

TENNESSEE VALLEY AUTHORITY
DIVISION OF NUCLEAR POWER
SEQUOYAH NUCLEAR PLANT

MONTHLY OPERATING REPORT
TO THE
NUCLEAR REGULATORY COMMISSION
June 1, 1985 - June 30, 1985

UNIT 1

DOCKET NUMBER 50-327
LICENSE NUMBER DPR-77

UNIT 2

DOCKET NUMBER 50-328
LICENSE NUMBER DPR-79

Submitted by:

P.R. Wallace

P. R. Wallace, Plant Manager

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Operations Summary

June 1985

The following summary describes the significant operational activities for the month of June. In support of this summary, a chronological log of significant events is included in this report.

Unit 1

Unit 1 returned to service at 2124 CDT on June 26, 1985. The reactor was critical for 110.25 hours, produced 82,330 MWH (gross), resulting in an hourly gross load of 834,990 kW during the month. The capacity factor for the month was 9.7 percent. There are 73.24 full power days estimated remaining until the end of cycle 3 fuel. The cycle 3 refueling/modification outage is scheduled to begin September 27, 1985.

During the month the unit experienced no reactor scrams, one power reduction, and no manual shutdowns.

Unit 2

Unit 2 was critical for 720.0 hours, produced 829,310 MWH (gross), resulting in an average hourly gross load of 1,151,819 kW during the month. The capacity factor for the month was 97.4 percent. There are 204.46 full power days estimated remaining until the end of cycle 3 fuel. With a capacity factor of 85 percent, the target EOC exposure would be reached February 26, 1986.

During the month, the unit experienced no reactor scrams, manual shutdowns or power reductions.

Significant Operational Events

Unit 1

<u>Date</u>	<u>Time</u>	<u>Event</u>
06/01/85	0001	The reactor was in mode 4. Reassembly and leak testing of the electrical generator continued.
	0012	The reactor entered mode 5 so the RCS could be drained to work on number 4 reactor coolant pump number one seal.
06/10/85	1915	The reactor entered mode 4.
06/12/85	0950	1-PCV-68-334 (PORV) opened due to COMS actuations. Recently revised setpoints were not provided to the operator prior to obtaining the setpoint limit.
	2136	The reactor entered mode 3.

Significant Operational Events (Cont.)

<u>Unit 1</u>		
<u>Date</u>	<u>Time</u>	<u>Event</u>
06/14/85	1423	The reactor entered mode 4.
	1921	The reactor entered mode 5.
		The number 1 and 3 reactor coolant pump flange seals required maintenance.
06/19/85	2040	The reactor entered mode 4.
06/20/85	1243	The reactor entered mode 3.
06/23/85	1120	The reactor entered mode 4.
	1635	The reactor entered mode 5 due to electrical problems with CRD J-13 connector in the head. The center missile shield was required to be lifted.
06/24/85	1025	The reactor entered mode 4.
06/25/85	0035	The reactor entered mode 3.
06/26/85	0945	The reactor was taken critical.
	1200	The reactor entered mode 1.
	1519	Completed the turbine trip test.
	2124	The unit was tied on-line. The duration of the ice weighing/maintenance outage was 74 days, 5 hours, and 37 minutes.
06/28/85	0930	The reactor was in mode 1 at 80% power and increasing when the condenser backpressure began increasing. Started a load reduction.
	1033	The reactor was at 50% power and holding due to condenser back pressure problems.
	1042	Began power ascension.
06/29/85	0155	The reactor obtained 100% power.
06/30/85	2359	The reactor was in mode 1 at 100% power, producing 1150 MWE.
<u>Unit 2</u>		
06/01/85	0001	The reactor was in mode 1 at 100% power, producing 1170 MWE.
06/30/85	2359	The reactor was in mode 1 at 100% power, producing 1150 MWE.

Spent Fuel Pit Storage Capabilities

The total storage capability in the spent fuel pit (SFP) is 1386. However, there are five cell locations which are not capable of storing spent fuel. Four locations (A10, A11, A24, A25) are unavailable due to a suction strainer conflict and one location (A16) is unavailable due to an instrumentation conflict. Presently, there is a total of 276 spent fuel bundles stored in the SFP. Thus, the remaining storage capacity is 1105.

There was no fuel movement within the reactor. However, the final three shipments of cycle 4 fuel (36 bundles) were received during the month. A total of 72 fuel assemblies have been received, inspected, and properly stored in the new fuel storage vault. There were no reported discrepancies. An inventory was performed on June 19, 1985 per TI-1 and there were no discrepancies.

Four (4) thimble plugs (PD-217, PD-180D, PD-126, PD-72) with suspected bent pins were shipped to Westinghouse Fabrication Plant, Columbia, S.C. on June 4, 1985 so that proper and professional repairs could be performed by qualified Westinghouse personnel. These repairs were made so that these thimble plugs could be used for future fuel cycles. TVA incurred no costs regarding repairs, shipping or handling.

The four (4) thimble plugs were returned to Sequoyah Nuclear Plant on June 19, 1985. After inspection, they were found to be satisfactory. They were properly stored in the new fuel storage vault.

PORVs and Safety Valves Summary

1-PCV-68-334 was challenged at 0950 on June 12, 1985, by actuation of the cold overpressure mitigation system (COMS). The COMS setpoints were changed but the revised data was not given to the unit operator prior to obtaining the new setpoint limit.

Licensee Events and Special Reports

The following licensee event reports (LER) were sent during June 1985 to the Nuclear Regulatory Commission.

<u>LER</u>	<u>DESCRIPTION OF EVENT</u>
1-85024	Starting at 1000 CDT on May 24, 1985, with unit 1 in mode 4, the hourly firewatch for the auxiliary building supply air fan room was not performed because door A-123 would not open. The hourly fire watch resumed at 1300 CDT.
1-85004 R2	<p>This LER was originally reported in the February 1985 Monthly Operating Report to the NRC. The LER concerns SI-261, "Visual Inspection of Fire Doors" not being performed within the technical specification time limits.</p> <p>On March 29, 1985, it was discovered that there were an additional sixteen (16) deficiencies which were inadvertently omitted from the original report and revision 1 was submitted. Revision 2 provides details and schedules for completion of the outstanding deficiencies.</p>

Licensee Events and Special Reports (Continued)

- 1-85019 A design review revealed that during a seismic event, a one-inch demineralized water pipe and a high pressure fire protection (HPFP) header could have failed. This failure could have caused water spray damage on motors and electrical control panels in the main control room air conditioning chiller systems. The demineralized water pipe has been isolated, and there is ample-time for repair if the HPFP header should fail.
- 1-85020 On May 14, 1985, with unit 1 in mode 5, both trains of residual heat removal (RHR) were inadvertently isolated by closure of the train B suction valve. The isolation occurred while work was being performed on the reactor vessel level instrumentation system (RVLIS) to refill the sense lines. The RCS wide range pressure transmitter 1-PT-68-66, which is used for RHR overpressure protection, receives its process signal from the RVLIS sense lines and was increased approximately 2000 psi during testing (RHR isolation is at 700 psi increasing). The suction was reestablished within approximately 16 minutes and there was no indicated change in RCS temperature.
- 1-85021 An inadvertent main control room (MCR) ventilation isolation occurred at 0705 CDT on May 23, 1985, while unit 1 was in mode 4 and unit 2 was in mode 3. Radiation monitor RM-90-125 was found to have a bad power supply. The power supply was replaced, the monitor functionally tested and returned to service.
- 1-85023 This report details four inadvertent auxiliary building ventilation isolation (ABIs).
- The first event occurred at 1601C on May 30, 1985 with unit 1 in mode 4 and unit 2 was in mode 1 at 100% power. The ABI was caused by electromagnetic interference (EMI) on spent fuel pool (SFP) radiation monitor RM-90-102.
- The second event occurred at 0417C on May 31, 1985, when the SFP monitor RM-90-101 high radiation alarm actuated. The spikes were probably caused by EMI generated by a combination of operation of the motor operated disconnects (MODs) switches in the switchyard and the replacement of filter paper in the monitor.
- The third event occurred at 0449C on May 31, 1985, while the ventilation system was being realigned after the previous event. RM-90-101A spiked on particulate with the magnitude of the spike of 2000 cpm. The set point was 1340 cpm. The particulate set point has been reevaluated per TJ-18 and raised to 11,040 cpm based on Colbolt-50 instead of Iodine 131.
- The fourth event occurred on June 7, 1985, while electrical maintenance personnel were working on a plug mold inside a cabinet in the main control room. The personnel inadvertently bumped a high radiation time delay relay.

LERDESCRIPTION OF EVENT (Continued)

1-85023 (Continued)

The bump and subsequent vibration from working on the plug mold caused the relay to fall out of its socket causing an ABI on the SFP radiation monitor RM-90-102. The relay is a plug-in type which deenergize to trip.

1-85015 With unit 1 in mode 5 and unit 2 in mode 1 at 100% power, the service building radiation monitor 0-RM-90-132 was declared inoperable at 1010C on May 28, 1985. The STS requires a noble gas sample to be taken every eight (8) hours while the monitor is inoperable for up to 30 days or all releases must be suspended. On June 1, 1985, the 1810C sample was not taken until 2055C.

2-85010 On May 22, 1985, at 1329C with unit 1 in mode 4 and unit 2 at 100% power, unit 2 received a reactor trip from the reactor coolant system (RCS) overpower/differential temperature trip function. The trip was the result of a personnel error while collecting data from the RCS hot and cold leg temperature loops.

Special Reports

No special reports were transmitted during the month.

Diesel Generator Failure Report

There were no diesel generator failure reports transmitted during the month.

Offsite Dose Calculation Manual Changes

No changes were made to the Sequoyah Offsite Dose Calculation Manual during the month.

OPERATING DATA REPORT

DOCKET NO. 50-327
DATE JULY 2 1985
COMPLETED BY M G EDDINGS
TELEPHONE (615)870-6421

OPERATING STATUS

1. UNIT NAME: SEQUOYAH NUCLEAR PLANT, UNIT 1
2. REPORT PERIOD: JUNE 1985
3. LICENSED THERMAL POWER(MWT): 3411.0
4. NAMEPLATE RATING (GROSS MWE): 1220.6
5. DESIGN ELECTRICAL RATING (NET MWE): 1148.0
6. MAXIMUM DEPENDABLE CAPACITY (GROSS MWE): 1183.0
7. MAXIMUM DEPENDABLE CAPACITY (NET MWE): 1148.0
8. IF CHANGES OCCUR IN CAPACITY RATINGS(ITEMS NUMBERS 3 THROUGH 7)SINCE LAST REPORT, GIVE REASONS:-----
9. POWER LEVEL TO WHICH RESTRICTED, IF ANY(NET MWE):-----
10. REASONS FOR RESTRICTIONS, IF ANY:-----

NOTES:

THIS MONTH YR.-TO-DATE CUMULATIVE

11. HOURS IN REPORTING PERIOD	720.00	4343.00	35064.00
12. NUMBER OF HOURS REACTOR WAS CRITICAL	110.25	2577.35	23225.01
13. REACTOR RESERVE SHUTDOWN HOURS	0.00	0.00	0.00
14. HOURS GENERATOR ON-LINE	98.60	2563.70	22672.65
15. UNIT RESERVE SHUTDOWN HOURS	0.00	0.00	0.00
16. GROSS THERMAL ENERGY GENERATED (MWH)	257388.60	8397224.36	73074910.31
17. GROSS ELECTRICAL ENERGY GEN. (MWH)	82330.00	2900740.00	24637156.00
18. NET ELECTRICAL ENERGY GENERATED (MWH)	74759.00	2777899.00	23659529.00
19. UNIT SERVICE FACTOR	13.69	59.03	64.66
20. UNIT AVAILABILITY FACTOR	13.69	59.03	64.66
21. UNIT CAPACITY FACTOR(USING MDC NET)	9.04	55.72	58.78
22. UNIT CAPACITY FACTOR(USING DER NET)	9.04	55.72	58.78
23. UNIT FORCED OUTAGE RATE	79.66	13.09	17.77
24. SHUTDOWNS SCHEDULED OVER NEXT 6 MONTHS (TYPE, DATE, AND DURATION OF EACH):	<u>Cycle 3 Refueling/Modification - September 27, 1985 - 51 Days</u>		

25. IF SHUTDOWN AT END OF REPORT PERIOD, ESTIMATED DATE OF STARTUP:-----

NOTE THAT THE THE YR.-TO-DATE AND CUMULATIVE VALUES HAVE BEEN UPDATED.

OPERATING DATA REPORT

DOCKET NO. 50-328
DATE JULY 2 1985
COMPLETED BY D.C.DUPREE
TELEPHONE (615)870-6933

OPERATING STATUS

1. UNIT NAME: SEQUOYAH NUCLEAR PLANT, UNIT 2
2. REPORT PERIOD: JUNE 1985
3. LICENSED THERMAL POWER(MWT): 3411.0
4. NAMEPLATE RATING (GROSS MWE): 1220.6
5. DESIGN ELECTRICAL RATING (NET MWE): 1148.0
6. MAXIMUM DEPENDABLE CAPACITY (GROSS MWE): 1183.0
7. MAXIMUM DEPENDABLE CAPACITY (NET MWE): 1148.0
8. IF CHANGES OCCUR IN CAPACITY RATINGS(ITEMS NUMBERS 3 THROUGH 7)SINCE LAST REPORT, GIVE REASONS:-----

9. POWER LEVEL TO WHICH RESTRICTED, IF ANY(NET MWE):-----

10. REASONS FOR RESTRICTIONS, IF ANY:-----

NOTES:

	THIS MONTH	YR.-TO-DATE	CUMULATIVE
--	------------	-------------	------------

11. HOURS IN REPORTING PERIOD	720.00	4343.00	27024.00
12. NUMBER OF HOURS REACTOR WAS CRITICAL	720.00	4045.22	20740.34
13. REACTOR RESERVE SHUTDOWN HOURS	0.00	0.00	0.00
14. HOURS GENERATOR ON-LINE	720.00	3980.17	20250.35
15. UNIT RESERVE SHUTDOWN HOURS	0.00	0.00	0.00
16. GROSS THERMAL ENERGY GENERATED (MWH)	2444832.45	12900202.35	64899213.22
17. GROSS ELECTRICAL ENERGY GEN. (MWH)	829310.00	4424850.00	22116530.00
18. NET ELECTRICAL ENERGY GENERATED (MWH)	796418.00	4255037.00	21276045.60
19. UNIT SERVICE FACTOR	100.00	91.65	74.93
20. UNIT AVAILABILITY FACTOR	100.00	91.65	74.93
21. UNIT CAPACITY FACTOR(USING MDC NET)	96.35	85.34	68.58
22. UNIT CAPACITY FACTOR(USING DER NET)	96.35	85.34	68.58
23. UNIT FORCED OUTAGE RATE	0.00	8.22	8.52
24. SHUTDOWNS SCHEDULED OVER NEXT 6 MONTHS (TYPE, DATE, AND DURATION OF EACH):	----- ----- -----		

25. IF SHUTDOWN AT END OF REPORT PERIOD, ESTIMATED DATE OF STARTUP:-----

NOTE THAT THE THE YR.-TO-DATE AND CUMULATIVE VALUES HAVE BEEN UPDATED.

UNIT SHUTDOWNS AND POWER REDUCTIONS

DOCKET NO. 50-327

UNIT NAME Sequoyah One

DATE July 2, 1985

COMPLETED BY M. G. Eddings

TELEPHONE (615) 870-6421

REPORT MONTH JUNE

No.	Date	Type ¹	Duration (Hours)	Reason ²	Method of Shutting Down Reactor ³	Licensee Event Report #	System Code ⁴	Component Code ⁵	Cause & Corrective Action to Prevent Recurrence
2	850413	S	235.25	H	4				Electrical Generator Maintenance Outage
3	850610	F	386.15	A	9				Reactor Coolant Pump Seal Housing leak repairs and feedwater valve maintenance.

1

F: Forced
S: Scheduled

2

Reason:
A-Equipment Failure (Explain)
B-Maintenance or Test
C-Refueling
D-Regulatory Restriction
E-Operator Training & License Examination
F-Administrative
G-Operational Error (Explain)
H-Other (Explain)

3

Method:
1-Manual
2-Manual Scram.
3-Automatic Scram.
4-Cont. of Existing
Outage
5-Reduction
9-Other

4

Exhibit G-Instructions
for Preparation of Data
Entry Sheets for Licensee
Event Report (LER) File (NUREG-
0161)

5

Exhibit I-Same Source

(9/77)

UNIT SHUTDOWNS AND POWER REDUCTIONS

DOCKET NO. 50-328
UNIT NAME Sequoyah Two
DATE July 2, 1985
COMPLETED BY D. C. Dupree
TELEPHONE (615) 870-6933

REPORT MONTH JUNE

No.	Date	Type ¹	Duration (Hours)	Reason ²	Method of Shutting Down Reactor ³	Licensee Event Report #	System Code ⁴	Component Code ⁵	Cause & Corrective Action to Prevent Recurrence
						NONE			

1

F: Forced
S: Scheduled

2

Reason:

A-Equipment Failure (Explain)
B-Maintenance or Test
C-Refueling
D-Regulatory Restriction
E-Operator Training & License Examination
F-Administrative
G-Operational Error (Explain)
H-Other (Explain)

3

Method:

1-Manual
2-Manual Scram.
3-Automatic Scram.
4-Cont. of Existing
Outage
5-Reduction
9-Other

4

Exhibit G-Instructions
for Preparation of Data
Entry Sheets for Licensee
Event Report (LER) File (NUREG-
0161)

5

Exhibit I-Same Source

(9/77)

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ATTACHMENT 1
AVERAGE DAILY UNIT POWER LEVEL

FILE PACKAGE NO. 55
REPORT REQUIREMENTS

DOCKET NO. 50-327
UNIT 1
DATE Jul. 1, 1985
COMPLETED BY M. G. Eddings
TELEPHONE (615) 870-6421

MONTH <u>June</u>			
Day	AVERAGE DAILY POWER LEVEL (MWe-Net)	DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
1	NA	17	NA
2	NA	18	NA
3	NA	19	NA
4	NA	20	NA
5	NA	21	NA
6	NA	22	NA
7	NA	23	NA
8	NA	24	NA
9	NA	25	NA
10	NA	26	11
11	NA	27	262
12	NA	28	890
13	NA	29	1111
14	NA	30	1116
15	NA	31	NA
16	NA		

INSTRUCTIONS

On this format, list the average daily unit power level in MWe-Net for each day in the reporting month. Compute to the nearest whole megawatt.

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ATTACHMENT 1
AVERAGE DAILY UNIT POWER LEVEL

FILE PACKAGE NO. 55
REPORT REQUIREMENTS

DOCKET NO. 50-328
UNIT 2
DATE July 2, 1985
COMPLETED BY D. C. Dupree
TELEPHONE (615) 870-6933

MONTH <u>June</u>			
Day	AVERAGE DAILY POWER LEVEL (MWe-Net)	DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
1	<u>1132</u>	17	<u>1112</u>
2	<u>1133</u>	18	<u>1111</u>
3	<u>1129</u>	19	<u>1110</u>
4	<u>1128</u>	20	<u>1110</u>
5	<u>1126</u>	21	<u>1109</u>
6	<u>1124</u>	22	<u>1109</u>
7	<u>1123</u>	23	<u>1108</u>
8	<u>1114</u>	24	<u>1110</u>
9	<u>1113</u>	25	<u>1106</u>
10	<u>1113</u>	26	<u>1103</u>
11	<u>1115</u>	27	<u>1104</u>
12	<u>1113</u>	28	<u>1106</u>
13	<u>1109</u>	29	<u>1105</u>
14	<u>1111</u>	30	<u>1105</u>
15	<u>1111</u>	31	<u>NA</u>
16	<u>1114</u>		

INSTRUCTIONS

On this format, list the average daily unit power level in MWe-Net for each day in the reporting month. Compute to the nearest whole megawatt.

NUCLEAR PLANT OPERATING STATISTICS

SEQUOYAH

Plant

Period Hours 720

Month JUNE 19 85

Item No.	Unit No.	UNIT ONE	UNIT TWO	PLANT				
Generation	1	Average Hourly Gross Load, kW	834,990	1,151,819	1,113,657			
	2	Maximum Hour Net Generation, MWh	1,124	1,140	2,232			
	3	Core Thermal Energy Gen, GWD (t) ²	10.7245	101.8680	112.5925			
	4	Steam Gen. Thermal Energy Gen., GWD (t) ²	10.7621	102.3239	113.0860			
	5	Gross Electrical Gen., MWh	82,330	829,310	911,640			
	6	Station Use, MWh	7,571	32,892	40,463			
	7	Net Electrical Gen., MWh	74,759	796,418	871,177			
	8	Station Use, Percent	9.20	3.97	4.44			
	9	Accum. Core Avg. Exposure, MWD/Ton ¹	11,385	6,114	17,499			
	10	CTEG This Month, 10 ⁶ BTU	878,467	8,344,213	9,222,680			
	11	SGTEG This Month, 10 ⁶ BTU	881,543	8,381,554	9,263,097			
	12							
Factors & Use	13	Hours Reactor Was Critical	110.25	720.0	830.25			
	14	Unit Use, Hours-Min.	98:36	720:00	818:36			
	15	Capacity Factor, Percent	9.7	97.4	53.5			
	16	Turbine Avail. Factor, Percent	13.7	100.0	56.8			
	17	Generator Avail. Factor, Percent	13.7	100.0	56.8			
	18	Turbogen. Avail. Factor, Percent	13.7	100.0	56.8			
	19	Reactor Avail. Factor, Percent	15.3	100.0	57.7			
	20	Unit Avail. Factor, Percent	13.7	100.0	56.8			
	21	Turbine Startups	1	0	1			
	22	Reactor Cold Startups	1	0	1			
	23	PERIOD HOURS			720			
Efficiency	24	Gross Heat Rate, Btu/kWh	10,670	10,060	10,120			
	25	Net Heat Rate, Btu/kWh	11,750	10,480	10,590			
	26	GHR (w.o. oil)			10,120			
	27	NHR (w.o. oil)			10,590			
Temp & Press	28	Throttle Pressure, psig	901.8	866.3	884.1			
	29	Throttle Temperature, °F	531.9	527.46	529.7			
	30	Exhaust Pressure, InHg Abs.	3.24	3.2	3.22			
	31	Intake Water Temp., °F	77.3	75.7	76.5			
	32							
Flows	33	Main Feedwater, M lb/hr	10.6	15.0	12.8			
	34							
	35							
	36							
Misc.	37	Full Power Capacity, EFPD	370.00	363.65	733.65			
	38	Accum. Cycle Full Power Days, EFPD	296.7608	159.1856	455.9464			
	39	Oil Fired for Generation, Gallons			1,518			
	40	Oil Heating Value, Btu/Gal.			138,000			
	41	Diesel Generation, MWh			23			
	42							
Station Data	43	Max. Hour Net Gen.			Max. Day Net Gen.		Load Factor, %	<div></div>
		MWh	Time	Date	MWh	Date		
		2232	1200	6/30/85	53,304	6/30/85		
	Remarks: ¹ For BFNP this value is MWD/STU and for SQNP and WBNP this value is MWD/MTU.							
	² (t) indicates Thermal Energy.							

Date Submitted

Date Revised

P.R. Waller

Plant Superintendent

TVA 6560C (PP-3-76)

UNIT OUTAGE AND AVAILABILITY

SEQUOYAH Nuclear Plant

Licensed Reactor Power 3411 MW(th)Unit No. ONEGenerator Rating 1220.5 MW(e)Month/Year JUNE 1985Design Gross Electrical Rating 1183 MWPeriod Hours 720

Day	Time Unit Available						Time Not Available								Unit				OUTAGE CAUSE	METHOD OF SHUTTING DOWN REACTOR	UNIT STATUS DURING OUTAGE	CORRECTIVE ACTION TAKEN TO PREVENT REPETITION
	Total		Gen.		Not Used		Turbine		Gen.		Reactor		Unit		Time Out		Time In					
	Hrs	Min	Hrs	Min	Hrs	Min	Hrs	Min	Hrs	Min	Hrs	Min	Hrs	Min	Hrs	Min	Hrs	Min				
1	00	00	00	00			24	00	24	00	24	00	24	00								
2	00	00	00	00			24	00	24	00	24	00	24	00								
3	00	00	00	00			24	00	24	00	24	00	24	00								
4	00	00	00	00			24	00	24	00	24	00	24	00								
5	00	00	00	00			24	00	24	00	24	00	24	00								
6	00	00	00	00			24	00	24	00	24	00	24	00								
7	00	00	00	00			24	00	24	00	24	00	24	00								
8	00	00	00	00			24	00	24	00	24	00	24	00								
9	00	00	00	00			24	00	24	00	24	00	24	00								
10	00	00	00	00			24	00	24	00	24	00	24	00								
11	00	00	00	00			24	00	24	00	24	00	24	00								
12	00	00	00	00			24	00	24	00	24	00	24	00								
13	00	00	00	00			24	00	24	00	24	00	24	00								
14	00	00	00	00			24	00	24	00	24	00	24	00								
15	00	00	00	00			24	00	24	00	24	00	24	00								
16	00	00	00	00			24	00	24	00	24	00	24	00								
17	00	00	00	00			24	00	24	00	24	00	24	00								
18	00	00	00	00			24	00	24	00	24	00	24	00								
19	00	00	00	00			24	00	24	00	24	00	24	00								
20	00	00	00	00			24	00	24	00	24	00	24	00								
21	00	00	00	00			24	00	24	00	24	00	24	00								
22	00	00	00	00			24	00	24	00	24	00	24	00								
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26	02	36	02	36			21	24	21	24	09	45	21	24			21	24				
27	24	00	24	00			00	00	00	00	00	00	00	00								
28	24	00	24	00			00	00	00	00	00	00	00	00								
29	24	00	24	00			00	00	00	00	00	00	00	00								
30	24	00	24	00			00	00	00	00	00	00	00	00								
31																						
Total	98	36	98	36			621	24	621	24	609	45	621	24								

UNIT OUTAGE AND AVAILABILITY

SEQUOYAH

Nuclear Plant

Licensed Reactor Power 3411 MW(th)

Unit No. TWO

Generator Rating 1220.5 MW(e)

Month/Year JUNE 1985

Design Gross Electrical Rating 1183 MW

Period Hours 720

Day	Time Unit Available						Time Not Available								Unit				OUTAGE CAUSE	METHOD OF SHUTTING DOWN REACTOR	UNIT STATUS DURING OUTAGE	CORRECTIVE ACTION TAKEN TO PREVENT REPETITION
	Total		Gen.		Not Used		Turbine		Gen.		Reactor		Unit		Time Out		Time In					
	Hrs	Min	Hrs	Min	Hrs	Min	Hrs	Min	Hrs	Min	Hrs	Min	Hrs	Min	Hrs	Min	Hrs	Min				
1	24	00	24	00			00	00	00	00	00	00	00	00								
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3	24	00	24	00			00	00	00	00	00	00	00	00								
4	24	00	24	00			00	00	00	00	00	00	00	00								
5	24	00	24	00			00	00	00	00	00	00	00	00								
6	24	00	24	00			00	00	00	00	00	00	00	00								
7	24	00	24	00			00	00	00	00	00	00	00	00								
8	24	00	24	00			00	00	00	00	00	00	00	00								
9	24	00	24	00			00	00	00	00	00	00	00	00								
10	24	00	24	00			00	00	00	00	00	00	00	00								
11	24	00	24	00			00	00	00	00	00	00	00	00								
12	24	00	24	00			00	00	00	00	00	00	00	00								
13	24	00	24	00			00	00	00	00	00	00	00	00								
14	24	00	24	00			00	00	00	00	00	00	00	00								
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30	24	00	24	00			00	00	00	00	00	00	00	00								
31	24	00	24	00			00	00	00	00	00	00	00	00								
Total	720	00	720	00																		

UNIT OUTAGE AND AVAILABILITY

SEQUOYAH

Nuclear Plant

Licensed Reactor Power 3411 MW(th)Unit No. ONEGenerator Rating 1220.5 MW(e)Month/Year JUNE 1985Design Gross Electrical Rating 1183 MWPeriod Hours 720

Day	Time Unit Available						Time Not Available								Unit				OUTAGE CAUSE	METHOD OF SHUTTING DOWN REACTOR	UNIT STATUS DURING OUTAGE	CORRECTIVE ACTION TAKEN TO PREVENT REPETITION
	Total		Gen.		Not Used		Turbine		Gen.		Reactor		Unit		Time Out		Time In					
	Hrs	Min	Hrs	Min	Hrs	Min	Hrs	Min	Hrs	Min	Hrs	Min	Hrs	Min	Hrs	Min	Hrs	Min				
1	00	00	00	00			24	00	24	00	24	00	24	00								
2	00	00	00	00			24	00	24	00	24	00	24	00								
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4	00	00	00	00			24	00	24	00	24	00	24	00								
5	00	00	00	00			24	00	24	00	24	00	24	00								
6	00	00	00	00			24	00	24	00	24	00	24	00								
7	00	00	00	00			24	00	24	00	24	00	24	00								
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9	00	00	00	00			24	00	24	00	24	00	24	00								
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26	02	36	02	36			21	24	21	24	09	45	21	24		21	24					
27	24	00	24	00			00	00	00	00	00	00	00	00								
28	24	00	24	00			00	00	00	00	00	00	00	00								
29	24	00	24	00			00	00	00	00	00	00	00	00								
30	24	00	24	00			00	00	00	00	00	00	00	00								
31																						
Total	98	36	98	36			621	24	621	24	609	45	621	24								

TVA 6560C (PP-3-76)

UNIT OUTAGE AND AVAILABILITY

SEQUOYAH

Nuclear Plant

Licensed Reactor Power 3411 MW(th)

Unit No. TWO

Generator Rating 1220.5 MW(e)

Month/Year JUNE 1985

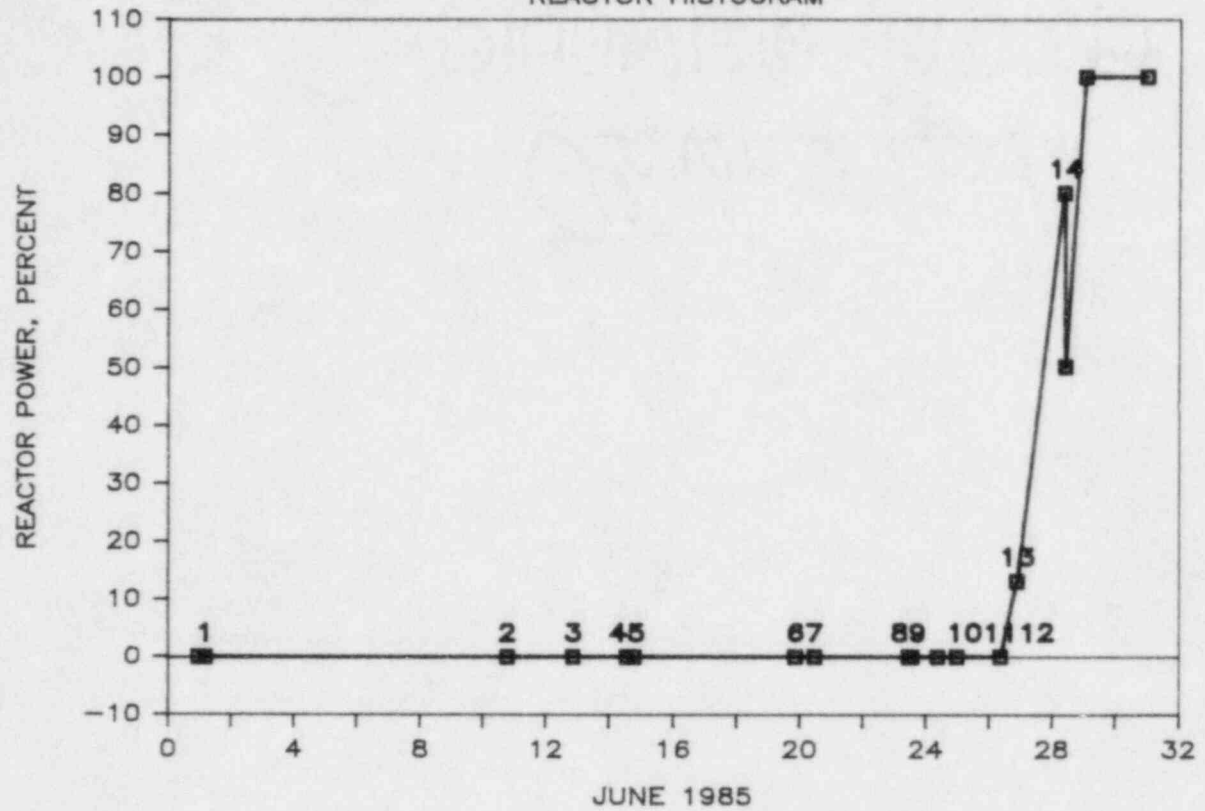
Design Gross Electrical Rating 1183 MW

Period Hours 720

Day	Time Unit Available						Time Not Available								Unit				OUTAGE CAUSE	METHOD OF SHUTTING DOWN REACTOR	UNIT STATUS DURING OUTAGE	CORRECTIVE ACTION TAKEN TO PREVENT REPETITION
	Total		Gen.		Not Used		Turbine		Gen.		Reactor		Unit		Time Out		Time In					
	Hrs	Min	Hrs	Min	Hrs	Min	Hrs	Min	Hrs	Min	Hrs	Min	Hrs	Min	Hrs	Min	Hrs	Min				
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30	24	00	24	00			00	00	00	00	00	00	00	00								
31	24	00	24	00			00	00	00	00	00	00	00	00								
Total	720	00	720	00																		

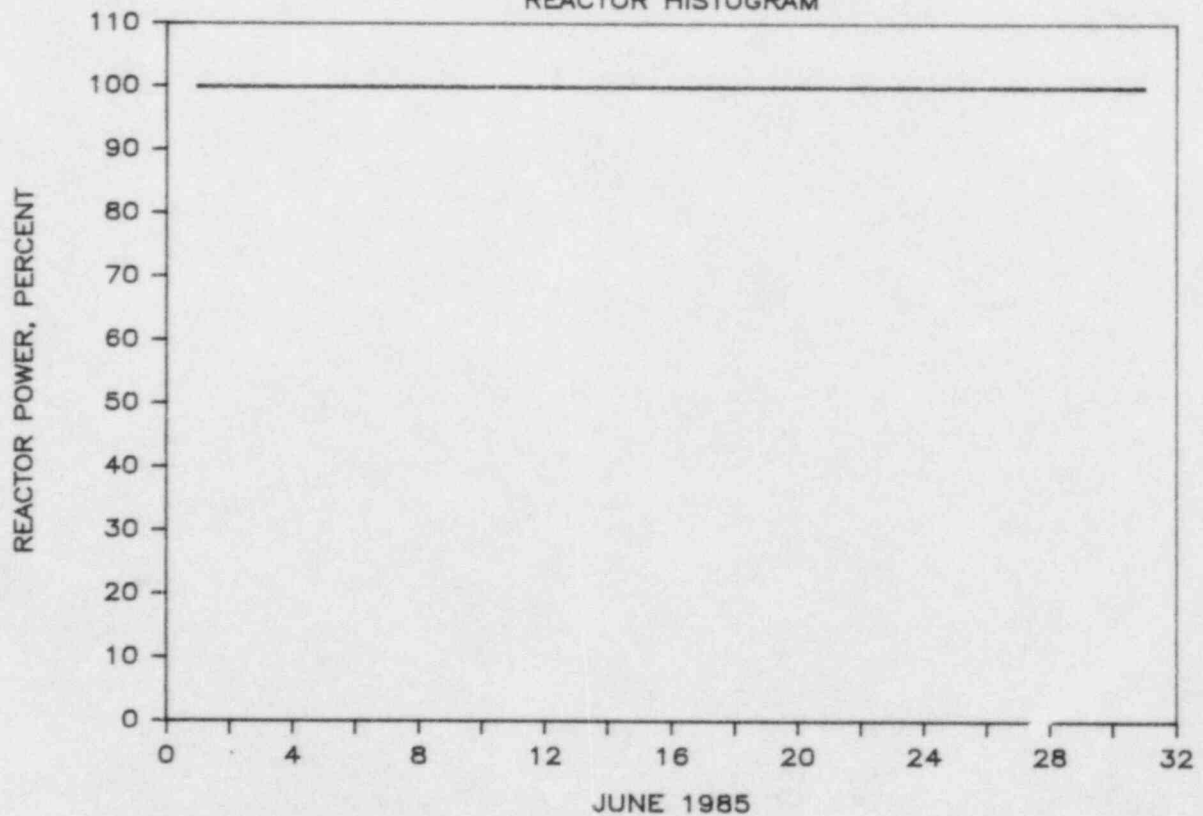
SEQUOYAH ONE

REACTOR HISTOGRAM



SEQUOYAH TWO

REACTOR HISTOGRAM



Reactor Histogram Comments

June 1985

Unit 1

1. 0012 The reactor entered mode 5 to repair the #1 seal housing on reactor coolant pump #4.
2. 1915 The reactor entered mode 4.
3. 2136 The reactor entered mode 3.
4. 1423 The reactor entered mode 4.
5. 1921 The reactor entered mode 5 to repair the flange seals on reactor coolant pumps #1 and #3.
6. 2040 The reactor entered mode 4.
7. 1243 The reactor entered mode 3.
8. 1120 The reactor entered mode 4.
9. 1635 The reactor entered mode 5 for repairs to CRD connector J-13.
10. 1025 The reactor entered mode 4.
11. 0035 The reactor entered mode 3.
12. 0945 The reactor was taken critical.
13. 2124 The unit tied on-line.
14. 0930 Began power reduction to 50% power due to loss of condenser vacuum.

Unit 2

The reactor maintained 100% power the entire month.

11:41:48 06-26-85 ELECTRICAL MAINTENANCE MONTHLY REPORT FOR JUNE PAGE 1
DATE.... COMPONENT..... FAILURE DESCRIPTION..... CAUSE OF FAILURE..... CORRECTIVE ACTION..... MR.NO..

05-08-85	2-MTRB-3-47	DISCONNECT MOTOR AND RECONNECT AFTER MECHANICAL REPAIR	MECHANICAL FAILURE	DISCONNECTED MOTOR AND RECONNECTED AFTER MECH. REPAIRED	A539164
05-16-85	1-HS-3-173A-B	HANDSWITCH BROKEN	CAUSE UNKNOWN	REPLACED HANDSWITCH	A528872
05-22-85	2-BKRC-099-KG/319	BROKEN CONTACT ON BACK OF BYPASS BREAKER	BAD CONTACTS	INSTALLED CONTACTS AND TOOK "B" BYPASS BREAKER OFF	A526430
05-29-85	1-MVOP-003-0047	DISCONNECT WIRING TO VALVE	MOTOR TO BE INSTALLED	INSTALLED MOTOR AND WIRING AND SET LIMITS	A542212
05-29-85	2-CHR-313-B	LOW OIL LEVEL	CAUSE UNKNOWN	CHARGED 8 GALLONS 4GS OIL	A535231
05-30-85	1-LCV-3-156A	LCV IS FULL OPEN WITH CONTROLLER IN AUTO AND MOTOR DRIVEN AUX FEEDWATER PUMP OFF	SHORTED DIODE IN JUNCTION BOX	REPLACED DIODE	A531067
06-03-85	1-LCV-003-0164 A-A	WHILE PERFORMING WORK ON 1-LCV-3-156 FOUND BLOCKING DIODE FROM 15624 TO 15625 IN JB3076 WAS NOT INSTALLED PER 45N1630-56 R5 VALVE WILL NOT PASS SI 276	BAD BLOCKING DIODE	REPLACED DIODE PER 45N1630-56 USING M&AI 12 AND MI 6.20	A542217
06-03-85	1-ZS-68-0340A-1-A	RED LIMIT LIGHT WILL NOT COME ON WHEN SPRAY VALVE OPENS	POSSIBLY BREAKER HAD BEEN LEFT OPEN WHEN PREVIOUSLY WORKED ON	FOUND BREAKER 24 OPEN ON 120V AC VITAL INSTRUMENT PANEL I-1 CLOSED BREAKER AND CHECKED FOR PROPER OPERATION	A531166
06-03-85	1-LCV-3-156-A	VALVE OPENS WHEN AUXILIARY FEEDWATER PUMP	DEFECTIVE DIODE	REPLACED DIODE PER PRINT 65N1630-56	A527725

11:41:48 06-26-85 ELECTRICAL MAINTENANCE MONTHLY REPORT FOR JUNE PAGE 2
DATE.... COMPONENT..... FAILURE DESCRIPTION..... CAUSE OF FAILURE..... CORRECTIVE ACTION..... MR.NO..

RUNNING WITH LESS THAN
500 PSIG IS SIMULATED
VALVE SHOULD OPEN UNDER
THESE CONDITIONS

06-05-85	2-FCV-63-175	VALVE BACK SEATS WHEN OPENED FROM HANDSWITCH	OPERATOR WAS REPLACED BY FSG WITH A FASTER GEAR RATIO MAKING IT A FAST-SPEED VALVE	PERFORMED SMI 2-317-19 AND ADJUSTED THE OPEN LIMIT TO 90%	A300111
06-05-85	1-FCV-68-333	CRYDON RELAY DEFECTIVE	LEAKING	REPLACED CRYDON RELAY PER MI 6.20	A100641
06-05-85	1-FCV-68-332-B	CRYDON RELAY DEFECTIVE	LEAKING	REPLACED CRYDON RELAY PER MI 6.20	A100640
06-06-85	1-FCV-001-0183	LIMIT SWITCH SHOWS BOTH RED AND GREEN WITH VALVE CLOSED	LIMIT SWITCH OUT OF ADJUSTMENT	ADJUSTED LIMITS	A531023
06-07-85	1-FCV-68-334	RED AND GREEN LIGHT ON WITH VALVE IN CLOSED POSITION	BAD REED SWITCH	REPLACED REED SWITCH	A543974
06-11-85	2-VLV-3-818	CHECK VALVE FOR LEAK THRU	VALVE LEAK	CLEANED ALL PARTS AND REINSTALLED	A103625
06-13-85	0-BAT8-82-C-S	REMOVE JARS FROM BATTERY UNTIL ONLY 63 CELLS ARE IN BATTERY AND ADJUST CHARGE	PREVENTIVE MAINTENANCE	REMOVED 9 CELLS FROM BATTERY ADJUSTED FLOAT VOLTAGE AND EQUALIZE VOLTAGE	A291971
06-14-85	2-MTR8-30-74	BREAKER IS TRIPPING OUT ON OVERLOAD	BAD MOTOR	REINSTALLED MOTOR AND CHECKED FOR PROPER OPERATION	A528704

INSTRUMENT MAINTENANCE

Unit 1

During startup of unit 1, while the operators were attempting to take the reactor critical, a problem developed in the Rod Control System. As shutdown bank B was being withdrawn, rod J-13 lagged behind and quit moving at 50 steps out. The operator then drove shutdown bank B back in. Rod J-13 fell in about 10 steps, moved in a few steps and then fell in to 0 steps. During trouble shooting by Instrument Maintenance, the movable gripper coil/cable set appeared to be open. It should have indicated ≈ 10 ohms. The unit was taken to mode 5, and the center missile shield removed. Electrical Maintenance inspected the connector and found the movable circuit pins burned. The connector was replaced and the problem resolved.

The following events occurred during startup of unit 1 following the ice weighing outage.

The turbine driven auxiliary feed water pump was declared inoperable. The level controller would not control in automatic and was replaced. Following replacement of the controller, the governor controls would not control speed. Two modules in the control circuit were replaced and the governor recalibrated. The LCV-3-148A would not operate properly and was declared inoperable. The I/P on the valve had a low output and was replaced.

Unit 2

Containment Sump Level Transmitter, 2-LT-63-176 required refilling of the sealed system. PRO 2-85-89 was initiated.

The turbine driven auxiliary feed water pump was declared inoperable because of a failure of the governor controls dropping resistor. A modification is being proposed to provide ventilation of the resistor to eliminate prolonged overheating which has been determined to be the root cause of the failure. This is the second failure of the dropping resistor this year.

Common

Three UHI level switches were identified as having been replaced with wrong QA Level material. Switches 1-LS-87-22, 1-LS-87-23, and 2-LS-87-23 were replaced per MRs with QA Level 2 material. These switches are Class 1E equipment and have been replaced with QA Level I switches.

Employees of the section participated in the walk down of the 5th diesel generator.

The computer staff is upgrading the PRIME to process computer link to provide a faster scan rate and include disk resident calculated values. Hardware has been tested and software development is in progress. Assisted in the setup of the Health Physics dosimetry computer (HPDC) and development of software for the HPDC. Assisted in the checkout and startup of the TSC computer system.

COMP

MR. COMP	U	FUNC	SYS	ADDRESS	DATE	DESCRIPTION	CORRECTIVE ACTION
A233484	2	FT	003	147	06/10/85	2-FT-003-147, VERIFY CALIB OF 2 FT 3 147 AND 2 FM NONE, VERIFIED CAL ON LOOP FOR PMT 53 3 147A UP TO TEST POINT TP12 2 L 11B	NONE.
A233486	2	FT	003	163	06/10/85	2-FT-003-163, VERIFY CALIB OF 2 FT 3 163 AND 2 FM NONE, VERIFIED CAL ON LOOP FOR PMT 53 3 163A UP TO THE TEST POINT TP13 IN 2 L 11B	NONE.
A292782	1	TM	068	1F	06/21/85	1-TM-068-1F, CHECK THE CALIB OF 1TM 68 1E	BAD MOD. REPAIRED MOD ON ANOTHER MR 299271 AND RECAL PER THIS MR
A298495	1	LIC	003	172	06/04/85	1-LIC-003-172, ALIGN & CALIBRATE CONTROLLER PER MANUFACTURES VALIDATED MANUAL	NONE. NONE VERIFIED ALIGNMENT AND OPERATION AND RETURNED TO SERVICE
A298496	1	LIC	003	173	06/04/85	1-LIC-003-173, ALIGN & CALIBRATE CONTROLLER PER MANUFACTURED VALIDATED MANUAL	NONE. NONE VERIFIED ALIGNMENT AND OPERATION AND RETURNED TO SERVICE
A298497	1	LIC	003	174	06/04/85	1-LIC-003-174, ALIGN & CALIBRATE CONTROLLER PER MANU. VAL. MANUAL	NONE. NONE VERIFIED ALIGNMENT AND OPERATION AND RETURNED TO SERVICE
A298498	1	LIC	003	175	06/04/85	1-LIC-003-175, ALIGN & CALIBRATE CONTROLLER PER MANUFACTURES VALIDATED MANUAL	NONE. ALIGNED AND CALIB AND RETURNED TO SERVICE
A299160	0	CLAN	043	205A	06/14/85	0-CLAN-043-205A, REPLACE ELECTRODE	OLD ELECTRODE WAS BROKEN AND REPAIRED. REPLACED ELECTRODE
A299265	1	TM	068	374E	06/07/85	1-TM-068-374E, CAL CARD PER CALIB	CARD OUT OF CAL. RECAL CARD
A299266	1	TM	068	380E	06/07/85	1-TM-068-380E, CALIB CARD PER CAL CARD	CARD OUT OF CAL. RECAL CARD
A299267	1	TM	068	381D	06/07/85	1-TM-068-381D, CALIB CARD PER CAL CARD	CARD OUT OF CAL. RECAL CARD
A299269	1	PT	068	69	06/12/85	1-PT-068-69, CHECK CALIB OF XMTR	XMTR WAS OUT OF CALIB. RECALIB XMTR AND RETURNED TO SERV
A300780	2	LS	087	23	06/12/85	2-LS-087-23, REPLACE 2 LS 87 23 WITH 1 LS 87 23 REMOVED FROM UI	NONE. NONE REPLACED SW WITH ONE REMOVED FROM UI
A301620	1	LS	087	24	06/17/85	1-LS-087-24, *I* CHANGE SW OUT WITH ONE REPAIRED	NONE. CHANGE SW OUT WITH REPAIRED ONE
A301734	1	LCV	003	148	06/20/85	1-LCV-003-148, INSTALL PROPER METERING TUBE	WRONG METERING TUBE IN VLV POSITIONER. INSTALLED PROPER METERING TUBE AND VERIFIED CAL AND STROKE
A527724	1	FCV	003	164A	06/03/85	1-FCV-003-164A, FCV WILL NOT FULLY OPEN WHEN IN AUTO	CURRENT TO PRESS CONVERTER WAS OUT OF CALIB. RECALIB THE CURRENT TO PRESS CONVERTER
A528060	2	XI	092	5000	06/05/85	2-XI-092-5000, *I* ALARM COMES IN WHEN CALIBRATING FOR SI 78	BAD POT. REPLACED POT
A528896	1	LI	003	93	06/21/85	1-LI-003-93, *I* INDICATED LVL ON 3 93 IS 72% WHEN 4 OTHER LVL INDICATORS INDICATE APPROX 90 % EA	SUSPECT BACKSEATING OF ROOT VLV. VERIFIED BACK PORTION OF LOOP
A528897	1	LCV	003	164A	06/20/85	1-LCV-003-164A, *I* VLV WILL NOT PMP CLOSE IN MANUAL	FOUND BLACK WIRE ON CURRENT TO PRESS CONVERTER GROUNDED. REPAIRED GROUNDED WIRE AND RETURNED TO SERV
A528953	1	LCV	003	148	06/20/85	1-LCV-003-148, CHECK VLV FOR PROPER STROKE AND ENSURE VLV'S ARE SEATING PROPERLY	NONE. NO PROBLEM FOUND AFTER CHECKING STROKE AND FOUND VLV NOT LEAKING THROUGH
A528954	1	LCV	003	148A	06/20/85	1-LCV-003-148A, CHECK VLV FOR PROPER STROKE AND	BAD MOD. MODIFIER REPLACED ON MR 545836

COMP

MR.	COMP	U	FUNC	SYS	ADDRESS.	DATE....	DESCRIPTION.....	CORRECTIVE ACTION.....
A528955	1	LCV	003	164		06/20/85	ENSURE VLV IS SEATING PROPERLY 1-LCV-003-164, CHECK VLV FOR PROPER STROKE AND ENSURE VLV IS SEATING PROPERLY	AND RECAL PER THIS MR VLV STROKE WAS NOT CALIB PROPERLY. RESTROKED VLV AND RETURNED TO SERV
A528957	1	LCV	003	164A		06/20/85	1-LCV-003-164A, CHECK VLV FOR PROPER STROKE AND SEATING	PACKING WAS BAD IN VLV. REPHCKED VLV-LUBRICATING STM AND RESTROKED VLV AND RETURNED TO SERV
A528972	1	LCV	003	148A		06/25/85	1-LCV-003-148A, *I* *NPRD* VLV IS LEAKING THRU	CURRENT TO PRESS CONVERTER WOULDN'T CALIB. REPLACED THE CURRENT TO PRESS CONVERTER
A528973	1	LCV	003	148		06/25/85	1-LCV-003-148, *I* VLV IS LEAKING THRU	NONE. VERIFIED PROPER CAL STROKE AND PRESSURES
A529094	2	PS	003	150A		06/08/85	2-PS-003-150A, REPLACE DAMAGED INSTRUMENT	TEE WAS BAD ON SENSE LINE. REPLACED INST LINE TEE; A SHORT SECT OF PIPING; AND RETURNED LOOP TO SERV
A529618	1	LIC	003	174		06/19/85	1-LIC-003-174, IND 72% WHEN LVL IS ABOUT 33%	PULLEYS THAT CONTROL IND WERE BINDING. ADJUSTED CONTROLLER HOUSING TO PREVENT BINDING
A531024	1	LT	003	174		06/04/85	1-LT-003-174, *I* LI 3 174 INDICATES ZERO WHEN 4 OTHER NARROW RANGE FOR THE SAME LOOP INDICATES 41%	WIRING WAS CONNECTED TO THE WRONG TERMINAL. CORRECTED THE WIRING AND RETURNED TO SERVI
A531185	1	PI	003	132A		06/09/85	1-PI-003-132A, *IE NPRD* PRESS IND INDICATING 930 PSIG WITH PMP NOT RUNNING	PRESS XMTR AMPLIFIER CARD HAD A BAD REGULATOR IN THE PWR SUPPLY SECT OF THE CARD. REPLACED REGULATOR IN PWR SUPPLY; RECALIB XMTR AND RETURNED LOOP TO
A531193	1	LI	063	109		06/11/85	1-LI-063-109, VERIFY PROPER CALIB	AIR IN SENSE LINE. VENTED UPPER AND LOWER BELLOWS
A531194	1	LI	063	99		06/11/85	1-LI-063-99, VERIFY PROPER CALIB	AIR IN SENSE LINES. VENTED UPPER AND LOWER BELLOWS
A531195	1	PI	063	108		06/11/85	1-PI-063-108, VERIFY PRESS IND	ROOT VLV CLOSED AND DRAIN VLV OPEN. CLOSED DRAIN VLV AND OPENED ROOT VLV CORRODED CONNECTION WELDED CABLE PER IMI 85
A531315	2		085	M2		06/10/85	2--085-M2, RPI IS DRIFTING >12 STEPS FROM 228 STEPS	XMTR WAS FOUND OUT OF CALIB AND WOULDN'T CALIB. RECALIB XMTR AFTER BACKFILLING FILLED SENSE LEG AND RETURNED TO SERV
A531321	2	LI	063	174		06/24/85	2-LI-063-176, *I* CONTAINMENT SUMP LVL IND IS OUT OF TOLERANCE/SI 2	REACTOR TRIP BKRS WERE RACKED OUT DURING TEST WHICH CAUSED PROBLEM. CHANGED THE UNDER VOLTAGE CARD BEFORE REALIZING CAUSE OF PROBLEM
A538637	1		099	TRNA		06/20/85	1--099-TRNA, DURING FT 18 SI 93 SSPS FT THE UNDER VOLTAGE METER DID NOT GO TO '0' VOLTS	DIRTY GAIN POT. REPLACED GAIN POT ON B
A541768	2	XX	092	5005		06/12/85	2-XX-092-5005, REPLACE GAIN POTENTIOMETER ON B	

COMP

MR. COMP	U	FUNC	SYS	ADDRESS.	DATE.....	DESCRIPTION.....	CORRECTIVE ACTION.....
A541769	2	XX	092	5006	06/12/85	DRAWER 2-XX-092-5006, REPLACE GAIN POTENTIOMETER ON B	DRAWER POT GAIN DIRTY. REPLACED GAIN POT ON B
A541770	2	XX	092	5007	06/12/85	DRAWER 2-XX-092-5007, REPLACE GAIN POTENTIOMETER ON B	DRAWER DIRTY GAIN POT REPLACED GAIN POT ON B
A541865	1	H2AN	043	200	06/03/85	DRAWER 1-H2AN-043-200, INST DOES NOT MEET SI 287/SI219	DRAWER LINE LOOSE ON DISCHARGE OF PMP. REINSTALLED LINE AND RETURNED TO SERVICE
A543970	1	LT	077	1	06/01/85	SPECS 1-LT-077-1, CHECK SENSING LINES FILLED & VENTED PROPERLY	LOSS OF FILLED LEG. REFILLED LEG
A543971	1	PT	077	02	06/01/85	1-PT-077-02, CHECK SENSING LINES FILLED & VENTED	OUT OF CAL. RECAL XMTR
A543999	1	FT	030	242	06/20/85	1-FT-030-242, SHIELD BLDG EXH FLOW XMTR IS SHOWING >4000CFM WITH ALL FANS OFF	LOOP OUT OF CAL. FLOW XMTR RECAL FM RECAL
A545836	1	LM	003	145A	06/20/85	1-LM-003-148A, WILL NOT CALIB	CONVERTER WAS DEFECTIVE. REPLACED THE CONVERTER
A545838	1	LM	003	164A	06/20/85	1-LM-003-164A, WILL NOT REPEAT	CURRENT TO PRESS CONVERTER WOULDN'T CALIB. REPLACED THE CURRENT TO PRESS CONVERTER
A548779	2	XX	092	5007	06/28/85	2-XX-092-5007, *IM CALIB NIS PER APPLICABLE SECTIONS OF IMI 92 PRM CAL	CHANNEL OUT OF CAL. RECAL CHANNEL INST
A548955	2	LS	087	22	06/11/85	2-LS-087-22, LOCKING SETSCREW STRIPPED ON SW	SET SCREW STRIPPED WHILE PERFORMING SI
A564863	1	LIC	003	148	06/04/85	1-LIC-003-148, ALIGN AND CALIBRATE CONTROLLER PER MANUFACTURES MANUAL	REPLACED SCREW AND RECAL
A564864	1	LIC	003	156	06/04/85	1-LIC-003-156, ALIGN AND CALIBRATE CONTROLLER PER MANUFACTURES VALIDATED MANUAL	NONE. NONE VERIFIED ALIGNMENT REPLACED PROCESS. VARIABLE INDICATOR AND RETURNED TO SERVICE
A564865	1	LIC	003	164	06/04/85	1-LIC-003-164, ALIGN AND CALIBRATE CONTROLLER PER MANUFACTURES VALIDATED MANUAL	NONE. NONE PERFORMED ALIGNMENT AND CALIB AND RETURNED TO SERVICE
A564866	1	LIC	003	171	06/04/85	1-LIC-003-171, ALIGN AND CALIBRATE CONTROLLER PER MANUFACTURES VALIDATED MANUAL	NONE. NONE VERIFIED ALIGNMENT AND OPERATION AND RETURNED TO SERVICE
A564867	1	LCV	062	118	06/04/85	1-LCV-062-118, ULV DOES NOT CONTROL PROPERLY IN P AUTO	NONE. ALIGNED AND RECAL CONTROLLER PER ENG'R REQUEST
							TOOK READINGS AND GAVE TO ENGR FOR REVIEW AND RETURNED TO SERVICE

51 records listed.

COMP

MR.HIST U FUNC SYS ADDRESS. DATE.... DESCRIPTION..... CORRECTIVE ACTION.....

A233485	2 FT	003 155	06/03/85	2-FT-003-155, VERIFY CALIB OF 2 FT 3 155 AND 2 FM OUT OF CAL I/I	RECAL 2 FM 3 155A
				3 155A UP TO THE REST POINT TP13 IN 2 L 11A	
A538605	1 PM	068 690	06/03/85	1-PM-068-690, INPUT TO RVLIS IS READING LOW.	OUT OF CAL I/I
				CHECK CAL	RECAL PM 68 690

2 records listed.

Mechanical Maintenance Section

June 1985

Unit 0

- 1) Repaired the potable water line to the Field Services Building.
- 2) Completed the monthly inspection of the diesel generators.
- 3) Completed the 1-BB diesel generator annual inspection.

Unit 1

- 1) Replaced the O-ring below the #1 seal housing on #4 reactor coolant pump.
- 2) Completed reassembly of the main generator.
- 3) Installed, adjusted, and/or aligned 38 damage hangers in the Turbine Building on the steam dump line.
- 4) Replaced the solenoid valve on 1-LCV-62-118.
- 5) Replaced the main flange O-ring and installed an additional O-ring in the bore below the #1 seal housing on reactor coolant pumps #1 and #3.
- 6) Applied RTV to the #1 seal housing and main flange on #1 and #3 reactor coolant pumps to avoid any oil which could seep in between the main flange and the #1 seal housing.
- 7) Replaced a defective regulator on 1-FCV-1-182.
- 8) Installed an isolation valve on the relief valve discharge to build up the auto stop oil pressure on the EHC system.

Unit 2

- 1) Furmanited 2-FCV-3-100.
- 2) Installed a furmanite box over a leaking gland seal water elbow to stiffen the piping and prevent shearing of the piping at the threaded joint.

SUMMARY OF WORK COMPLETED

MODIFICATIONS

JUNE 1985

NUREG 0588

ECN 5970 - MOV Operator Replacement

One remaining operator will be replaced the first week of July.

ECN 6278 - ABGTS and EGTS Heater Controller Replacement

Two rubber hoses were replaced the first week in June.

Appendix R

ECN 5435 - Install Weatherstripping on ABSCE Doors

We started replacing the weatherstripping on several ABSCE architectural doors. Material was ordered for heavy equipment doors.

ECN 5484 - Emergency Lights

This work is complete.

ECN 6235 - Reroute Various Cables

Work continues with five workplans in work.

ECN 6305 - Elevation 714 Fire Barrier

We are in the process of removing interferences required to install wall. We are also laying out the wall and installing the baseplates.

ECN 6319 - Fire Protection Piping

We have rerouted approximately 30 percent of the heads required to be relocated. Work is ongoing to plug sprinkler heads that are to be abandoned (approximately 70 percent complete). We are adding heads on elevations 734 and 749 and are laying out work on elevation 714 for relocation. Work continues on the workplan to install two new deluge valve systems.

Other Items

ECNs 2783 and 5202 - Fifth Diesel Generator

Permanent power tie-in will be made in July. The ERCW discharge (ECN 5202) project has been completed, and the backfilling has been done. Work on permanent power to the building is continuing. A punchlist is being prepared for the incomplete items.

Other Items (Continued)

ECN 5009 - ERCW Piping Changeout from Carbon Steel to Stainless Steel

The changeout for the auxiliary feedwater/boric acid area cooler (train B) is complete. Work has started on train A.

ECNs 5111 and 5503 - Office and Power Stores Facility

Work is continuing to complete the remaining punchlist items by July 15, when Power Stores will begin moving into the new facility. Interior items include wiring for alarms and fire protection, cleanouts for subdrain, high-pressure fire protection, and completion of the pallet stacker system. Exterior items include yard grading, catch basins, sidewalks, handrails, and manhole covers.

ECN 5119 - Install Radiation Monitor Cables in Conduit

Final tie-ins are underway. Five monitors remain.

ECN 5194 - Iodine Monitor

Doors A230 and A231 have been tied in and are awaiting acceptance testing.

ECN 5200 - Postaccident Sampling Facility

Rework of postmodification test deficiencies is in hold for the final design of one level loop.

ECN 5237 - Laundry Facility

Remaining work is on hold until the last wall is built.

ECN 5373 - Condensate Demineralizer Air Compressor

This project is almost complete, with the exception of the installation of the motor coupling and pressure control instrument. Functional testing of the air compressor will be done with the help of a vendor representative during August. Construction is trying to buy a control box to replace a faulty one. Compressor startup is awaiting this new control box.

ECN 5599 - Fifth Vital Battery

The Appendix R-related conduit wrap is almost complete. Labels for the breakers have arrived. The protective coating in the battery room is to be done during August.

ECNs 5609 and 5610 - Makeup Water Treatment Plant

This project is near completion. Remaining punchlist items include modifications to the demineralizer system and tank, interior potable water work, and installation of a septic tank.

Other Items (Continued)

ECN 5613 - Installation of Emergency Lights

Work has commenced to install emergency lights in the Service Building. Five of nine are complete.

ECN 5657 - Installation of Isolation Valves, Moisture Separator Reheaters

The unit 2 workplan to install approximately 150 isolation valves is written and in the approval cycle. Insulation reinstallation on unit 1 still remains incomplete.

ECN 5664 - Replace Relays in Wells Fargo Alarms

Remaining work has been restarted and will continue during Power Block reconfiguration.

ECN 5795 - Field Services Building

Fire detection system work is in hold for materials.

ECN 5841 - Hot Machine Shop

All work is complete with the exception of the hallway on elevation 706. This will be completed when Power Stores is moved to their new facility. We are still awaiting drawings on the monorail to be added to the decontamination room.

ECN 5855 - Replace Air Lock Doors A56 and A57

We have started working on chipping concrete columns between doors to ready wall for air tanks. Workplans are being written to install doors and hardware.

ECN 5878 - CDWE Outage Work

The CDWE outage is in progress, and the modification work is going well. We are in the process of expediting some material that is slowly coming in.

ECNs 5932 and 5935 - Power Block Implementation

Work is continuing on Power Block-related cameras, fence sectors, gates, doors, and E-fields. Most of the primary work for Power Block-related roads is underway. Also added to the scope of the project are the installation of high mast lighting and painting of ERCW cofferdam for improved visibility. A workplan has been initiated for the high mast lighting.

Other Items (Continued)

ECN 5938 - Feedwater Heater Replacement

Monorail and electric hoist work required to handle feedwater heater Nos. 1 and 2 has been completed. Work is in process on the platforms needed to bring the feedwater heaters into the Turbine Building. All the Nos. 3 and 4 feedwater heaters from Hartsville have been modified and are being hydrostatically tested and flushed.

ECN 5990 - CDWE Outage Work

The CDWE outage is in progress, and the modification work is going well. We are in the process of expediting some material that is slowly coming in.

ECN 6057 - Cable Tray Covers

Approximately 240 out of 290 cable tray covers have been remanufactured or replaced.

ECN 6202 - Component Cooling System Surge Tank Instrumentation

All mechanical work has been completed.

ECN 6204 - Electrical Penetration Overcurrent Protection

Fuse replacement and fuse block installation are complete. We are waiting for a Technical Specification change to place the circuits in operation.

ECN 6326 - CDWE Outage Work

The CDWE outage is in progress, and the modification work is going well. We are in the process of expediting some material that is slowly coming in.

ECN 6342 - Health Physics Calibration Facility

This project has been completed.

ECN 6351 - CDWE Outage Work

The CDWE outage is in progress, and the modification work is going well. We are in the process of expediting some material that is slowly coming in.

ECN 6362 - CDWE Outage Work

The CDWE outage is in progress, and the modification work is going well. We are in the process of expediting some material that is slowly coming in.

Other Items (Continued)

ECN 6417 - CDWE Outage Work

The CDWE outage is in progress, and the modification work is going well. We are in the process of expediting some material that is slowly coming in. We are waiting on the Office of Engineering to complete their work on ECN 6417.

Training Buildings

Exterior work is near completion. The sidewalks are almost finished. Completed interior items include lighting, walls, ceilings, doors, and windows. The HVAC system is approximately 75 percent complete. This project is running on schedule to be completed by July 29, 1985.

Miscellaneous Yard Work

Roads have been paved to the cooling towers and the Health Physics Calibration Facility, near the switchyards, west of the Office and Power Stores Facility, and leaving the plant along access highways. The pavers have left the project for now and will return in August. Power Block-related fencing and road work are being done. A workplan has been initiated for completion of the staff parking lot.

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TENNESSEE VALLEY AUTHORITY
Sequoyah Nuclear Plant
P. O. Box 2000
Soddy-Daisy, Tennessee 37379

July 15, 1985

Nuclear Regulatory Commission
Office of Management Information
and Program Control
Washington, DC 20555

Gentlemen:

SEQUOYAH NUCLEAR PLANT - MONTHLY OPERATING REPORT - JUNE 1985

Enclosed is the June 1985 Monthly Operating Report to the NRC for Sequoyah Nuclear Plant.

Very truly yours,

TENNESSEE VALLEY AUTHORITY

P.R. Wallace

P. R. Wallace
Plant Manager

Enclosure

cc (Enclosure):

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