



Boston Edison

Pilgrim Nuclear Power Station
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E. T. Boulette, PhD

Senior Vice President — Nuclear

November 26, 1996
BECo Ltr. #2.96-102

U.S. Nuclear Regulatory Commission
Attention: Document Control Desk
Washington, DC 20555

Docket No. 50-293
License No. DPR-35

Proposed Administrative Technical Specification Changes

In accordance with the requirements of 10CFR50.90, Boston Edison Company proposes several administrative changes to the Pilgrim Station Technical Specifications.

The description of the proposed changes is included in Attachment A. Attachment B contains the amended Technical Specification pages. Attachment C contains a marked-up copy of the existing Technical Specification pages.

Should you have any questions, please contact M. Lenhart at 508-830-7937 in our Regulatory Affairs Department.

E. T. Boulette, PhD

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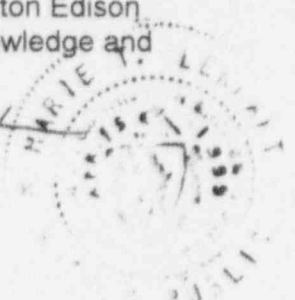
Commonwealth of Massachusetts)
County of Plymouth)

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Then personally appeared before me, E. T. Boulette, who being duly sworn, did state that he is Senior Vice President - Nuclear of Boston Edison Company and that he is duly authorized to execute and file the submittal contained herein in the name and on behalf of Boston Edison Company and that the statements in said submittal are true to the best of his knowledge and belief.

My commission expires: April 15, 1999
DATE

Notary Public



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Attachment A: Description of Proposed Changes
Attachment B: Amended Technical Specification Pages
Attachment C: Marked-up Technical Specification Pages

Original plus 37 copies

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Attachment A
Description of Proposed Changes

Proposed Changes and Reason for Change:

- Definition 1.M, "Primary Containment Integrity," is modified to include "all instrument line flow check valves" to make this definition consistent with Limiting Condition for Operation 3/4.7.A.2.a.4.
- Note 6 on Table 3.2.A for the high flow main steam line instrumentation that initiates primary containment isolation is reworded for added clarity. Note 6 has been the subject of past confusion. The current wording of "Two [channels per trip system] required for each steam line" could be misinterpreted to mean a total of 8 instrument channels are needed. Actually, the four main steam lines have two instrument trip channels for each of the two trip systems. Therefore, 4 instruments per main steam line must be operable. An internal Boston Edison memorandum was issued to assure proper understanding of this Technical Specification (TS) requirement. Rewording the note in the TS will resolve this issue.
- Table 3.2.D of the TS contains a typographical error. There are two notes labeled number 1. This change labels the second note as number 2. This change does not change the content of the note.
- Table 3.2.F is modified as a result of a change made in instrument type for the suppression chamber water temperature instrumentation. The instrument numbers and indication type have changed. Table 3.2.F currently lists instruments TI-5021-01A and TRU 5021-01A, and TI 5022-01B and TRU 5022-01B as the instruments comprising the channels for suppression chamber water temperature indication.

Table 3.2.F also lists TI-5021-01A and TI-5022-01B as dual indicators. The original instruments indicated both suppression chamber bulk and local temperatures. A plant change replaced the dual instruments with individual instruments. The instrument requirements of TS 3.7.A.1.e through h are based on bulk temperature. Therefore, TI-5021-2A is the indicator for TRU-5021-1A and TI-5022-2B is the indicator for TRU-5022-1B.

Also, the instruments and nomenclature for the drywell/torus differential pressure, drywell pressure and torus pressure instrumentation are changed. The previous instruments had age-induced drift as well as a lack of sufficient accuracy in the display. As a result, monthly calibration was required to maintain minimum acceptable accuracy. These new instruments allow us to revert to the once per six month test frequency currently in TSs.

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Attachment A
Description of Proposed Changes

- An obsolete reference to Group I welds in the section 3/4.6G Bases is removed. This Group I designation pre-dates the current ASME XI code and should have been removed when the Technical Specifications were revised in Amendment 149, dated 9/28/93. The ASME XI code requirement is to assure high-stress welds are included in the weld selection process for inspection. The current PNPS ISI Program includes this requirement; however, we no longer include the Group I welds in the ISI Program unless they meet or exceed the code stress criteria.
- The description of Primary Containment Group 1 initiation signals in Bases 3/4.7.A is being revised to remove "high radiation." The main steam high radiation signal was removed from Technical Specifications in Amendment 154 dated June 21, 1994. This Bases reference was inadvertently overlooked and is corrected in this submittal.

Discussion:

These changes are proposed to correct typographical errors, add clarity and consistency, modify instrument numbers due to plant modifications and update the Bases. These changes are considered administrative. These changes have no effect on plant design, safety limit settings, or plant system operation.

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Attachment A
Description of Proposed Changes

Determination of No Significant Hazards Considerations

The Code of Federal Regulations (10CFR50.91) requires licensees requesting an amendment to provide an analysis, using the standards in 10CFR50.92, that determines whether a significant hazards consideration exists. The following analysis is provided in accordance with 10CFR50.91 and 10CFR50.92 for the proposed amendment.

1. The proposed amendment does not involve a significant increase in the probability or consequences of an accident previously evaluated.

The proposed administrative changes involving typographical errors, additions for clarity and consistency, and updating the Bases do not affect plant design, safety limit settings, or plant system operation and, therefore, do not modify or add any initiating parameters that would significantly increase the probability or consequences of any previously analyzed accident.

The changes to instrument numbers and type do not change the parameters being surveyed or the number of operable channels for these parameters. These changes do not modify or add any initiating parameters and do not affect plant design, safety limit settings, or plant system operation. Therefore, these instrument changes do not involve a significant increase in the probability or consequences of an accident previously evaluated.

2. The proposed amendment does not create the possibility of a new or different kind of accident from any accident previously evaluated.

These proposed changes do not involve any potential initiating events that would create any new or different kind of accident. Therefore, the proposed changes do not create the possibility of a new or different kind of accident from any accident previously evaluated.

3. The proposed amendment does not involve a significant reduction in a margin of safety.

These changes do not affect any safety analysis assumptions, system operation, structures, potential initiating events or safety limits. Therefore, it is concluded that the proposed amendment does not involve a significant reduction in a margin of safety.

The proposed changes have been reviewed and recommended for approval by the Operations Review Committee and reviewed by the Nuclear Safety Review and Audit Committee.

Schedule of Change

This change will become effective 30 days following Boston Edison's receipt of the Commission's approval.

Attachment B

Amended Technical Specification Pages

(TSMarie.doc)