



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

Docket File
50-265

December 3, 1996

Ms. Irene Johnson, Acting Manager
Nuclear Regulatory Services
Commonwealth Edison Company
Executive Towers West III
1400 Opus Place, Suite 500
Downers Grove, IL 60515

SUBJECT: EVALUATION OF THE AUGMENTED EXAMINATION OF THE REACTOR PRESSURE
VESSEL SHELL WELDS PURSUANT TO 10 CFR 50.55a(g)(6)(ii)(A) FOR QUAD
CITIES NUCLEAR POWER STATION, UNIT 2 (TAC NO. M96937)

Dear Ms. Johnson:

The staff has reviewed the information provided by Commonwealth Edison Company (ComEd, the licensee) in its letter dated October 10, 1996, related to the augmented examination of the reactor pressure vessel shell welds pursuant to 10 CFR 50.55a(g)(6)(ii)(A) for Quad Cities Nuclear Power Station, Unit 2. Our evaluation follows.

In its letter dated October 10, 1996, the licensee proposed that the augmented reactor pressure vessel shell weld (American Society of Mechanical Engineers (ASME) Boiler and Pressure Vessel Code (Code), Section XI, Category B-A, Item B1.10 shell welds) examinations be ultrasonically examined (a) remotely from the inside of the reactor vessel to the extent possible, and (b) manually from the outside of the vessel where remote inside examination is not achieved and where scheduled bioshield wall blocks and insulation materials disassembly allows access.

In addition, the licensee proposed that the remote examinations will be performed in accordance with the remote ultrasonic examination procedure developed by General Electric, which has been demonstrated at the Performance Demonstration Initiative Qualification in accordance with the rules of Appendix VIII of the ASME Code, Section XI, 1992 Edition through 1993 Addenda. Based on the results of the Quad Cities, Unit 2, reactor vessel accessibility study performed by the licensee before the installation of the core shroud repair hardware, it concluded that the examination coverage from the inside of the vessel would be approximately 68.7 percent of the reactor pressure vessel shell weld volume. Quad Cities does not intend to remove any shroud repair hardware to perform the remote internal examination. Therefore, the reactor pressure vessel shell weld volume coverage, with the shroud repair hardware installed, is approximately 65.1 percent. Furthermore, as noted above, the licensee will supplement the remote internal examinations with manual ultrasonic examinations from the outside surface of the reactor pressure vessel to the extent possible.

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The staff believes that the licensee's proposed examination plan to meet the augmented reactor pressure vessel shell weld inspection requirements contained in 10 CFR 50.55a(g)(6)(ii)(A), is appropriate. Upon completion of the augmented reactor vessel shell welds examinations, if the examination volume for each weld examined is less than 90 percent, the licensee is to submit to the NRC an alternative to its augmented reactor pressure vessel shell weld examinations pursuant to 10 CFR 50.55a(g)(6)(ii)(A).

Sincerely,

Original signed by:

Robert M. Pulsifer, Project Manager
Project Directorate III-2
Division of Reactor Projects - III/IV
Office of Nuclear Reactor Regulation

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cc: See next page

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