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The Northeast Utilities System

February 26, 1997
Docket No. 50-336
B16257

U.S. Nuclear Regulatory Commission
Attention: Document Control Desk
Washington, DC 20555

Millstone Nuclear Power Station, Unit No. 2
Readiness-for-Restart Worklist

In response to your January 13, 1997, letter,[1] Northeast Nuclear Energy Company (NNECO) hereby transmits for Millstone Unit No. 2 the Readiness-for-Restart Worklist (worklist) which identifies the currently known items to achieve readiness for restart. These are provided in Attachments 1 through 5 and consist of Success Objectives, Activities Necessary to Raise Standards, Design and Configuration Discrepancies, Regulatory Readiness Items, and Common Millstone Site Issues Applicable to Millstone Unit No. 2.

The worklist was developed and reviewed by the individuals accountable for the completion of the contained activities and their line management. The worklist will be updated and resubmitted periodically. The next update is planned for April 1997. The additions to the worklist will be reviewed and approved by the respective department managers. The removal or deletion of an activity from the worklist will be reviewed by the impacted department managers and approved by the Plant Operations Review Committee and the Unit Director.

The successful accomplishment of the seven success objectives delineated in Attachment 1 are fundamental to raising the standard for the conduct of safe operations at Millstone Unit No. 2. The activities necessary to raise standards are provided in Attachment 2 and consist of organizational and management requirements and key performance indicators from the Millstone Unit No. 2 Operational Readiness Plan (ORP). The Millstone Unit No. 2 ORP is currently being revised and will be provided

030005

(1) W. D. Lanning (NRC) letter to B. D. Kenyon, "Recovery and Restart Plans Meeting of December 17, 1996," dated January 13, 1997.

A001

under separate correspondence when the revision is complete. The Millstone Unit No. 2 ORP Nuclear Oversight Plan provided in Attachment 6 is for information only.

We plan on additional communications with your Staff in the near future to review this approach and identify additional details necessary to support the Special Projects Office's informational needs.

The following are NNECO's commitments made within this letter. All other statement's made within this letter are for information only.

B16257-01	The worklist items will be dispositioned prior to the start of Millstone Unit No. 2.
B16257-02	The Millstone Unit No. 2 Operational Readiness Plan, Revision 1 will be submitted by March 31, 1997.

If you have any questions, please contact Mr. R. G. Joshi at (860) 440-2080.

Very Truly yours,

NORTHEAST NUCLEAR ENERGY COMPANY

Martin L. Bowling, Jr.
Millstone Unit No. 2 - Recovery Officer

Attachments (6)

cc: H. J. Miller, Region I Administrator
D. G. McDonald, Jr., NRC Senior Project Manager, Millstone Unit No. 2
D. P. Beaulieu, Senior Resident Inspector, Millstone Unit No. 2
W. D. Travers, Dr., Director, Special Projects Office

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Attachment 1

Millstone Nuclear Power Station, Unit No. 2

Success Objectives

February 1997

Attachment 1
Millstone Nuclear Power Station, Unit No. 2
Success Objectives

Success Objectives

The Millstone Unit No. 2 Recovery Organization has identified seven key success objectives to raise operating standards. High operating standards are essential for the safety and reliability of Millstone Unit No. 2 operations. The seven key success objectives also reflect fundamental changes in how the Millstone Unit No. 2 organization will function. The objectives are as follows:

1. High Standards and Clear Accountabilities

   - Incorporate best practices from other utilities
   - Benchmark with other nuclear utilities
   - Performance indicators show improvement toward excellence
   - Commitments are met

2. Strong Nuclear Safety Policy

   - Careful adherence to high nuclear safety standards
   - Conservative decision making

3. Licensing and Design Bases Restored with Process in Place to Ensure Bases are Maintained.

   - ICAVP contractor confirmation that the design and licensing bases have been restored
   - Implement configuration control processes to ensure design and licensing bases are maintained

4. An Environment That Supports the Identification and Effective Resolution of Employee Concerns

   - Improved Employee Concerns Program actively supported by management
   - Employee concerns addressed promptly, effectively, and respectfully
   - Independent review of employee safety concerns resolution effectiveness

5. Effective Self-assessment Program
   - Line management ownership and support
   - Significant issues identified by NU rather than the regulator

6. Effective Corrective Action Process.

   - Corrective actions and commitments are prioritized and resolved in a timely manner
   - Improved regulatory performance demonstrated by a decrease in NRC violations and LERs

7. Commitment to Resolve Long-standing Issues and Maintain Safe and Reliable Operations

   - Resource commitments meet or exceed those of similar well run units
   - Problems are prioritized and resolved in a timely manner
   - Commitment to resolve longer term issues is captured by continuous improvement.

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Attachment 2

Millstone Nuclear Power Station, Unit No. 2

Activities Necessary to Raise Standards

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ACTIVITIES NECESSARY FOR RAISING STANDARDS

Affirm that Maintenance Rule risk significant systems are ready for restart.
Communicate standards and expectations to employees in accordance with the MP2 Communications Plan.
Complete actions to ensure compliance with 10CFR50, Appendix R for Thermo-Lag.
Complete development of Unit Performance Indicator for ORP.
Complete employee familiarization with Employee Concerns Program.
Complete independent (external) assessment of conduct of Operations.
Complete independent review of ORP.
Complete management training on expectations and standards for resolving employee concerns, as well as, the Employee Concerns Program.
Complete required design change modifications.
Complete response for SOER 96-01.
Complete self-assessment of work flow process.
Complete work flow process changes.
Conduct independent Organizational Readiness Assessment (use INPO and NRC 0350 criteria).
Conduct operational readiness self-assessment prior to each mode change and significant plant configuration change.

Conduct periodic self-assessments for adherence to Surveillance Testing schedule and requirements.
Conduct periodic self-assessments for conduct of operations.
Conduct periodic self-assessments for Configuration Management.
Conduct periodic self-assessments for Corrective Actions.
Conduct periodic self-assessments for PORC effectiveness.
Conduct periodic self-assessments for technical procedures adequacy.
Conduct periodic self-assessments of Safety Evaluations (10CFR50.59) Program and Process.
Conduct self-assessment to ensure MP2 Nuclear Safety Policy expectations are embraced by MP2 organization.
Coordinate 10CFR50.59 training schedule for all applicable MP2 staff.
Develop and implement an operational readiness self-assessment process (pre-INPO/pre-IPAP methodology).
Develop and implement Human Performance Enhancement Program.
Develop and implement MP2 Nuclear Safety Policy.
Develop and plan for effective use of Nuclear Oversight for periodic assessment of daily activities, processes and programs.
Develop and rollout Operations Department standard for reactivity management.
Develop CMP Plan and Schedule.

Develop plan for effective self-assessment of relevant industry operating experience.
Develop schedule and agenda for Human Performance Standdowns.
Develop schedule and methodology for benchmarking trips.
Develop schedule for Improved Standard Technical Specifications.
Develop set of Operations Standards.
Establish a Level 1 Management Accountability System.
Establish appropriate linkage between operability determinations and bypass jumpers with the 50.59 process.
Establish Engineering Assurance Program.
Establish goals and trends for corrective actions.
Establish housekeeping standards and program.
Establish indicators and goals for determining effectiveness of the Corrective Action Program.
Establish ownership for system design and technical programs.
Establish ownership of corrective action program within line organization through revision of Corrective Action Program procedure.
Establish plan to review technical procedures.
Establish schedule for facilitated working sessions to introduce and reinforce behavioral characteristics for first line and above supervisors.

Establish Single Technical Procedures Group.
Establish WIN team.
Identify actions necessary to ensure full compliance with 10CFR50 Appendix B for Oversight Audits.
Implement Observation Training.
Implement required actions to ensure timely response to NRC violations. Develop schedule to achieve timely compliance.
Implement Station Qualified Review (SQR) Program, applicable for Millstone Unit 2.
Open Bypass Jumpers, Operability Determinations, and NCRs not resolved prior to startup are screened against 10CFR50.59.
Open safety related AWOs not addressed prior to restart have been assessed by an Operations SRO and PORC.
Open unresolved items (URIs) and LER commitments have been addressed prior to restart.
Perform benchmarking assessment at well-performing plants
Perform independent assessment of restart readiness.
Perform periodic independent technical review of CMP.
Perform self-assessment of AITTS effectiveness.
Prepare Startup and Power Ascension Plan.
Provide plans for Engineering Drawing Backlog.

Provide self-assessment training to applicable personnel.
Provide/set standards and expectations for self-assessment.
Publish and review on a monthly frequency UPIs for ORP.
Request independent (external) restart readiness assessment.
Review a sample of previously closed ACRs for adequacy.
Review and revise Operating procedures to ensure water hammer events are precluded.
Review Tech Spec Surveillance Procedures to ensure compliance with Tech Specs.
Set and communicate expectations and standards for resolution of employee concerns.
Specific departmental operational readiness assessments will be performed prior to mode changes.
Train department coaches.

Millstone Unit No. 2
Restart Readiness Indicators

Corrective Actions

Restart List
NCR Backlog
AITTS Assignments Required for Startup
Restart Related NRC Commitments
Open/Overdue Licensee Event Reports
Open/Overdue Notices of Violations
License Amendments Required for Restart
AITTS Assignments in NOTIFY Status
Overdue Assignments by Department
Assignments Against ACRs Open >120 Days by Dept.
ACR Aging/Overdue Report
Open/Overdue Assignments - Audits/Surveillances

Configuration Management Program

Unresolved Items Report
Drawing Update Backlog
Restart Modification Implementation Status
Configuration Management Plan - System Assessments
Configuration Management Plan - Program Assessments

Physical Plant

Maintenance Rule AWO Backlog
Maintenance Rule (a)(1) Systems Status
MEPL Systems Validated
Open Operability Determinations
Operator Burden Reduction
Control Room Panel Deficiencies
Temporary Modifications
Liquid Radwaste Release Data
Gaseous Radwaste Release Data

Organizational Performance and Safety

Restart Required Technical Procedure Revisions
Reportable Events
Ratio of Cited/Non-Cited Notices of Violations
Work Planning/Scheduling Effectiveness
Industrial Safety Accident Rate
Personnel Contamination Events
Collective Radiation Exposure
MEPL Systems Validated

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Attachment 3

Millstone Nuclear Power Station, Unit No. 2

Design and Configuration Discrepancies

February 1997

Design and Configuration Discrepancies

As required by NRC letter dated May 21, 1996, [2] attached is an initial list of Millstone Unit No. 2 design and configuration discrepancies based on a review of the Adverse Condition Reports (ACRs). In addition, this list includes other discrepancies identified during routine operation, maintenance, and design activities. As stated in the cover letter, this worklist will be updated in April 1997.

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[2] W.T. Russell letter to R.E. Busch, "MP1, 2, and 3 - 10CFR50.54 (f) Letter In Regards to the Operation of All Three (3) Millstone Units", dated May 21, 1996.

Millstone Nuclear Power Station, Unit 2
Design and Configuration Deficiencies

Number	Description
00503	RCS LOW FLOW TRIP MAY NOT SATISFY BOUNDS ASSUMED IN MP2 SAFETY ANALYSIS
01991	VITAL DC POWER FAILURE (201A/B) PREVENTS POST-LOCA FLOWPATH ALIGNMENT FOR HYDROGEN MONITOR SYSTEM
02797	AUX FEED REG VALVE WILL NOT FULLY SHUT WITH FLOW THROUGH THE VALVE (2-FW-43B)
03531	SERVICE WATER STRAINER MOTOR OPERATION NOT DESCRIBED IN APPENDIX R OF FLOOD DOCUMENTS
04071	REED SWITCHES PROVIDED ERRONEOUS POSITION INDICATION FOR CEA 1-28 DURING SURV TESTING
05595	REACTOR CORE POWER EXCEEDED 2700 MW-THERMAL FOR 11 HOURS (BLOWDOWN CURVE ERRORS)
05857	EBFS IS DRAWING LESS VACUUM THAN EXPECTED WHEN RUNNING ON ENCLOSURE BUILDING WITH AC 1 & 3 SHUT
06351	MP1 DESIGN CHANGE (ROLL-UP DOOR) MAY AFFECT MP2 TURBINE BUILDING OVER PRESSURIZATION
06372	RCS HEATUP RATE EXCEEDED TECH SPEC LIMIT OF 50 DEGREES PER HOUR
06668	TRIAL INSTALLATION OF SERVICE WATER PUMP FLOOD CAN ON "A" MOTOR WAS UNSUCCESSFUL; LER INITIATED
07506	PASS IS OUTSIDE DESIGN BASIS - SLOW POST-LOCA CTMT PRESSURE REDUCTION PREVENTS TIMELY PASS AIR SAMPLE
07896	50542:DISCREPANCIES FOUND IN TECH SPECS 3.4.1.3, 3.4.3, 3.4.7, 3.4.9.1, 3.4.9.2 & 3.4.9.3

Number	Description
07910	2-RB-13.1 A/B ACCUMULATOR MAY BE SIZED INCORRECTLY BASED ON WRONG TIME FROM EVENT TO SRAS
07916	50542:EBFS HUMIDITY MAY NOT MEET DESIGN BASIS DUE TO DESIGNED FLOW INCREASE WITHOUT RESIZING HEATER
07924	HYDROGEN MONITORING SYSTEM MAY NOT MEET SINGLE FAILURE CRITERIA
07958	50542: DESIGN BASIS FOR MCC ENCLOSURES IS NOT EASILY RETRIEVABLE OR UNDERSTOOD
08038	SERVICE WATER STRAINER BACKWASH LINES MAY FREEZE IF VENTILATION HEATERS LOST FOR PROLONGED PERIODS
08331	HYDROGEN ANALYZER SAMPLE PUMP DEADHEADS AT 25#, IT WILL NOT DISCHARGE INTO CTMT IN ALL CONDITIONS
08391	H2 ANALYZER PRESSURE REGULATOR DESIGN MAY PREVENT H2 SAMPLING WHEN CTMT IS AT ATMOSPHERIC PRESSURE
08395	SIEMENS REPORTED A SIGNIFICANT LARGE BREAK LOCA ANALYSIS ERROR; PCT REMAINS BELOW 2200 DEGREES
08490	CONTAINMENT SUMP SCREEN ENCLOSURE HAS OPENINGS LARGER THAN DESIGN (MESH SIZE ALONG END & 3 HOLES)
09563	AUX FEED REG VALVE BACKUP AIR BOTTLE PRESSURE MAY BE TOO LOW TO ISOLATE S/G IN STEAM LINE BREAK
09590	50542:NUREG 0612 EVALUATION OMITS TURBINE BLDG CRANE'S IMPACT ON SAFETY-RELATED SWITCHGEAR ROOM
09600	50542: CIRC PUMP LOSS OF LUBE WATER AUTOMATIC TRIP LISTED IN THE FSAR DOES NOT ACTUALLY EXIST
09612	50542: DIESEL GENTERATOR START TIME REQUIREMENTS DIFFER IN TECH SPECS (<15 SEC) & FSAR (<12 SEC)

Number	Description
09623	DUAL FUNCTION VALVES MAY NOT BE SET UP PROPERLY - PAST CORRECTIVE ACTIONS NOT TIMELY OR EFFECTIVE
09643	50542: RBCCW HEADER DRAINAGE PATH MAY RESULT FROM AN LNP COINCIDENT WITH A LOCA OR SEISMIC EVENT
10107	ELECTRICAL SEPARATION VIOLATIONS & INADEQUACIES FOUND IN MAIN CONTROL BOARDS
11104	50542: POST-LOCA INITIATION OF SPENT FUEL POOL COOLING PER FSAR 9.4.3.2 MAY NOT BE PROCEDURALIZED
11116	DESIGN CALC'S SHOW RBCCW MOTOR AMBIENT TEMPERATURE MAY EXCEED RATED TEMPERATURE BY 20 DEGREES C
13527	50542: APPLICABILITY OF RCS PRESSURE/TEMPERATURE LIMITS IS 12 EFPY IN FSAR & 20 EFPY IN TECH SPEC
13530	50542: TS LCO 3.4.9.1 HU/CD LIMITS BASED ON TAVG, SHOULD BE TC OR VESSEL INLET TEMPERATURE
13531	50542: INSUFFICIENT ANALYSIS DONE AFTER THERMAL SHIELD REMOVAL
13532	50542, SG PRIMARY MANWAY HU/CD LIMITS NOT UPDATED IN FSAR AFTER SG'S REPLACED
13534	50542:VCT OUTLET HEADER RELIEF SETPOINT IS NOT CONSISTENT WITH ASSOCIATED HIGH PRESSURE ALARM
M2-96-0017	LTOP PRESSURE ANALYSES MAY BE NON-CONSERVATIVE; PORV ACTUATION LOOP UNCERTAINTY NOT USED (50542)
M2-96-0018	50542: CONTROL ROOM INDICATION ERROR EXCEEDS P-T CURVE'S RCS COLD LEG TEMPERATURE ERROR ASSUMPTION
M2-96-0019	50542: CONTROL ROOM INDICATOR LOOP ERROR EXCEEDS P-T CURVE'S PRESSURIZER PRESSURE ERROR ASSUMPTION

Number	Description
M2-96-0241	50542: QUESTION ABOUT TC SHADOWING FACTOR IN POWER RANGE NEUTRON FLUX SAFETY CHANNEL LOOP CALC
M2-96-0242	50542:QUESTION ABOUT NEUTRON DETECTOR LINEARITY ASSUMPTIONS IN LOOP UNCERTAINTY CALCULATION
M2-96-0252	50542:PAST SIEMENS ANALYSIS DOES NOT JUSTIFY PAST REMOVAL OF STARTUP RATE (SUR) TRIP.
M2-96-0261	BORATING DURING COOLDOWN MAY CONFLICT WITH SOME DESIGN BASIS DOCUMENTS.
M2-96-0267	50542: WIDE RANGE NI HIGH VOLTAGE POWER SUPPLY CAN PRODUCE 2000V - OUTPUT CABLE IS RATED FOR 1000V.
M2-96-0283	50542: ELECTRICAL SEPARATION DISCREPANCIES IN CONTROL BOARD CO6
M2-96-0284	50542: ELECTRICAL SEPARATION DISCREPANCIES IN CONTROL BOARD CO7
M2-96-0289	50542: SPENT FUEL POOL RADMONITORS' INSTRUMENT UNCERTAINTY ALLOWANCES QUESTIONED
M2-96-0296	MSIV CHECK VALVE NON-RETURN FEATURE COULD FAIL CAUSING STEAM GENERATOR BLOWDOWN INTO CONTAINMENT
M2-96-0318	50542: ELECTRICAL SEPARATION DISCREPANCIES IN CONTROL BOARDS C06 AND C07
M2-96-0322	50542: CONTAINMENT RAD MONITOR CALCULATIONS DO NOT ADDRESS INSTRUMENT UNCERTAINTY ALLOWANCES
M2-96-0323	50542: RPS POWER SUPPLY MAY NOT MEET DESIGN REQUIREMENTS
M2-96-0325	50542: MP2 RAD MONITOR MANUAL DOES NOT INCLUDE INSTRUMENT UNCERTAINTY IN SETPOINT CALCULATIONS
M2-96-0329	50542: ESAS ACTUATION MODULE CONFIGURATION DOES NOT MATCH DRAWINGS

Number	Description
M2-96-0351	50542: WR NI OUTPUT TO NI CHART RECORDER NOT PROPERLY ISOLATED FROM POTENTIAL RECORDER FAULT
M2-96-0352	50542: SOME WEIDMULLER TERMINAL BLOCKS DO NOT HAVE A LOCKING WIRE INSTALLED
M2-96-0379	50542: MP2 DOES NOT CONFORM TO LICENSING BASIS FOR BORON DILUTION EVENT
M2-96-0412	50542: POWER SUPPLIES MAY NOT MEET RPS LICENSING & DESIGN BASIS REQUIREMENTS
M2-96-0433	50542: ESAS UNDERVOLTAGE SETPOINT PROCEDURE MAY NOT MEET TECH SPEC SETPOINT CRITERIA
M2-96-0437	Single MSIV closure event puts SG's outside design basis per ASME.
M2-96-0448	50542: SHORT CIRCUIT FAULT MAY CAUSE SUSTAINED UNDER VOLTAGE CONDITION ON VITAL 120VAC
M2-96-0449	50542: PASS TESTING & TRAINING DOES NOT MEET PAST COMMITMENTS
M2-96-0450	50542: AN RBCCW HEAT CALCULATION WAS NOT PUT INTO RECORDS
M2-96-0455	50542: REACTOR COOLANT VENT SYSTEM MAY NOT BE OPERABLE DURING PLANT OPERATION
M2-96-0460	50542: NUMEROUS DISCREPANCIES FOUND ON SHUTDOWN PANELS C21, C09, C10, C70A, C70B
M2-96-0486	50542: SEISMIC IMPACT NOT EVALUATED WHEN COMPONENTS REMOVED FROM MCC'S
M2-96-0507	Aux Steam Unit Heaters 132 & 133 do not match design
M2-96-0542	50542: TECH SPEC REACTOR POWER LIMITS FOR INOP MSSV'S ARE NON-CONSERVATIVE
M2-96-0642	Closure times for CTMT Purge valves in FSAR not same as Tech Specs.
M2-96-0658	TS LCO 3.9.8.2 non-conservative: Second SDC train not required if any (1) of (4) stated conditions satisfied.

Number	Description
M2-96-0662	Changes to relief valve setpoints resulted in two ACR's which failed to identify the Root Cause.
M2-96-0663	Metal blocks used for vibration monitoring were installed without deign documentation or 50.59 evaluation.
M2-96-0670	TSAS 3.9.7, Crane travel - SFP Bldg, & Surveillance Requirement 4.9.7 are not compatible.
M2-96-0761	Issue of Wide Range NI's becoming a Mode 6 hold should have been submitted in a more timely manner.
M2-96-0777	Apparent problem with OP2209A "Refueling Operations" and directions provided on filling the refuel pool as it pertains to applying the Tech Specs.
M2-96-0844	The Spent Fuel Pool Monorail Crane does not have interlocks or surveillance requirements to ensure compliance with Tech Specs
M2-96-0860	CRAC supply damper 2-HV-210 is not single failure proof and could prevent the performance of the safety related function.
M2-96-456	BAST volumes differ from procedure to design basis documents
M2-97-0024	SFP decay heat calculation concerns. The current SFP decay heat load calculation for establishing the max heat load utilized nonbounding assumptions.
M2-97-0083	Containment Spray Pumps RBCCW seal cooler flow of 2gpm dose does not appear adequate. Seal flow may not be adequate to preserve pump performance during post LOCA containment sump recirc.
M2-97-0089	FSAR and TS are in conflict with respect to when AES is required during fuel movement.
MP2-96-0583	Electrical Separation Criteria not consistent between Electrical Specifications, the FSAR, and other regulatory documents.
MP2-96-0613	CTMT Purge Valve testing may not be adequate enough to satisfy Tech Specs

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Attachment 4

Millstone Nuclear Power Station, Unit No. 2

Regulatory Readiness Items

- NRC Open Items
- Required License Amendments
- Required Relief Requests
- Required Applicable Exemptions

February 1997

Attachment 4
Millstone Unit No. 2
Regulatory Readiness Items

Regulatory Readiness Items

1. NRC Open Items

The NRC Open Items shown on the attached are required to be resolved prior to restart of Millstone Unit No. 2 (i.e., prior to entering Mode 4). These items include:

- Escalated Enforcement Items
- Inspection Open Items
- Licensee Event Report (LER) Commitments
- Notice of Violation Response Commitments
- Other NRC docketed correspondence (e.g., commitments made in response to the generic letters and bulletins and other Licensing submittals.

2. Required License Amendments

These include the technical specification changes that are required prior to the startup of Millstone Unit No. 2.

3. Required Relief Requests

At present, there are no known relief requests that require NRC review and approval.

4. Required Applicable Exemptions

At present, there are no known exemptions that require NRC review and approval.

Millstone Nuclear Power Station, Unit 2
Regulatory Readiness Items

Description	A/R Number	Source Document
NRC Open Items		
Escalated Enforcement Items		
Failure to prevent the use of potentially non-conforming material in safety related applications due to lack of adequate MEPL dispositions (EEI 96-201-43)	96031368	NRC IR 96-201
Dynamic testing of steam admission valve (EEI 96-05-09)	96005104	NRC IR 96-05
Failure to prepare a safety evaluation for borating RCS during cooldown to Cold S/D (EEI 96-06-05)	9700198515	NRC IR 96-06
Failure to identify the cause and to take corrective actions for the incorrect RBCCW flow rates used in the CTMT accident analysis (EEI 96-04-10)	9700198508	NRC IR 96-04
Failure to properly implement the requirements of the Technical Specifications during the formation of an ice plug in the SW Strainer backwash line (EEI 95-44-05)	9700198502	NRC IR 95-44
Failure to promptly resolve deficiencies in the analysis and design of MCC enclosures (EEI 96-201-20)	9700198533	NRC IR 96-201
Failure to identify and implement corrective actions concerning CTMT Sump Screen discrepancies (EEI 96-06-11)	9700198517	NRC IR 96-06
Full review and assessment of the function of the PASS, H2 Monitor and CTMT Radmonitors (EEI 96-01-05)	9700198506	NRC IR 96-01
Failure to maintain the plant configuration within the Licensing Bases for the CTMT Gaseous and Particulate RMs, H2 Monitoring system and PASS (EEI 96-201-41)	9700198545	NRC IR 96-201

Description	A/R Number	Source Document
Failure to adequately evaluate the installation of electrical jumpers for a malfunction of a different type as required by 10 CFR 50.59 (EEI 96-201-03)	9700198523	NRC IR 96-201
Failure to update the FSAR which states that the H2 monitoring system is manually initiated in 12 hours versus the 3 hours previously docketed in a letter addressing NUREG 0737 response (EEI 96-08-13)	96033790	NRC IR 96-08
Failure to provide up-to-date data or reflect results of dynamic tests in GL 89-10 Closure Report (EEI 96-05-11)	9700198512	NRC IR 96-05
Identifies the condition where both H2 monitors were inoperable due to insufficient air flow and not taking a representative sample (EEI 96-08-11)	96033783	NRC IR 96-08
Failure to adequately control the installation of a temporary modification to the RBCCW Surge Tank (EEI 96-201-11)	9700198526	NRC IR 96-201
Failure to adequately control standards resulted in inappropriate downgrade of QA Cat 1 equipment to a Non-QA status (EEI 96-201-42)	96031361	NRC IR 96-201
Inadequate corrective actions concerning a seismic design deficiency for a Vital Switchgear room cooler (EEI 96-201-36)	9700198544	NRC IR 96-201
Failure to implement timely corrective action for identified significant conditions adverse to quality (EEI 96-201-30)	9700198539	NRC IR 96-201
Failure to address the SBO issues identified in the VECTRA assessment (EEI 96-201-28)	96004632	NRC IR 96-201
Failure to effectively track and implement corrective actions concerning "dual function" isolation valves (EEI 96-201-25)	9700198535	NRC IR 96-201
Corrective actions for Turbine Bldg crane usage as a heavy load did not adequately prevent the Unit 1 crane from carrying loads over Vital Swgr. (EEI 96-08-10)	97001985	NRC IR 96-08

Description	A/R Number	Source Document
Inadequate corrective action in LER 96-024 (EEI 96-08-08)	97001985	NRC IR 96-08
Change made to OP2305 was inadequate in that it failed to lock open the valves. This was corrective action for the seismic issues associated with the refueling drain line. (EEI 96-08-06)	97001985	NRC IR 96-08
Inadequate design verification for RBCCW Surge Tank seismic restraint modification (EEI 96-201-31)	9700198542	NRC IR 96-201
Failure to take appropriate corrective actions for audit issues involving trending and prioritization of NCR's (EEI 96-201-29)	9700198537	NRC IR 96-201
Failure to establish and maintain measures and design controls to maintain design basis for the WR NI Channels (EEI 96-201-12)	9700198529	NRC IR 96-201
Adequate design control measures not established for verifying the accuracy of the DBDP's (EEI 96-201-09)	9700198524	NRC IR 96-201
Failure to establish adequate design controls when CTMT response analysis was changed during the SGRP (EEI 96-08-12)	96033772	NRC IR 96-08
Failure to adequately establish the qualification of connectors for seven solenoid valves (EEI 96-06-12)	9700198520	NRC IR 96-06
Inspection Open Items		
Deficiencies in establishing tagging boundaries (IFI 95-201-06)	97004014	NRC IR 95-201
Failure to comply with RG 1.97 Commitments (IFI 94-201-06)	97003982	NRC IR 94-201
Failure to follow NEO Procedures 3.03 and 5.11 (IFI 94-201-02)	97003987	NRC IR 94-201
Lack of NCRs and verifying ACR database (URI 95-81-01)	97004029	NRC IR 95-81

Description	A/R Number	Source Document
The site MCP was terminated with no replacement identified. Open items resulting from the previous program are not tracked (IFI 96-08-21)	97004112	NRC IR 96-08
Leakage through letdown isolation valves (IFI 93-20-05)	97003985	NRC IR 93-20
Containment sump screen mesh size (URI 96-06-10)	97004025	NRC IR 96-06
Shutdown Cooling system water hammer investigation (URI 96-06-08)	96030559	NRC IR 96-06
Complete HELB analysis of Turbine Bldg as well as analysis of CST pipe trench (URI 93-19-02)	96232872	NRC IR 93-19
ERT item resolution - Complete the commitments and recommendations identified in the ERT report (URI 95-42-03)	96002502	NRC IR 95-42
EDG fuel oil supply questions (IFI 94-201-90)	97004028	NRC IR 94-201
Review of Cycle 13 Reload Safety Analysis Report (URI 95-27-01)	97003995	NCR IR 95-27
Appropriateness of the SUR Trip removal in 1978 (URI 96-08-14)	96035513	NRC IR 96-08
Loss of DC Bus Event (URI 96-01-04)	97004015	NRC IR 96-01
Core thermal power exceeded (URI 96-06-09)	97003996	NRC IR 96-06
Weakness in procedure classification system (IFI 95-201-03)	97004010	NRC IR 95-201
Numerous GL 89-10 MOV items to be resolved (URI 95-001-01)	97004004	NRC IR 95-01
Use of locked rotor current in degraded voltage calculations ((URI 96-05-10)	96029543	NRC IR 96-05

Description	A/R Number	Source Document
Refueling Pool drain line seismic issues (URI 96-06-07)	97003989	NRC IR 96-06
Resolution of adequacy of the as-left RBCCW flow rates (URI 96-04-11)	97002641	NRC IR 96-04
Licensee Event Report (LER) Commitment		
Corrective Actions Recommended by Heatup ERT -	96005424	LER 96-007-00
LER 95-019-02: Perform Analysis	97002778	LER 95-019-02
LER 96-012-00-SOV Operability (Mode 4)	96005447	LER 96-012-00
LER 95-019-02: Evaluate 79-02 & 79-14 Reviews	97002778	LER 95-019-02
LER 95-019-03: Return RWST Piping to full design compliance	97002778	LER 95-019-03
LER 96-019-00, SOV Circuit Full Review (Mode 2)	96005591	LER 96-019-00
LER 96-019-00 SOV Circuits Configuration Reviews (Mode 4)	96005591	LER 96-019-00
LER 96-019-00, EEQ SOV Circuits in Containment (Mode 4)	96005591	LER 96-019-00
LER 96-019-00 SOV Circuit Modifications (Mode 4)	96005591	LER 96-019-00
LER 96-020-00 DGN. Chng. for Loss of RBCCW (LER 96-020-01) (N) (M4)	96005922	LER 96-020-00
LER 96-019-00, SOV Circuit SFR Review (Mode 2)	96005591	LER 96-019-00
LER 96-022-00-Dgn Chg. to Correct sgl. Fail. Susp.- EBFS HTRs(SU)	96007508	LER 96-022-00

Description	A/R Number	Source Document
LER 96-016-00 CWP Trip Function - Cond. Pit Level Switches	96004898	LER 96-016-00
Supplement LER 96-031-00, re-analysis of single MSIV Closure	96032859	LER 96-031-00
Submit Supplement to LER 96-003-00 - (heat up and cool down report)	96002914	LER 96-003-00
Submit Supplement LER - 96-032	96033653	LER 96-032-00
LER 96-041-00: Develop Procedure for Fuel Transfer Tube Flange	97001426	LER 96-041-00
LER 96-029-00- SUR Trip Analysis	96030253	LER 96-029-00
LER 96-007-00 Heatup/Cooldown Corrective Actions (N)	96003852	LER 96-007-00
LER 96-018-00 Gaps in Enclosure Door Seals	96030988	LER 96-018-00
LER 96-030-00 Verify and Correct Bases and Surveillances	96031106	LER 96-030
LER 96-009-00 Complete Prior to Startup (N)	96003567	LER 96-009-00
LER 96-009-00 Post LOCA CTMT Press & Temp Curve Review (S/U)(N)	96003567	LER 96-009-00
Operating Guidance and Operator Burdens From ERT Heat Up	96005424	LER 96-007-00
LER 96-028-00 SW Strainer Backwash Solenoid Install. (Mode 4)	96027503	LER 96-028-00
LER 96-012-00 - TS ESFAS Response Time (N)	96005447	LER 96-012-00
LER 96-029-00 Prepare Suppl. LER-SUR Trip Analy. (Mode 4)	96030253	LER 96-029-00

Description	A/R Number	Source Document
LER 96-018-00 MCC B51/B61 Enclosure Door Seals	96030988	LER 96-018-00
LER 96-007-00 Heatup/cooldown Rate Proc. Changes & Train. (Mode4)	96003852	LER 96-007-00
LER 97-001-00, Replace Horncock Fire Seal Material	97002940	LER 97-001-00
LER 96-043-00 Review of Design Calibration Calculations	97002697	LER 96-043-00
Re-analysis of single MSIV closure event, LER 96-031-00	96032859	LER 96-031-00
Notice of Violation Response Commitment		
ATWS system testing (VIO 96-01-06)	96008733	NRC IR 96-01
Inadequate corrective action (VIO 95-11-01)	95025539	NRC IR 95-11
Inadequate retest of solenoid valves (VIO 96-04-08)	96027036	NRC IR 96-04
Operation of Service Water and RBCCW cooling systems outside of their design basis during condenser backwashing (VIO 94-17-10)	96032418	NRC IR 94-17
Failure to address the leaking of check valve 2-CS-15A and to take action to assess the impact of water being present on the sump side of 2-CS-16.1A (95-08-01)	97004029	NRC IR 95-08
Potential violation of T.S. which requires two separate and independent ECCS subsystems due to inadequate pressure locking/thermal binding susceptibility evaluation (95-08-02)	97004026	NRC IR 95-08
EOP Procedure Upgrade NOV 94-17-10	96032418	NRC IR-94-17
Failure to document review of Vendor QA design services IAW NU Procedures (95-08-04)	97004024	NRC IR 95-08

Description	A/R Number	Source Document
Intake structure ventilation system deficiencies (VIO 95-42-04)	97003992	NRC IR 95-42
Failure to adequately evaluate information on pressure locking and thermal binding IAW PIR procedure (95-08-03)	97004030	NRC IR 95-08
Other NRC Docketed Correspondence		
Complete review required by Generic Letter 96-01	96033301	GL 96-01
Complete 110X/Y MODS For Instrumentation Loop Class 1E Power	95026372	B15100
Mode 2 - Review Tech Spec Surveillance Procedures	96037089	B16076
10CFR 50.54(F) for MP2	96003732	A12753
Self-assessment of ESAS related issues and deficiencies - B15283	95027429	B15283
Develop Alternate means of Appendix R compliance	96036400	B16065
Effectiveness of Corrective Actions Program - A12863	96008778	A12863
Reference TRM in the FSAR Chapter 12 (N) - B15702	96007779	B15702
Thermo-lag Resolution - Fire Rating Evaluations - B15558	96005888	B15558
Provide Training Regarding GL 96-01	96035429	GL 96-01
Reevaluation & Modification of SG Hydraulic Snubbers - B16019	96034250	B16019
Submit the Revised LBLOCA Analysis to the NRC	96036408	A13114

Description	A/R Number	Source Document
Review design processes and take appropriate actions - B16019	96034250	B16019
Ensure implementation of the ICAVP	96032302	A12884
Thermo-Lag Resolutions - Appendix "R" Re-analysis	96005888	B15558
GL 96-06 Actions to Eliminate Thermally Induced Overpressure	97002555	GL 96-06
Owners Groups' RPV Integrity Program Results - A12931	96027793	A12931
Complete Resolution of Seismic Outliers Prior to Startup - GL 87-02	97001885	GL87-02
Modify RM 8123A/B & RM-8262 A/B Instrume[illegible]	97001960	B16159
Generic Letter 96-06 RBCCW Calculations	97002555	GL 96-06
Revise Procedures to Ensure Continued Compliance with GL 96-01	96035429	GL 96-01
Generic Letter 96-06 Actions to mitigate RBCCW Waterhammer	97002555	GL 96-06
Operation of Thermolag Cables not Meeting Design Ampacity Requirement	96036400	B16065
Mode 1 - Review Tech Spec Surveillance Procedure	96037089	B16076
Mode 6 - Review Tech Spec Surveillance Procedure	96037089	B16076
Mode 5 - Review Tech Spec Surveillance Procedures	96037089	B16076
Mode 4 - Review Tech Spec Surveillance Procedures	96037089	B16076

Description	A/R Number	Source Document
Mode 3 - Review Tech Spec Surveillance Procedure	96037089	B16076
Thermo-Lag Resolutions - Appendix "R" Full Compliance	96005888	B15558
Thermo-Lag Resolutions - Select Product Upgrades	96005888	B15558
Request for Additional Inform. Related to 50.54(f) Letter	96007039	A12852
Steam Generator Wide Range Level Incorpor into EOP	96005888	B15100
Resolution of 59 design issues (ACR 8761)	96001273	ACR 8761

Description	A/R Number	Source Document
Required License Amendments		
Revise SG Safety valve setpoints in Table 3.7-1	-	-
Section 6.0, Organization Structures/ Titles	-	-
Steam Generator Blowdown Monitor Action statement changes - Section 3.3.3.9	-	-
MET Tower Instrument Accurracies - Section 3.3.3.4	-	-
Ultimate Heat Sink Temperature measurement location changes - Section 3.7.11	-	-
Enclosure Building Integrity and application of Specification 3.0.3 - Sections 3.6.5.1 and 3.6.5.2	-	-
Fuel oil capacity (7 day requirement) for Emergency Diesel Generator - 3.8.1	-	-
Revise Pressure/Temperature in Specification 3.4.9.1	-	-
Control Room Emergency Ventilation System - Surveillance Requirements - Section 3.7.6.1	-	-
Spent fuel storage area ventilation system - Section 3.9.14	-	-
Containment Integrity / Containment Isolation Valves - Section 3.6.3	-	-
Reinstate a note to Tech Spec related DNB LCO - Section 3.2.6	-	-
LTOP condition (specification 3.4 - 9.3), Reinstate the applicability of specification 3.0.4	-	-

Description	A/R Number	Source Document
Addition of 2 second time delay to 4.16 kv under voltage Level 1, Table 3-3-4	-	-
RCS - Mode 4 and 5. Action statement changes - Section 3.4.1.3	-	-

Description	A/R Number	Source Document
Required Relief Requests		
None identified at this time	-	-

Description	A/R Number	Source Document
Required Applicable Exemptions		
None identified at this time	-	-

Docket No. 50-336
B16257

Attachment 5

Millstone Nuclear Power Station, Unit No. 2

Common Millstone Site Issues Applicable to
Millstone Unit No. 2

February 1997

Attachment 5
Millstone Nuclear Power Station, Unit No. 2
Common Millstone Site Issues Applicable to
Millstone Unit No. 2

The following list includes Common Millstone Site Issues that are applicable to Millstone Unit No. 2:

- Employee Concerns Program
- Fire Protection Program
- HELB/EEQ
- MOV Program, GL 89-10
- Dual Function Valve Control and Testing
- Review of NUREG-0737 Action Items
- Control of Safeguard Information
- Procedural Adherence
- Procedure Upgrade Program
- Communication Between Units
- MEPL Program Evaluation'
- Operator Training
- Formality of Non-Routine Security Activities
- Control and Use of Vendor Information
- Work Planning and Controls, Engineering and Maintenance Backlogs
- Work Planning and Controls, Automated Work Order Process
- Oversight Effectiveness of Each Group
- Emergency Preparedness Effectiveness

Each of the above issues will be addressed by one or more specific actions and each issue will be dispositioned prior to restart of Millstone Unit No. 2.

Attachment 6

Millstone Nuclear Power Station, Unit No. 2

Operational Readiness Plan - Nuclear Oversight Plan

February 1997