

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1) Millstone Point Unit 2										DOCKET NUMBER (2) 0 5 0 0 0										PAGE (3) 1 OF 02																															
TITLE (4) Degradation of Steam Generator Tubing																																																			
EVENT DATE (5)						LER NUMBER (6)						REPORT DATE (7)						OTHER FACILITIES INVOLVED (8)																																	
MONTH			DAY			YEAR			YEAR			SEQUENTIAL NUMBER			REVISION NUMBER			MONTH			DAY			YEAR			FACILITY NAMES						DOCKET NUMBER(S)																		
0 3			1 4			8 5			8 5			0 0 4			0 1			0 9			0 6			8 5									0 5 0 0 0																		
0 3			1 4			8 5			8 5			0 0 4			0 1			0 9			0 6			8 5									0 5 0 0 0																		
OPERATING MODE (9)		THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR § (Check one or more of the following) (11)																																																	
6		20.402(b)										20.406(e)										80.73(a)(2)(iv)										73.71(b)																			
POWER LEVEL (10)		0 0 0										20.406(a)(1)(i)										80.36(a)(1)										80.73(a)(2)(v)										73.71(c)									
		20.406(a)(1)(ii)										80.36(a)(2)										80.73(a)(2)(vi)										OTHER (Specify in Abstract below + in Text, NRC Form 366A)																			
		20.406(a)(1)(iii)										80.73(a)(2)(i)										80.73(a)(2)(vii)(A)																													
		20.406(a)(1)(iv)										80.73(a)(2)(ii)										80.73(a)(2)(vii)(B)																													
		20.406(a)(1)(v)										80.73(a)(2)(iii)										80.73(a)(2)(ix)																													
LICENSEE CONTACT FOR THIS LER (12)																																																			
NAME R. T. Blanchard, Jr.										TELEPHONE NUMBER AREA CODE 2 0 3 4 4 7 - 1 7 9 1																																									
COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)																																																			
CAUSE	SYSTEM	COMPONENT	MANUF. TURER	REPORTABLE TO NPROS		CAUSE	SYSTEM	COMPONENT	MANUF. TURER	REPORTABLE TO NPROS																																									
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SUPPLEMENTAL REPORT EXPECTED (14)										EXPECTED SUBMISSION DATE (15)																																									
YES (If yes, complete EXPECTED SUBMISSION DATE)										X NO																																									

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

While in Mode 6 (Refueling Mode) scheduled inservice examination of steam generator tubing identified that sufficient numbers of tubes in each steam generator had flaws greater than 40 percent through-wall to place the steam generators in Category C.3 of Technical Specification Table 4.4-6. The inspections identified that the flaws were limited to the area of the tube between the top of the tubesheet and the first tube support.

As required by Technical Specification Table 4.4-6 and Technical Specification 4.4.5.1.2 tubing examinations were continued until 100% of the tubes had been inspected in the region where flaws were identified. All defective tubes were repaired in accordance with Technical Specification 4.4.5.1.4.b.

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LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

APPROVED OMB NO 3150-0104

EXPIRES 8/31/85

FACILITY NAME (1)	DOCKET NUMBER (2)	LER NUMBER (6)			PAGE (3)	
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER		
Millstone Point Unit 2	0 5 0 0 0 3 3 6 8 5	-	0 0 4	-	0 1	0 2 OF 0 2

TEXT (If more space is required, use additional NRC Form 365A's) (17)

While in Mode 6 (Refueling Mode) scheduled inservice examination of steam generator tubing identified that sufficient numbers of tubes in each steam generator had flaws greater than 40 percent through-wall to place the steam generators in Category C.3 of Technical Specification Table 4.4-6. The inspections identified that the flaws were limited to the area of the tube between the top of the tubesheet and the first tube support.

As required by Technical Specification Table 4.4-6 and Technical Specification 4.4.5.1.2 tubing examinations were continued until 100% of the tubes had been inspected in the region where flaws were identified.

The flaws have been characterized as small volume pits originating in the sludge pile region of the steam generator. This type defect has been identified in each of the two preceding inspections and was anticipated in this examination. Actions to minimize the flaw progression including sludge lancing and chemical cleaning of the sludge pile region have been taken.

During examination of some tubes, distorted eddy current signals were observed. Testing on removed tube samples following the 1983 refueling shutdown indicated that these distorted signals could be related to or mask flaws. Since the extent of through-wall penetration could not be accurately quantified for these tubes they were considered defective and plugged or sleeved as appropriate. Steam Generator No. 1 had 464 tube ends with distorted signals. Steam Generator No. 2 had 97 tube ends with distorted signals.

Repair actions were taken in accordance with Technical Specification 4.4.5.1.4.b. as follows:

Steam Generator No. 1 - 23 tubes plugged
Hot Leg Plenum - 591 tube ends sleeved
Cold Leg Plenum - 1116 tube ends sleeved

Steam Generator No. 2 - 40 tubes plugged
Hot Leg Plenum - 143 tube ends sleeved
Cold Leg Plenum - 1068 tube ends sleeved

In accordance with Technical Specification 4.4.5.1.4.b. the steam generators were declared OPERABLE following completion of these actions.

NORTHEAST UTILITIES



THE CONNECTICUT LIGHT AND POWER COMPANY
WESTERN MASSACHUSETTS ELECTRIC COMPANY
HOLYOKE WATER POWER COMPANY
NORTHEAST UTILITIES SERVICE COMPANY
NORTHEAST NUCLEAR ENERGY COMPANY

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September 6, 1985
MP-8163

U. S. Nuclear Regulatory Commission
Document Control Desk
Washington, D.C. 20555

Reference: Facility Operating License No. DPR-65
Docket No. 50-336
Reportable Occurrence RO 50-336/85-004-01

Gentlemen:

This letter forwards the Licensee Event Report 85-004-01 originally required to be submitted within thirty (30) days pursuant to paragraph 50.73 (a) (2) (ii), a condition that resulted in a principle safety barrier being degraded.

Yours truly,

NORTHEAST NUCLEAR ENERGY COMPANY

A handwritten signature in dark ink, appearing to read 'Wayne D. Romberg'.

Wayne D. Romberg
Station Superintendent
Millstone Nuclear Power Station

WDR/TB:ejl

Attachment: LER 50-336/85-004-01

cc: Dr. T. E. Murley, Region I

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