spuriously cycled resulting in high levels in the first and second point heaters and closure of steam extraction block valves 2HY-8804 and 2HY-8806 causing first and second point heaters to be automatically isolated from the steam supply. Due to the resulting steam loss, colder feedwater flowed through the steam generators and the reactor experienced a power surge. Reactor power increased to 103% (3492 megawatts thermal). Prompt action was taken to reduce turbine power resulting in reactor power being reduced to 100% in less than 30 minutes.

This event is considered an isolated occurrence. No corrective actions are planned.

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LICENSEE EVENT REPORT (LER)
TEXT CONTINUATIONU.S. NUCLEAR REGULATORY COMMISSION
APPROVED OMB NO. 3150-0104
EXPIRES: 8/31/85

FACILITY NAME (1)

BOCKET NUMBER (2)

LER NUMBER (6)

PAGE (3)

SAN ONOFRE NUCLEAR GENERATING STATION,
UNIT 2YEAR SEQ. REV.
NUMBER NUMBER NUMBER

0 5 0 0 0 3 6 1 8 4 - 0 1 0 - 0 1 0 0 2 OF 0 2

TEXT (If more space is required, use additional NRC Form 366A's) (17)

This submittal provides an informational Licensee Event Report describing an occurrence involving a reactor power surge.

On February 23, 1984, at approximately 2215 with Unit 2 in Mode 1 at 100% power, a technician was raising the water level in first point heater E036 (EIIS component identifier HX). After first point level controller 2LC-3591 (EIIS component identifier LIC) was adjusted, extraction isolation valves 2HV-8804 and 2HV-8806 (EIIS component identifier SHV) received a high level extraction steam isolation valve closure signal, and the valves shut. The temperature of feedwater flowing through the steam generators decreased, resulting in a reduction in the Reactor Coolant System (EIIS system identifier AB) cold leg temperature, T_c , of 4°F, and caused reactor power to rise to about 103%. Operators noted that 2HV-8804 and 2HV-8806 had shut, and began to reduce turbine output to raise T_c . At 2223, on February 23, 1984, the steam extraction valves were reopened and the turbine load was returned to 100% power. The reactor operated at above 100% for less than 30 minutes.

The cause of this event was determined to be spurious cycling of the level controller for the first point heater, which resulted in extraction block valves 2HV-8804 and 2HV-8806 closing. This is considered to be an isolated occurrence and no further corrective actions are planned.

This power transient was within the design basis of the plant. Plant overpower trips are set at 107% rated power and there are no credible circumstances under which this event could have been more severe.

Southern California Edison Company

SCE

SAN ONOFRE NUCLEAR GENERATING STATION

P.O. BOX 128

SAN CLEMENTE, CALIFORNIA 92672

March 26, 1984

J. G. HAYNES
STATION MANAGER

U. S. Nuclear Regulatory Commission
Document Control Desk
Washington, D.C. 20555

Subject: Docket No. 50-361
30-Day Report
Licensee Event Report No. 84-010
San Onofre Nuclear Generating Station, Unit 2

APR
TDC

00001112 C08

LER
LER # 361-84010
EVENT DATE 840223
INPO RCVD DATE 840404 pec
NSAC RCVD DATE

This submittal provides an informational Licensee Event Report (LER) for an occurrence involving a partial loss of extraction steam feedwater heating which resulted in a reactor power increase of 3%. The health and safety of plant personnel or the public were not affected by this event.

If you require any additional information, please so advise.

Sincerely,

JG Haynes

Enclosure: LER No. 84-010

cc: A. E. Chaffee (USNRC Resident Inspector, Units 1, 2 and 3)
J. P. Stewart (USNRC Resident Inspector, Units 2 and 3)

J. B. Martin (Regional Administrator, NRC Region V)

Institute of Nuclear Power Operations (INPO)

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